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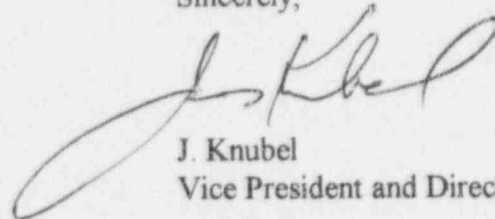
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for November 1995

Enclosed are two copies of the November 1995 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,



J. Knubel
Vice President and Director, TMI

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector
T95001

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OPERATIONS SUMMARY
November 1995

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 820 MWe during November.

MAJOR SAFETY RELATED MAINTENANCE

The following is a summary of major safety related maintenance items accomplished during the month:

Fire Service Pump Check Valve FS-V-27

Fire Service Pump FS-P-3 Discharge Check Valve FS-V-27 was inspected for Microbiologically Induced Corrosion inspection by Plant Engineering. The valve internals and piping were accessed through the valve body to bonnet mechanical connection. This inspection was a follow-up to an inspection performed in 1994. Ultrasonic testing was performed to evaluate any variation in valve body material thickness since the last inspection and samples were taken for microbiological study. Upon completion of the inspection, the valve was reassembled and returned to service. The results of the inspection will be evaluated after the microbiological data has been received.

RC-P-1D #1 Seal Leakoff Flow Transmitter MU-9-FT4

Reactor Coolant Pump "D" #1 seal leakoff flow transmitter, MU-9-FT4, failed. A replacement transmitter assembly with amplifier and cable was obtained from the warehouse and installed during a Reactor Building entry. During this work, a tubing leak on MU-9-FE4 was corrected. In place checks were performed on the new transmitter prior to returning it to service and the results were satisfactory. Inspection of the old transmitter revealed a reddish substance on the cross hair divider and on the flow meter. During the next Reactor Building entry, scheduled for December, inspections are being planned for MU-9-FT2, Reactor Coolant Pump "B" #1 seal leakoff transmitter for comparison.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPEFATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: NOVEMBER 1995
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 872
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	720.0	8016.0	186241.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	720.0	7210.4	108799.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	720.0	7182.2	107637.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1847111	18289296	264205567
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	624698	6124586	88807840
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	590553	5777793	83433211
19. UNIT SERVICE FACTOR	(%)	100.0	89.6	57.8
20. UNIT AVAILABILITY FACTOR	(%)	100.0	89.6	57.8
21. UNIT CAPACITY FACTOR (USING MDC NET)		104.4	91.7	57.0
22. UNIT CAPACITY FACTOR (USING DER NET)		100.1	88.0	54.7
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	36.0
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (77) 948-8191

MONTH: NOVEMBER

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	813	17	825
2	804	18	823
3	808	19	821
4	822	20	821
5	823	21	821
6	824	22	823
7	822	23	823
8	819	24	823
9	822	25	825
10	820	26	821
11	811	27	819
12	822	28	814
13	824	29	824
14	822	30	825
15	821	31	NA
16	824		

REPORT MONTH November 1995

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code ⁴ & ⁵	Component Code ³ & ⁴	Cause & Corrective Action to Prevent Recurrence
						None			

1
 F Forced
 S Scheduled

2
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit G - Instructions for preparation of Dca Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 12, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **NA.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **NA**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **None**
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **683**
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.