

December 8, 1995

Mr. Neil S. Carns  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, Kansas 66839

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI), GENERIC LETTER 95-03,  
"CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES" - WOLF CREEK  
NUCLEAR GENERATING STATION (TAC NO. M92289)

Dear Mr. Carns:

The NRC staff has reviewed your June 23, 1995, response to Generic Letter 95-03, "Circumferential Cracking of Steam Generator Tubes" (GL 95-03), for the Wolf Creek Generating Station and has identified several areas where additional information is needed.

The enclosure details the additional information the staff requires in order to complete their review of your response to GL 95-03. Please provide your response within 30 days of receipt of this letter. This request is within the original reporting burden for information collection of 350 hours covered by the Office of Management and Budget clearance number 3150-0011, which expires July 31, 1997.

If you have any questions, please contact me.

Sincerely,  
Original signed by:  
James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosure: Request for Additional Information

cc w/encl: See next page

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Mr. Neil S. Carns

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WOLF CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-482

REQUEST FOR ADDITIONAL INFORMATION REGARDING RESPONSE TO GENERIC LETTER 95-03

1. The following areas have been identified as being susceptible to circumferential cracking:
  - a. Expansion transition circumferential cracking
  - b. Small-radius U-bend circumferential cracking
  - c. Dented location (including dented TSP) circumferential cracking
  - d. Sleeve joint circumferential cracking

In your response, area d was not specifically addressed. These areas have been identified as being susceptible to circumferential cracking as evidenced by operating experience at other recirculating steam generators. Please submit the information requested in Generic Letter (GL) 95-03 per the guidance contained in the GL for this area (and any other area susceptible to circumferential cracking). The staff realizes that some of the above areas may not have been addressed since they may not be applicable to your plant; however, the staff requests that you clarify this (e.g., no sleeves are installed; therefore, the plant is not susceptible to sleeve joint circumferential cracking).

In your response, you indicated that a small-radius U-bend motorized rotating pancake coil (MRPC) inspection program has not been performed at your plant. You also indicated that the tubes have wide-radius U-bends and are fabricated from thermally treated Alloy 600. If the small-radius U-bends are susceptible to circumferential cracking, please provide your inspection plans including expansion criteria, if applicable, for the next steam generator tube inspection outage per the guidance in GL 95-03.

2. During the Maine Yankee outage in July/August 1994, several weaknesses were identified in their eddy current program as detailed in NRC Information Notice 94-88, "Inservice Inspection Deficiencies Result in Severely Degraded Steam Generator Tubes." In Information Notice 94-88, the staff observed that several circumferential indications could be traced back to earlier inspections when the data was reanalyzed using terrain plots. These terrain plots had not been generated as part of the original field analysis for these tubes. For the rotating pancake coil (RPC) examinations performed at your plant at locations susceptible to circumferential cracking during the previous inspection (i.e., the previous inspection per your Generic Letter 95-03 response), discuss the extent to which terrain plots were used to analyze the eddy current data. If terrain plots were not routinely used at locations susceptible to circumferential cracking, discuss whether the RPC eddy current data has been reanalyzed using terrain mapping of the data. If terrain plots were not routinely used during the outage and your data has not been reanalyzed with terrain mapping of the data, discuss your basis for not reanalyzing your previous RPC data in light of the findings at Maine Yankee.

Discuss whether terrain plots will be used to analyze the RPC eddy current data at locations susceptible to circumferential cracking during your next steam generator tube inspection (i.e., the next inspection per your Generic Letter 95-03 response).