

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY 1984

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Dave Kimler, telephone number 309-654-2241, extensions 127 and 192.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Unit One remained shutdown throughout the month for End of Cycle Seven Refueling and Maintenance.

B. Unit Two

July 1-6: Unit Two began the month in a reduced load state for weekly Turbine tests. At 0700 hours the unit began a normal load increase to full power. At 0230 hours, on July 3, load was dropped to 730 MWe for a Condensate Demineralizer changeover. At 0630 hours the unit began a normal load increase to full power. At 1115 hours, on July 4, and 0630 hours, on July 6, load was reduced for high Local Power Range Monitor (LPRM) indication.

July 7-17: At 0000 hours, on July 7, load was dropped to 550 MWe for weekly Turbine tests. During this reduction, a Control Rod pattern adjustment was initiated also. At 0520 hours the unit began normal increase to full power. At 0000 hours, on July 15, load was dropped to 700 MWe for weekly Turbine tests. At 0900 hours the unit began a normal load increase to full power. At 0915 hours, on July 17, load was dropped to 700 MWe for testing on the 2B Reactor Feedwater Pump. At 1150 hours the unit began a normal load increase to full power.

July 18-31: At 2330 hours, on July 22, load was dropped to 700 MWe for weekly Turbine tests. At 0630 hours the unit began a normal load increase to full power. At 0100 hours, on July 29, load was dropped to 700 MWe for weekly Turbine tests. At 0315 hours the unit began a normal load increase to full power.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On July 13, 1984, the NRC issued Amendments 77 and 86 to License DPR-29. Amendment 77 updates tables regarding instrumentation that initiates or controls core and containment cooling systems and the minimum test and calibration frequency for this instrumentation based on an undervoltage condition on the 4KV emergency buses. Amendment 77 also updates Auxiliary Electrical Systems Surveillance requirements for the Emergency Diesel Generators. Amendment 86 updates the Auxiliary Electrical Systems Surveillance Requirements, Limiting Conditions for Operation Bases, and Surveillance Requirements Bases for the Reactor Protection Bus Power Monitoring System.

B. Facility or Procedure Changes Requiring NRC Approval

There were no facility or procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no tests or experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Unit One and Unit Two during the reporting period. This summary includes the following headings: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q32294		Thermostat on 1/2 'B' Standby Gas Treatment System Heater	Thermostat tripped when the 1/2 'B' SBGTS was started.	The 1/2 'A' SBGTS was started and ran satisfactorily; thus, minimizing the con- sequences of this deviation. Reference Deviation Report 4-1-84-40.	The thermostat was reset, and did not trip when 1/2 'B' SBGTS was restarted. Reference Modifica- tion M-4-1/2-83-21 for further corrective action.
Q32779		Breaker tripped While Operating Valve MO 1-1402- 25B	Loose terminal at Motor Control Center.	The valve was opened manually immediately, and the in-line 1402- 24B was shut. The other loop of Core Spray was available, as was all other ECC systems. The safety implications were minimal.	Checked all termina- tions at the valve and Motor Control Center. Also, took current reading, which showed no excessive current during valve operation.
Q33755	84-2	Valve SO 1-2499- 4B Leaks Excessively	Valve did not seat completely.	Reference LER 84-2. The in-line valve SO 1-2499-3B would have provided PCI function.	The leakage rate was reduced to an acceptable value for startup.
Q33756	84-2	Valve SO 1-2499- 4A Leaks Excessively	Valve did not seat completely.	The in-line valve SO 1-2499-3A would have provided PCI function. Reference LER 84-2.	The leakage rate was reduced to an acceptable value for startup.
Q35138	84-2	A0 1-8803 Leaks Excessively	Incomplete seating of valve.	Reference LER 84-2. A0 1-8804 would have provided PCI function.	The leakage was reduced to an acceptable value for startup.

UNIT ONE MAINTENANCE SUMMARY

W. R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q35257	84-2	Shear Lug Inspection Hatch #1 Leaks	Degraded O-ring seals.	Leak was 8.5% of L _a . Reference LER 84-2.	Replaced O-rings on all shear lug inspection hatches.
Q35428	84-2	'B' Inboard Feedwater Check Valve Leaks	O-ring seating improperly.	Reference LER 84-2.	The leakage was reduced to an acceptable value for startup.
Q35586	84-2	Drywell Head Access Hatch Leaks	Bad gasket.	Reference LER 84-2.	Leakage was reduced to an acceptable level by replacing gasket.
Q35842		RHR Service Water Vault Sump Pump Discharge Check Valve Leaks	Dirt accumulation in lift-check valve.	Reference Deviation Report 4-1-84-33.	Valve was cleaned and tested satisfactorily.
Q35843		RHR Service Water Vault Sump Pump Discharge Check Valve Leaks	Dirt accumulation in lift-check valve.	Reference Deviation Report 4-1-84-33.	Valve was cleaned and tested satisfactorily.
Q35913		CRD 34-51 Will Not Latch-Up	Uncoupling rod inserted off centering hole.	Unit was shutdown for refueling. No safety implications associated with this defect.	Uncoupling rod was installed correctly and tested satisfactorily.
Q36038	84-2	HPCI Steam Exhaust Check Valve Leaks	Not seating properly.	The in-line 1-2301-74 valve would have provided the necessary PCI function. Reference LER 84-2.	A new valve was installed to bring leakage to an acceptable value before startup.

UNIT ONE MAINTENANCE SUMMARY

W. R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q36212		Corroded Leads on Condenser Pit 5 Ft. Level Switch 1-4441- 25A	Broken wire in switch mechanism, cyclic fatigue.	In the unlikely event of a condenser pit flooding, the other switches would have provided an alarm and a circulating water pump trip.	Reconnected wire. Tested satisfactorily. This is first failure of these switches. No other corrective action deemed necessary. Reference Deviation Report 4-1-83-13.
Q36248		RHR Service Water Vault Sump Pump Discharge Check Valve Leaks	Dirt accumulation in lift-check valve.	Reference Deviation Report 4-1-84-38.	Valve was cleaned and tested satisfactorily.
Q36396		ID RHR Service Water Vault Submarine Door Leaks	Latch stops out of adjustment.	Leak was very small. The sump pump inside could handle the flow incurred in the unlikely event of condensate pit flooding.	Adjusted latch stops.

UNIT TWO MAINTENANCE SUMMARY

W. R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q25444	83-18	Reactor Water Clean-up Line Pipe Cracks (2-1202-6''A)	Unknown. Suspect IGSCC induced indications.	Indication was 20% thru-wall, and 1¼ inch long.	Line was replaced from line 2-1025-20'' to M0 2-1201-2. Short downstream of M0 2-1201- 2 replaced also.
Q36070	84-8	Valve M0 2- 2301-48 Breaker Tripped	Unknown.	All other ECC systems available. Valve was immediately re-opened. HPCI was unavailable for only a short period of time.	Failure could not be reproduced. Current reading does not indicate a problem.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

There were no Licensee Event Reports for Unit One for the reporting period.

UNIT TWO

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-8	7-4-84	HPCI 2-2301-48 Valve Inoperable

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE August 8

COMPLETED BY DAVE KIMLER

TELEPHONE 309-654-2241X192

OPERATING STATUS

0000 070184

1. Reporting period: 2400 073184 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>0.0</u>	<u>1562.1</u>	<u>85117.7</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>0.0</u>	<u>1561.2</u>	<u>81909.1</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>0</u>	<u>3659732</u>	<u>168766438</u>
10. Gross electrical energy generated (MWH)	<u>0</u>	<u>1213148</u>	<u>54471764</u>
11. Net electrical energy generated (MWH)	<u>-3423</u>	<u>1144207</u>	<u>50750174</u>
12. Reactor service factor	<u>0.0</u>	<u>30.6</u>	<u>79.4</u>
13. Reactor availability factor	<u>0.0</u>	<u>30.6</u>	<u>82.6</u>
14. Unit service factor	<u>0.0</u>	<u>30.5</u>	<u>76.4</u>
15. Unit availability factor	<u>0.0</u>	<u>30.5</u>	<u>77.3</u>
16. Unit capacity factor (Using MDC)	<u>-0.6</u>	<u>29.1</u>	<u>61.6</u>
17. Unit capacity factor (Using Des. MWe)	<u>-0.6</u>	<u>28.4</u>	<u>60.0</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>6.1</u>
19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):			
20. If shutdown at end of report period, estimated date of startup			<u>8-11-84</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

#UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE August 8

COMPLETED BY DAVE KIMLER

TELEPHONE 309-654-2241X192

OPERATING STATUS

0000 070184

1. Reporting period: 2400 073184 Gross hours in reporting period: 744

2. Currently authorized power level (Mwt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>3565.1</u>	<u>81482.6</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>744.0</u>	<u>3455.0</u>	<u>78664.8</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>1820040</u>	<u>8100023</u>	<u>103482111</u>
10. Gross electrical energy generated (MWH)	<u>580384</u>	<u>2619090</u>	<u>52054870</u>
11. Net electrical energy generated (MWH)	<u>554452</u>	<u>2494629</u>	<u>48828689</u>
12. Reactor service factor	<u>100.0</u>	<u>69.8</u>	<u>76.7</u>
13. Reactor availability factor	<u>100.0</u>	<u>69.8</u>	<u>79.5</u>
14. Unit service factor	<u>100.0</u>	<u>67.6</u>	<u>74.0</u>
15. Unit availability factor	<u>100.0</u>	<u>67.6</u>	<u>74.7</u>
16. Unit capacity factor (Using MDC)	<u>96.9</u>	<u>63.5</u>	<u>59.8</u>
17. Unit capacity factor (Using Des. MWe)	<u>94.5</u>	<u>61.9</u>	<u>58.3</u>
18. Unit forced outage rate	<u>0.0</u>	<u>4.1</u>	<u>8.4</u>
19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):			
20. If shutdown at end of report period, estimated date of startup			<u>NA</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE August 8

COMPLETED BY DAVE KIMLER

TELEPHONE 309-654-2241X192

MONTH July 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-3.6</u>
2.	<u>-4.5</u>
3.	<u>-4.3</u>
4.	<u>-4.2</u>
5.	<u>-4.0</u>
6.	<u>-2.8</u>
7.	<u>-3.5</u>
8.	<u>-4.8</u>
9.	<u>-4.8</u>
10.	<u>-4.5</u>
11.	<u>-5.3</u>
12.	<u>-4.9</u>
13.	<u>-6.3</u>
14.	<u>-5.6</u>
15.	<u>-5.8</u>
16.	<u>-6.1</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-5.3</u>
18.	<u>-6.7</u>
19.	<u>-6.5</u>
20.	<u>-6.4</u>
21.	<u>-6.6</u>
22.	<u>-6.8</u>
23.	<u>-7.3</u>
24.	<u>-6.2</u>
25.	<u>-6.5</u>
26.	<u>-7.0</u>
27.	<u>-6.4</u>
28.	<u>-6.4</u>
29.	<u>-6.6</u>
30.	<u>-6.5</u>
31.	<u>-7.4</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE August 8

COMPLETED BY DAVE KIMLER

TELEPHONE 309-654-2241X192

MONTH July 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>740.5</u>
2.	<u>772.4</u>
3.	<u>734.2</u>
4.	<u>762.6</u>
5.	<u>739.6</u>
6.	<u>750.6</u>
7.	<u>571.6</u>
8.	<u>671.1</u>
9.	<u>757.3</u>
10.	<u>758.9</u>
11.	<u>764.1</u>
12.	<u>758.9</u>
13.	<u>762.4</u>
14.	<u>753.6</u>
15.	<u>720.0</u>
16.	<u>764.2</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>736.9</u>
18.	<u>773.8</u>
19.	<u>764.8</u>
20.	<u>770.8</u>
21.	<u>758.5</u>
22.	<u>730.9</u>
23.	<u>754.0</u>
24.	<u>760.3</u>
25.	<u>772.3</u>
26.	<u>742.9</u>
27.	<u>766.5</u>
28.	<u>764.3</u>
29.	<u>740.8</u>
30.	<u>760.3</u>
31.	<u>760.7</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit 2

CORRECTED COPY

COMPLETED BY D KimlerDATE August 6, 1984REPORT MONTH JUNE 1984TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-19	840601	S	0.0*	B	5		RC	CONROD	Reduced load for rod maneuver in preparation for unit shutdown
84-20	840602	S	57.8	B	2		ED	TRANSF	Manually scrambled unit to repair oil leak on Transformer 21
84-21	840606	S	0.0*	H	5		RC	CONROD	Reduced load to perform Control Rod maneuver per Nuclear Engineer
84-22	840610	S	0.0*	B	5		CD	VALVEX	Reduced load to perform bi-weekly Main Steam Isolation Valve test
84-23	840610	F	18.1	A	3		CC	VALVEX	Reactor scram due to 'FAST' closure of #4 Control Valve during testing
84-24	840611	S	0.0*	B	5		CH	VALVEX	Reduced load to allow Maintenance to work on Feedwater Heater Valve
84-25	840613	S	0.0*	H	5		RC	CONROD	Reduced load to perform Control Rod maneuver per Nuclear Engineer

*CORRECTED FIGURES FOR JUNE 1984 REPORT

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AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

CORRECTED COPY

COMPLETED BY D Kimler

DATE August 6, 1984

REPORT MONTH JUNE 1984

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-26	840616	S	0.0*	B	5		HA	TURBIN	Reduced load to perform weekly Turbine tests
84-27	840624	S	0.0*	B	5		HA	TURBIN	Reduced load to perform weekly Turbine tests
84-28	840630	S	0.0*	B	5		HA	TURBIN	Reduced load to perform weekly Turbine tests
*CORRECTED FIGURES FOR JUNE 1984 REPORT									

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AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY D Kimler

DATE August 2, 1984

REPORT MONTH JULY 1984

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-14	840306	S	744	C	1		RC	FUELXX	Unit One remains shutdown for End of Cycle Seven Refueling and Maintenance

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit 2COMPLETED BY D KimlerDATE August 2, 1984REPORT MONTH JULY 1984TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
84-29	840703	S	0.0	B			CH	DEMINX	Reduced load to perform Condensate Demineralizer changeover.
84-30	840704	S	0.0	H			IB	INSTRU	Reduced load due to high LPRM reading
84-31	840706	S	0.0	H			IB	INSTRU	Reduced load due to high LPRM reading
84-32	840707	S	0.0	B			HA	TURBIN	Reduced load to perform weekly Turbine tests
84-33	840715	S	0.0	B			HA	TURBIN	Reduced load to perform weekly Turbine tests
84-34	840717	S	0.0	B			CH	PUMPXX	Reduced load to perform test on 2B Reactor Feedwater Pump
84-35	840722	S	0.0	B			HA	TURBIN	Reduced load to perform weekly Turbine tests
84-36	840729	S	0.0	B			HA	TURBIN	Reduced load to perform weekly Turbine tests

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VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q1 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: Refueling Outage
Currently in Progress
3. Scheduled date for restart following refueling: 8-11-84
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Yes. Preparatory Technical Specification changes have been submitted to include MAPLHGR curve for one of the reload fuel types and extending MAPLHGR curve for BLTA to 45,000 MWD/t.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
Technical Specification change has been submitted February 21, 1984.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
1) All new fuel assemblies will be GE7B-type (barrier clad, extended exposure design).
2) A generic methodology was used for the analysis of the Control Rod Drop Accident and Rod Withdrawal Error events.
3) Four Barrier Lead Test Assemblies will be re-inserted to gather information on the effects of extended exposures.
7. The number of fuel assemblies.
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 1926
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3657
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q2 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 3-18-85
3. Scheduled date for restart following refueling: 5-26-85
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
Not as yet determined.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
January 18, 1985, if licensing action required.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
1) All new fuel assemblies will be GE7B-type (barrier clad, extended exposure design).
2) A generic methodology was used for the analysis of the Control Rod Drop Accident and Rod Withdrawal Error events.
7. The number of fuel assemblies.
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 414
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3897
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



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NJK-84-233

August 1, 1984

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of July 1984.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

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Enclosure

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