Docket No. 50-341

Mr. William S. Orser Senior Vice President - Nuclear Operations Detroit Edison Company 6400 North Dixie Highway Newport, Michigan 48166 DISTRIBUTION:

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JZwolinski LBMarsh
PShuttleworth TColburn
RScholl SNewberry
MPSiemien, OGC PD3-1 Plant File

Dear Mr. Orser:

SUBJECT: REQUEST FUR ADDITIONAL INFORMATIC ' "LATED TO FERM! 2 POWER UPRATE AMENDMENT REQUEST (TAC NO. 82102)

In reviewing your September 24, 1991 amendment request, NRC-91-0102, related to power uprate for Fermi 2, the staff requested additional information by letter dated February 25, 1992. You responded to our request by letter dated March 25, 1992. After reviewing your response, we have determined that we will need additional information identified in the enclosure in order to continue our review.

You are requested to respond as soon as possible, but not later than 30 days from receipt of this letter in order for us to maintain our review schedule. For your convenience a copy of the enclosure has been telecopied to T. Riley of your staff. If you have any questions, please call me at (301) 504-1341.

The reporting and/or recordkecping requirements of this letter affect fewer than ten respondents, therefore OMP clearance is not required under P.L. 96-511.

Sincerely,

original signed by

Timothy G. Colburn, Sr. Project Manager Project Directorate III-1 Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: See next page

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PShuttleworth	TColburn/tg	LBMarsh for
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## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

April 10, 1992

Docket No. 50-347

Mr. William S. Orser Senior Vice President - Nuclear Operations Detroit Edison Company 6400 North Dixie Highway Newport, Michigan 48166

Dear Mr. Orser:

SUBJECT: REQUEST FOR WITTIONAL INFORMATION RELATED TO FERMI 2 POWER UPRATE AMENDMENT REQUEST (TAC No. 82102)

In reviewing your September 24, 1991 amendment request, NRC-91-0102, related to power uprate for Fermi 2, the stiff requested additional information by letter dated February 25, 1992. You responded to our request by letter dated March 26, 1992. After reviewing your response, we have determined that we will need additional information identified in the enclosure in order to continue our review.

You are requested to respond as soon as possible, but not later than 30 days from receipt of this letter in order for us to maintain our review schedule. For your convenience a copy of the enclosure has been telecopied to T. Riley of your staff. If you have any questions, please call no at (301) 504-1341.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents, therefore OMB clearance is not required under P.L. 96-511.

Sincerely,

Timothy G. Colburn, Sr. Project Manager

Project Directorate III-1

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Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stayed

cc: See next page

cci

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Regional Administrator, Region III U.S. Nuclear Regulatory Commission 79° Roosevelt Road Glen Ellyn, Illinois 60137

Mr. A. Cecil Settles Director - Nuclear L.censing Detroit Edison Company Fermi Unii 2 6400 North Dixie Highway Newport, Michigan 48166

## ENCLOSURE 1

## REQUEST FOR ADDITIONAL INFORMATION FERMI UNIT 2 POWER UPRATE REVIEW

We have reviewed the sample calculations submitted with your March 26, 1892 letter. During our review we have identified some inconsistencies within your calculations presented in Attachment 1 to NRC-92-0043 and between your calculations and your principle reference in these matters (NEDC-31336). Therefore:

- 1. Provide the elations that define the relationship between volta displayed or le Fluke 8600A dvm and the process units of measurement is all of your calculations.
- Provide the calibration voltage and range used to determine the value of FL in the channel calibration accuracy calculations for the Resemburt model 1151 transmitters
- I Describe how the Rosemount transmitter correct signal is converted to a voltage for the dvm and identify where the calibration error for this shout is accounted for in your calculations.
- 4. Back calculations for the PSE on the Bosemount transmitters indicates that you used a value of 0.222 volum for the variation in unpoly voltage. Page 2-8 of NEDO-21017 ("Analog Transmitter/Triv Unit System for Engineered Safeguard bensor Trip Units") states that the design output voltage rip; le for the 24 volt power supply is 1% (.24 volts). What is your basis for using a lesser value for the ripule?
- 5. Your calculations for trib unit drift (page 1-8) indicate that you are using equations that NEDC-31336 states are applicable to a BWR-6. Ferm! Unit 2 is a BWR-4. Justify using the less conservative calculations.
- 6. Your calculations for PMA (page 1-10) present the results for four cases. It appears that these results are not random and have a positive bias. Justify treating the PMA error as a random variable for Reactor Vessel Steam Dome Pressure.
- 7. The formula used to calculate the SPE for a Rosemount model 1161 used in the differential mode is different from that stated on tage 2-11 of NEDC-31336. Justily the equation used for Fermi 2, and identify all other instances where the methods used to calculate your setpoints differ from NEDC-31336.
- 8. The formula used to calculate C for the Main Steam Line Flow (page 2-10) contains a coefficient for C2 that is not used in a similar calculation on page 1-9. Please explain the difference.

- 3. The specification for several error sources in Section 3.2 (page 3-8) of your calculations are different from the values given in NEDC-31336. For each of the examples given below. please explain why your approach is better than that presented in NEDC-31336:
  - a. VA -You give a value of 1% (with a separate drift value (VD) of 1.2%). NEDC-31336 (pages 3-129 AND 4-78) gives a value of 1.98% with drift and calibration errors included.
  - b. ATU -You give a value of 1.25% (with a separate drift value (DTU) of 0.63%). NEDC-31326 (page 4-80) gives a value of 2% for a flow biased trip with drift included.
  - c (1 -You refer to an off-rated condition which is identified by (1. Identify this off rate on dition and explain why it is not discussed in NEDT-31338.
  - d. NFTA-BPRA is a bins type error. Its value at earn to to determined by the type of trip unit used (see question 8.5 above). Explain why the type of trip unit does or does not influence this parameter and our est it if necessary.