

**GPU Nuclear Corporation**  
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Writer's Direct Dial Number:  
August 15, 1984

Mr. Dennis M. Crutchfield, Chief  
Operating Reactors Branch No. 5  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dear Mr. Crutchfield:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Plant Specific Analysis to Determine Gas and Water  
Clearing Thrust Loads Associated with Second  
SRV Actuations in Relief Valve Discharge Lines

Your letter dated June 15, 1984, inquired if any analyses were performed utilizing methodology not previously approved in NUREG 0661 in determining if setpoint logic changes were needed to reduce water clearing thrust loads associated with second SRV actuations. Additionally, we were requested to identify if any credit for operator action is required to prevent or reduce the thrust loads associated with second actuations of SRVs.

For the Oyster Creek plant, only loading and structural response methods contained in NUREG 0661 were used when evaluating the Mark I containment. By analysis, the Oyster Creek Mark I containment has been demonstrated to be structurally adequate for a five (5) valve actuation followed by an additional five (5) valve actuation.

ADS logic changes were incorporated in 1977 to assure that if an EMRV were to lift under high pressure during ADS time-out, the valve would remain open. This strategy rules out the possibility of a valve closing just before ADS time-out. If this were to happen, vacuum relief would not occur in the downcomer whereby an ADS actuation would have propelled a long slug of water into the torus.

ADS logic requires that one valve in each header be set lower than the other(s).

Oyster Creek takes no credit for operator action. Operator action is not required for ADS operation. Manually lifting all EMRVs, given a valid ADS signal, would not defeat the ADS modifications, but would instead effectively create a new timer limit. Manual opening of the low setpoint valve has the same effect as opening on high pressure. Manual opening of any other valve may cause operation beyond analyzed limits.


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Should you have any questions regarding this information, please contact  
Mr. Drew Holland, Oyster Creek Licensing Manager at (609)971-4643.

Very truly yours,

  
Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF/BH/dam

cc: Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
637 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731