

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	PAGE (3) 1 0 F 0 2
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TITLE (4)
MISSED IN-SERVICE INSPECTION TEST ON SHUTDOWN COOLING HEAT EXCHANGER VALVES

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
0 7	1 2	8 4	8 4	0 3 9	0 0	0 8	1 3	8 4	UNIT 3		
									DOCKET NUMBER(S) 0 5 0 0 0 3 6 2		
									0 5 0 0 0 1 1 1		

OPERATING MODE (9) 4

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(x)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME J. G. HAYNES, STATION MANAGER	TELEPHONE NUMBER AREA CODE 7 1 4 4 9 2 1 - 7 7 0 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/12/84, with Unit 2 in Mode 4 and Unit 3 in Mode 1 at 75% power, it was determined that the In-service Inspection Test (IST) of the Unit 2 and 3 Component Cooling Water Outlet Valves (HV-6500 and HV-6501) was missed. Technical Specification 4.0.5 requires testing pursuant to ASME Boiler and Pressure Vessel Code, Section XI, which results in testing these valves every Cold Shutdown, but not more frequently than every 92 days. Upon performing the required IST, these valves were demonstrated to have been operable. The valves had previously tested satisfactorily on 10/19/83 and 12/29/83 for Units 2 and 3, respectively.

The cause was administrative oversight, in that IST requirements for HV-6500 and HV-6501 were not properly transferred from Operating Instructions S02(3)-3-3.30 to S02(3)-3-3.31, as intended. As corrective actions, these valves have been included in S02(3)-3-3.31 and all procedural controls for valve IST have been validated.

Since testing of these valves confirmed their operability, there are no reasonable or credible alternative conditions under which this event would have been more severe.

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PDR ADOCK 05000361
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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQ. NUMBER	REV. NUMBER			
		8 4	- 0 3 9	- 0 0	0 2	OF	0 2

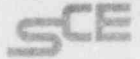
TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 7/12/84, with Unit 2 in Mode 4 and Unit 3 in Mode 1 at 75% power, it was determined that the In-service Inspection Test (IST) of the Unit 2 and 3 Component Cooling Water Outlet Valves (HV-6500 and HV-6501) (EIIS System Code HCV) from the Shutdown Cooling Heat Exchangers (EIIS System Code HX) was missed. Technical Specification 4.0.5 requires testing pursuant to ASME Boiler and Pressure Vessel Code, Section XI, which results in testing these valves every Cold Shutdown, but not more frequently than every 92 days. Upon performing the required IST, these valves were demonstrated to have been operable. The valves had previously tested satisfactorily on 10/19/83 and 12/29/83 for Units 2 and 3, respectively.

The cause was administrative oversight in that the IST requirements for HV-6500 and HV-6501 were not properly transferred from Operating Instructions S02(3)-3-3.30, "In-service Valve Test - Quarterly," under which these valves were previously tested, to S02(3)-3-3.31, "In-service Valve Test - Cold Shutdown." The procedure transfer was necessary to require IST during Cold Shutdown, Mode 5, because an IST could impair normal operation in other modes. As corrective actions, these valves have been included in S02(3)-3-3.31 and all procedural controls for valve IST have been validated.

Since testing of these valves confirmed their operability, there is no reasonable or credible alternative condition under which this event would have been more severe.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128
SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

August 13, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
Licensee Event Report No. 84-039
San Onofre Nuclear Generating Station, Units 2 and 3

Pursuant to 10 CFR 50.73(a)(2)(i)(B), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Component Cooling Water System. Since this occurrence involved the same components on Units 2 and 3, a single LER for Unit 2 is enclosed per NUREG-1022. Neither the health and safety of plant personnel nor the public were affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 84-039

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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