

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE August 16, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: JULY, 1984.
- 3. LICENSED THERMAL POWER (MWT): 2535.
- 4. NAMEPLATE RATING (GROSS MWE): 871.
- 5. DESIGN ELECTRICAL RATING (NET MWE): 819.
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

- 8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_
- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) \_\_\_\_\_
- 10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	744.	5111.	86904.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	35.9
20. UNIT AVAILABILITY FACTOR	0.0	0.0	35.9
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	35.1
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	33.5
23. UNIT FORCED OUTAGE RATE	100.0	100.0	60.9

- 24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH) \_\_\_\_\_
- 25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

8408240066 840731  
 PDR ADOCK 05000289  
 R PDR

TE24  
 111

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE August 16, 1984  
COMPLETED BY C. W. Smyth  
TELEPHONE (717) 948-8551

MONTH: JULY

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.
2	0.
3	0.
4	0.
5	0.
6	0.
7	0.
8	0.
9	0.
10	0.
11	0.
12	0.
13	0.
14	0.
15	0.
16	0.

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
17	0.
18	0.
19	0.
20	0.
21	0.
22	0.
23	0.
24	0.
25	0.
26	0.
27	0.
28	0.
29	0.
30	0.
31	0.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1984

DOCKET NO. 50-289  
 UNIT NAME TMI-I  
 DATE August 16, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

No.	Date	Type <sup>1</sup>	Duration (hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	84-01-08	F	744	D	:	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

## OPERATING SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The RCS was partially drained the entire month to enable investigations in OTSG tube leakage. These efforts and findings were discussed in detail in our letter to Mr. John F. Stolz, dated July 18, 1984.

## MAINTENANCE SUMMARY

### Major Safety Related Maintenance

The following major maintenance items were completed during the month:

#### OTSG A & B

Work accomplished consisted of the completion of Eddy Current Testing, Fiberoptic Inspection of tube #148-35 ("A" OTSG), and removal of the lower manway on the "A" OTSG. Repairs on the "A" OTSG included welding plugs in the upper tubesheet in tubes #135-72 and #148-35. Repairs on the "B" OTSG included welding plugs in the upper tubesheet in tubes #70-8, #79-41, and #80-45. Explosive plugs were also installed in the "B" OTSG lower tubesheet in tubes #70-8, #77-7, #79-41, and #80-45. The upper and lower tubesheets were mapped utilizing photographs for all plugged tubes in the upper and lower tubesheets of both the "A" and "B" OTSGs. Further inspections and testing of plugged tubes will continue in August.

#### Concentrated Waste Storage Tank Pump (WDL-P12 A/B)

WDL-P-12A was removed from the system, disassembled, repaired, reassembled, and reinstalled to correct seal leakage. A section of WDL-P12B suction piping, suction valve WDL-V-175, and suction pipe flange, were replaced due to deterioration of piping. Repairs to WDL-P-12B will continue in August to correct a cracked pump casing.

#### Fire Service Pump Diesel (FS-P-3)

To correct oil leakage on FS-P-3 diesel engine resulting from the #4 injector adjusting screw locknut loosening and cracking the valve cover, the following corrective action was taken:

The rocker box was removed from the diesel. The cam followers were removed and inspected. Three cam followers were replaced, the #4 injector adjusting screen locknut was replaced, 1 valve pushrod was replaced, and a new valve cover was installed. The diesel engine was satisfactorily timed and run. All work has been completed.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1982 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE August 16, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: JULY, 1984.
- 3. LICENSED THERMAL POWER (MWT): 2535.
- 4. NAMEPLATE RATING (GROSS MWE): 871.
- 5. DESIGN ELECTRICAL RATING (NET MWE): 819.
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 824.
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 776.

NOTES

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

---

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) \_\_\_\_\_

10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

---



---

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	744.	5111.	86904.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	35.9
20. UNIT AVAILABILITY FACTOR	0.0	0.0	35.9
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	35.1
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	33.5
23. UNIT FORCED OUTAGE RATE	100.0	100.0	60.9

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)

---

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-1  
DATE August 16, 1984  
COMPLETED BY C. W. Smyth  
TELEPHONE (717) 948-8551

MONTH: JULY

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.
2	0.
3	0.
4	0.
5	0.
6	0.
7	0.
8	0.
9	0.
10	0.
11	0.
12	0.
13	0.
14	0.
15	0.
16	0.

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
17	0.
18	0.
19	0.
20	0.
21	0.
22	0.
23	0.
24	0.
25	0.
26	0.
27	0.
28	0.
29	0.
30	0.
31	0.

**UNIT SHUTDOWNS AND POWER REDUCTIONS**

REPORT MONTH July, 1984

DOCKET NO. 50-289  
 UNIT NAME TMI-I  
 DATE August 16, 1984  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	84-01-08	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Component Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source



## OPERATING SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The RCS was partially drained the entire month to enable investigations in OTSG tube leakage. These efforts and findings were discussed in detail in our letter to Mr. John F. Stolz, dated July 18, 1984.

## MAINTENANCE SUMMARY

### Major Safety Related Maintenance

The following major maintenance items were completed during the month:

#### OTSG A & B

Work accomplished consisted of the completion of Eddy Current Testing, Fiberoptic Inspection of tube #148-35 ("A" OTSG), and removal of the lower manway on the "A" OTSG. Repairs on the "A" OTSG included welding plugs in the upper tubesheet in tubes #135-72 and #148-35. Repairs on the "B" OTSG included welding plugs in the upper tubesheet in tubes #70-8, #79-41, and #80-45. Explosive plugs were also installed in the "B" OTSG lower tubesheet in tubes #70-8, #77-7, #79-41, and #80-45. The upper and lower tubesheets were mapped utilizing photographs for all plugged tubes in the upper and lower tubesheets of both the "A" and "B" OTSGs. Further inspections and testing of plugged tubes will continue in August.

#### Concentrated Waste Storage Tank Pump (WDL-P12 A/B)

WDL-P-12A was removed from the system, disassembled, repaired, reassembled, and reinstalled to correct seal leakage. A section of WDL-P12B suction piping, suction valve WDL-V-175, and suction pipe flange, were replaced due to deterioration of piping. Repairs to WDL-P-12B will continue in August to correct a cracked pump casing.

#### Fire Service Pump Diesel (FS-P-3)

To correct oil leakage on FS-P-3 diesel engine resulting from the #4 injector adjusting screw locknut loosening and cracking the valve cover, the following corrective action was taken:

The rocker box was removed from the diesel. The cam followers were removed and inspected. Three cam followers were replaced, the #4 injector adjusting screen locknut was replaced, 1 valve pushrod was replaced, and a new valve cover was installed. The diesel engine was satisfactorily timed and run. All work has been completed.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1988 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



GPU Nuclear Corporation  
Post Office Box 480  
Route 441 South  
Middletown, Pennsylvania 17057-0191  
717 944-7621  
TELEX 84-2386  
Writer's Direct Dial Number:

August 16, 1984  
5211-84-2167

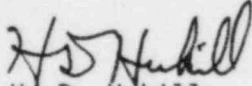
Office of Management Information and Program Control  
Attn: W. C. McDonald  
c/o Distribution Services Branch DPC, ADM  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. McDonald:

Three Mile Island Nuclear Station, Unit I (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
July Monthly Operating Report

Enclosed please find two (2) copies of the July Operating Report for Three Mile Island Nuclear Station Unit.

Sincerely,

  
H. D. Hukill,  
Director, TMI-1

HDH:vjf  
Attachments

cc: V. Stello  
Dr. T. E. Murley