U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3180-0104 EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

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On 07-17-84 engineering personnel determined that a control rod had been moved within the reactor pressure vessel with the refueling platform frame mounted auxiliary hoist prior to demonstrating operation of the hoist's loaded interlock as required by Tech. Specs. section 4.9.7.b. Upon determining that hoist operation was contrary to Tech. Specs. surveillance requirements, further hoist use was suspended. A procedure revision will be made before the hoist use is resumed. This event is reportable per 10CFR 50.73(:)(2)(i)(B), because an operation occurred which was prohibited by the plant's Technical Specifications.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6	684-0006-00001200012

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 07-17-84 with the reactor head removed, and during a routine procedure review, engineering personnel determined that a control rod had been moved within the reactor pressure vessel with the refueling platform (2F15-E003) frame mounted auxiliary hoist on 07-16-84 prior to demonstrating operation of the hoist's loaded interlock. Demonstration of the operation of the hoist's loaded interlocks prior to handling control rods or fuel assemblies in the reactor pressure vessel is required by Tech. Specs section 4.9.7.b. Therefore this event is contrary to the requirements of Tech. Specs. section 4.9.7.b., and is reportable per 10CFR 50.73(a)(2)(i)(B).

This event is the result of a procedure inadequacy. The "REFUELING INTERLOCKS AND HOIST LIMIT CHECKS" procedure (HNP-2-3940) did not include the Tech. Specs. surveillance requirement to demonstrate operation of the refueling platform (2F15-E003) FRAME MOUNTED hoist's loaded interlock. However the procedure (HNP-2-3940) did include all other Tech. Specs. surveillance requirements for the hoist.

When it was determined (on 07-17-84) that the procedure (HNP-2-3940) did not include the Tech. Specs. surveillance requirement to demonstrate the hoist's loaded interlock operation, an LCO was established and the hoist was removed from service on 07-19-84 per Tech. Specs. section 3.9.7 ACTION statement by attaching a tag to it per the "EQUIPMENT CLEARANCE AND TAGGING" procedure (HNP-501).

The refueling platform frame mounted hoist will not be used to handle fuel assemblies or control rods in the reactor vessel until after the procedure (HNP-2-3940) has been revised to include the Tech. Specs. surveillance requirement to satisfactorily demonstrate the hoist's loaded interlock operation.

There is no redundant or backup system for the refueling platform hoist's loaded interlock.

This non-repetitive event had no impact upon any other system in Unit 1, or Unit 2; nor did it affect the health and safety of the public.



Edwin I. Hatch Nuclear Plant

August 14, 1984 GM-84-672

PLANT E. I. HATCH Licensee Event Report Docket No. 50-366

United States Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1984-006. This report is required by 10078 50.73(a)(2)(i).

H. C. Nix

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