

November 29, 1995

Mr. R. E. Querio  
Site Vice President  
LaSalle County Station  
ComEd  
2601 North 21st Road  
Marseilles, IL 61341

SUBJECT: NRC INTEGRATED INSPECTION REPORT NOS. 50-373/95009(DRP)  
AND 50-374/95009(DRP)

Dear Mr. Querio:

This refers to the inspection conducted on September 1 through October 13, 1995, at the LaSalle facility. The purpose of the inspection was to determine whether activities authorized by the license were conducted safely and in accordance with NRC requirements. At the conclusion of the inspection, findings were discussed with those members of your staff identified in the enclosed report.

Areas examined during the inspection are identified in the report. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observation of activities in progress.

While no violations of NRC requirements were identified during this inspection, we are concerned with the performance of several plant ventilation systems. Poor system performance caused numerous challenges this past summer to plant equipment and personnel safety, including heat related stress to workers and an automatic plant scram. Problems with the ventilation systems include marginal design and non-existent preventive maintenance. The most safety significant issues involve the reactor building ventilation (VR) system design weaknesses.

Loss of the VR system requires operators to bypass safety signals, within a very short time, to avoid a dual unit automatic scram. It appears this bypassing of specific safety signals, by installing jumpers, has become a proceduralized operator work-around, compensating for plant deficiencies. While it is desirable to avoid unnecessary transients, we question the advisability of implementing work-arounds rather than addressing known design and material condition deficiencies. While we are aware that you are actively pursuing these issues, we are concerned that inadequate ventilation may be causing premature aging of equipment and hampering your maintenance staff's ability to perform work. Therefore, we request that you respond within 60 days of the date of this letter addressing our concerns. In your response, please provide: (1) a description of the actions you have taken, or plan to take, to identify and correct problems with your plant ventilation systems,

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particularly those that impact operations or necessitate bypassing of safety signals; (2) your plans to resolve the issue of needing to bypass the MSIV isolation signals; and (3) your planned schedule for the corrective actions.

We are also concerned that material condition improvement efforts were achieving slow progress. We have also noted problems with the quality and rigor of your maintenance activities. As examples, the maintenance outage on Unit 2 was precipitated by inadequate maintenance on the 2B reactor recirculation flow control valve during the last refueling. Also, subsequent to this report, the incorrect installation of a thrust bearing in the 1D condensate booster pump led to its catastrophic failure, only 3 weeks after a major overhaul.

Based on the results of the close-out inspection for your motor-operated valve (MOV) activities, we have determined that your program and implementation meet the intent of Generic Letter 89-10. Accordingly, we are closing the NRC review of your program. Credit should be given to your corporate and site engineers for their innovative MOV program initiatives that combined sophisticated statistical analyses and risk considerations to enhance the performance of highly important MOVs. ComEd has also assumed an industry leadership role in the area of pressure locking and thermal binding testing. Additionally, your motor and actuator testing program will help resolve longstanding technical questions on MOV performance.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practices," a copy of this letter and its enclosure will be placed in the NRC Public Document Room (PDR). We will gladly discuss any questions you have concerning this inspection.

Sincerely,

Original Signed by W. L. Axelson

William L. Axelson, Director  
Division of Reactor Projects

Docket Nos: 50-373; 50-374  
License Nos: NPF-11; NPF-18

Enclosure: Inspection Reports  
No. 50-373/95009(DRP);  
No. 50-374/95009(DRP)

cc w/encl: J. C. Brons, Vice President,  
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