

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	PAGE (3) 1 OF 0 2
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TITLE (4)
Potential Unreviewed Safety Question

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	1 9	8 4	8 4	0 1 2	0 0	0 8	1 7	8 4			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 b 1 0	20.402(b)		20.406(e)		80.73(a)(2)(iv)		73.71(b)			
	20.406(a)(1)(i)		80.38(c)(1)		80.73(a)(2)(v)		73.71(c)			
	20.406(a)(1)(ii)		80.38(c)(2)		80.73(a)(2)(viii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
	20.406(a)(1)(iii)		80.73(a)(2)(i)		80.73(a)(2)(viii)(A)					
	20.406(a)(1)(iv)		80.73(a)(2)(ii)		80.73(a)(2)(viii)(B)					
20.406(a)(1)(v)		80.73(a)(2)(iii)		80.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (14)	
NAME John J. Anderson	TELEPHONE NUMBER AREA CODE: 9 1 4 7 3 9 - 8 2 0 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 12, 1984, Westinghouse advised the New York Power Authority (NYPA) and several other utilities that a potential overpressure condition in the Component Cooling Water (CCW) system could occur during periods of system inleakage and heat load increases. After a thorough review of the information presented by Westinghouse, NYPA determined that a potential unreviewed safety question was involved. The NRC was notified within one hour of NYPA's determination. Consistent with Westinghouse's recommendation, a change to plant equipment was installed immediately to alleviate the potential overpressure concern.

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PDR ADOCK 05000286
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 8 4 — 0 1 2 — 0 0	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	— 0 1 2	— 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On July 12, 1984, Westinghouse advised the New York Power Authority (NYPA) and several other utilities of a potential overpressure condition in the Component Cooling Water (CCW) system designed by Westinghouse. This could be caused by increases in CCW system inventory.

The sequence of postulated events would be initiated by a tube rupture in an RCP thermal barrier heat exchanger. Reactor Coolant System (RCS) leakage to the CCW system then occurs via line 21. Flow Control Valve FCV-625 would normally close upon sensing high flow on this line but is assumed to remain open, constituting the "single failure". Radiation monitors R17A and R17B would detect the contamination entering the CCW system. These monitors would generate a "close" signal to the CCW surge tank vent valves (RCV-017A and RCV-017B) in order to prevent an escape of radioactive gases to the Primary Auxiliary Building (PAB). As the vent valves closed, the pressure would begin to increase in the CCW system. The relief valves (nos. 835A and 835B) on the CCW surge tanks would open at 125 psig. Westinghouse has reported that a system overpressurization of up to 170 percent downstream of the CCW pumps could then occur due to pump developed heads.

NYPA reviewed the Westinghouse scenario and determined that the potential overpressure condition was only valid for the CCW pump seals and for some heat exchangers supplied by the CCW system. Analysis has found that the ANSI temperature/pressure rating of the CCW system piping, including valves and fittings, would not have been exceeded during this "worst case" transient. Furthermore, Westinghouse did not acknowledge the existence of relief valves elsewhere in the CCW system which would mitigate the overpressurization concern. On July 19, 1984, NYPA determined that the postulated overpressure condition of the CCW system represented an potential unreviewed safety question. At that time, a one-hour telephone notification was made to the NRC as per the requirements of 10CFR50.72(b)(1)(ii)(B).

In order to relieve the potential overpressure concern, NYPA immediately implemented changes to plant equipment consistent with the recommendations made by Westinghouse. The internals of relief valves 835A and 835B were removed, allowing continuous venting of the surge tanks to the liquid waste system. In addition, the vent valves RCV-017A and 017B were blocked closed to prevent a direct gaseous path from the liquid waste system to the PAB. An engineering analysis of this potential overpressure concern is continuing.

No similar events have been reported to date. Reactor performance was not affected by this postulated incident. This event is reportable under 10CFR50.73(a)(2)(ii)(B) which became effective January 1, 1984.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200



August 17, 1984
IP-FWG-3126

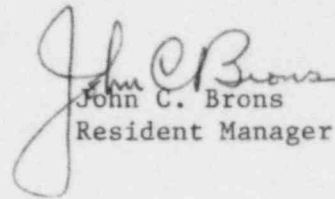
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 84-012-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(ii)(B).

Very truly yours,


John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

IP3 Resident Inspectors' Office
J. P. Bayne, WPO
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