PUBLIC SERVICE COMPANY OF COLORADO

FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 126

July, 1984

This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of July, 1984.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor has remained shutdown for the entire month of July for inspection and refurbishment of the six control rod drives which failed to automatically insert on June 23, 1984, and two others which successfully inserted. As of August 1, three control rod drives have been inspected and returned to the core. Inspection is continuing on the remaining control rod drives.

The main turbine generator has been balanced and is operating on the turning gear while blanketed with carbon dioxide.

The hot reheat Marmon flanges on B-2-5 and B-1-3 steam generator modules have been replaced with spool pieces, heat treated, and returned to service.

On July 7 and 8, 1984, a concentrated effort, by many plant personnel, was made to clean the plant as part of the overall housekeeping effort.

Numerous Nuclear Regulatory Commission (NRC) personnel were on site during the week of July 9, to audit and analyze Fort St. Vrain operations. Also, during the same week, Nuclear Regulatory Commission personnel administered oral requalification examinations to six plant persons. All involved personnel passed the Nuclear Regulatory Commission administered exams.

Segment 3 spent fuel shipping commenced this month.

To date, approximately 232 gallons of water have been removed from the primary coolant system.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None.

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATION

None.

4.0 MONTHLY OPERATING DATA REPORT

Attached.

OPERATING DATA REPORT

DOCKET NO. 50-267
DATE August 13, 1984
COMPLETED BY Chuck Fuller
TELEPHONE (303) 785-2224

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain
2. Reporting Period: 840701 through 840731
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   None
9. Power Level To Which Restricted, If Any (Net MWe): 280
10. Reasons for Restrictions, If Any: Per commitment to the NRC, long term operation above 85% power is pending completion of the B-0 Startup Testing.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	5,111	44,592
12. Number of Hours Reactor Was Critical	0.0	1,324.1	27,151.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	660.1	18,463.3
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	340,407.9	9,861,725.3
17. Gross Electrical Energy Generated (MWH)	0	95,144	3,248,594
18. Net Electrical Energy Generated (MWH)	-3,637	70,361	2,941,888
19. Unit Service Factor	0.0	12.9	41.4
20. Unit Availability Factor	0.0	12.9	41.4
21. Unit Capacity Factor (Using MDC Net)	0.0	4.2	20.0
22. Unit Capacity Factor (Using DER Net)	0.0	4.2	20.0
23. Unit Forced Outage Rate	100.0	59.7	40.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 8-1-84 through 11-1-84, 2208 hours, Control Rod Investigation
25. If Shut Down at End of Report Period, Estimated Date of Startup: 11-1-84

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain

Date August 13, 1984

Completed By Chuck Fuller

Telephone (303) 785-2224

Month July, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

*Generator on line but no net generation.

50-267

UNIT NAME Fort St. Vrain

DATE August 13, 1984

COMPLETED BY Chuck Fuller

TELEPHONE (303) 785-2224

REPORT MONTH July, 1984

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
84-006	840701	F	744.0	A	3	50-267/84-008	AA	JC	Control Rod Drive Investigation

REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	4th Refueling: February 1, 1986
3. Scheduled date for restart following refueling.	May 1, 1986
4. Will refueling or resumption of operation thereafter require a technic·l specification change or other license amendment?	No
If answer is yes, what, in general, will these be?	----------
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
If no such review has taken place, when is it scheduled?	1985
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	----------
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	----------
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	1482 HTGR fuel elements 245 spent fuel elements

REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.