December 5, 1995

Mr. C. K. McCoy
Vice President - Nuclear
Vogtle Project
Georgia Power Company
P. O. Box 1295
Birmingham, AL 35201

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING LICENSEE RESPONSE TO GENERIC LETTER 95-03, "CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES" (TAC NOS. M92286 AND M92287)

Dear Mr. McCoy:

Enclosed please find a Request for Additional Information (RAI) that was developed during the NRC's review of Georgia Power Company's June 27, 1995, response to Generic Letter 95-03, "Circumferential Cracking of Steam Generator Tubes." Please provide your written response to the questions contained in the enclosed RAI not later than 30 days from the date of this letter. If there are any questions regarding this action, please have your staff contact me at (301) 415-1444.

This requirement affects nine or fewer respondents and therefore is not subject to the Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original signed by:
Louis L. Wheeler, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

Enclosure: As stated

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 5, 1995

Mr. C. K. McCoy
Vice President - Nuclear
Vogtle Project
Georgia Power Company
P. O. Box 1295
Birmingham, AL 35201

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Docket Nos. 50-424 and 50-425

Enclosure: As stated

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cc:
Mr. J. A. Bailey
Manager - Licensing
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Mr. J. B. Beasley
General Manager, Vogtle Electric
Generating Plant
P. O. Box 1600
Waynesboro, Georgia 30830

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street, NW., Suite 2900
Atlanta, Georgia 30323

Office of Planning and Budget
Room 615B
270 Washington Street, SW.
Atlanta, Georgia 30334

Office of the County Commissioner
Burke County Commission
Waynesboro, Georgia 30830

Mr. J. D. Woodard
Senior Vice President
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Harold Reheis, Director
Department of Natural Resources
205 Butler Street, SE. Suite 1252
Atlanta, Georgia 30334

Attorney General
Law Department
132 Judicial Building
Atlanta, Georgia 30334

Mr. Ernie Toupin
Manager of Nuclear Operations
Oglethorpe Power Corporation
2100 East Exchange Place
Tucker, Georgia 30085-1349

Charles A. Patrizia, Esquire
Paul, Hastings, Janofsky & Walker
10th Floor
1299 Pennsylvania Avenue
Washington, DC 20004-9500

Arthur H. Domby, Esquire
Troutman Sanders
NationsBank Plaza
600 Peachtree Street, NE.
Suite 5200
Atlanta, Georgia 30308-2216

Resident Inspector
U.S. Nuclear Regulatory Commission
8805 River Road
Waynesboro, Georgia 30830

ENCLOSURE

**REQUEST FOR ADDITIONAL INFORMATION
ON THE GEORGIA POWER COMPANY
REPLY TO GENERIC LETTER (GL) 95-03**

1. Please clarify the expansion criteria to be used if a circumferential indication were detected at the expansion transition region.

2. Small radius U-bends and dented locations have been identified as being susceptible to circumferential cracking as evidenced by operating experience at plants with mill annealed Alloy 600 steam generators. If these locations are susceptible to circumferential cracking at Vogtle, please provide your inspection plans including expansion criteria, if applicable, for the next steam generator tube inspection outage per the guidance in GL 95-03.

   For dented locations, if applicable, the criteria for determining which dents, if any, are to be examined should be provided. If a dent voltage threshold is used for such a determination, a brief description of the calibration procedure should be provided (i.e., 2.75 volts peak-to-peak on 4-20% through-wall ASME holes at 550/130 mix).

3. During the Maine Yankee outage in July/August 1994, several weaknesses were identified in their eddy current program as detailed in NRC Information Notice 94-88, "Inservice Inspection Deficiencies Result in Severely Degraded Steam Generator Tubes". In Information Notice 94-88, the staff observed that several circumferential indications could be traced back to earlier inspections when the data was reanalyzed using terrain plots. These terrain plots had not been generated as part of the original field analysis for these tubes. For the rotating pancake coil (RPC) examinations performed at your plant at locations susceptible to circumferential cracking during the previous inspection (i.e., previous inspection per your GL 95-03 response), discuss the extent to which terrain plots were used to analyze the eddy current data. If terrain plots were not routinely used at locations susceptible to circumferential cracking, discuss whether or not the RPC eddy current data has been reanalyzed using terrain mapping of the data. If terrain plots were not routinely used during the outage and your data has not been reanalyzed with terrain mapping of the data, discuss your basis for not reanalyzing your previous RPC data in light of the findings at Maine Yankee.

   Discuss whether terrain plots will be used to analyze the RPC eddy current data at locations susceptible to circumferential cracking during your next steam generator tube inspection (i.e., the next inspection per your GL 95-03 response).