

NOVEMBER 13, 1995

Florida Power Corporation  
Crystal River Energy Complex  
Mr. P. M. Beard, Jr. (SA2A)  
Sr. VP, Nuclear Operations  
ATTN: Mgr., Nuclear Licensing  
15760 West Power Line Street  
Crystal River, FL 34428-6708

SUBJECT: CLOSURE OF NRC STAFF REVIEW OF GENERIC LETTER 89-10 PROGRAM

Gentlemen:

This is to inform you that we have completed our review of the program which you implemented in accordance with Generic Letter 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance." The NRC staff recognizes the significant effort required to implement GL 89-10 at Crystal River and commend you for the improvements in plant safety that have resulted from your MOV program.

During the week of June 19, 1995, NRC Region II conducted an inspection of 13 of 14 previously identified inspector follow-up items regarding your implementation of Generic Letter 89-10. We were satisfied with your response to those items. With respect to the one remaining item, IFI 50-302/94-18-09 (periodic verification of MOV design-basis capability), your letter of March 13, 1995, indicated you planned to perform static diagnostic tests on all safety-related MOVs every 6 years and dynamic diagnostic tests on a small sample of MOVs over the next 3 outages. The NRC staff is preparing a generic letter on the periodic verification of MOV design-basis capability. Therefore, the staff is not conducting a detailed evaluation of your long-term periodic verification plans at this time. The staff will review your MOV periodic verification program following issuance of the new generic letter. You should review your periodic verification program in light of the new generic letter and consider any appropriate adjustments. For example, you should consider the benefits (such as identification of decreased thrust output and increased thrust requirements) and potential adverse effects (such as accelerated aging or valve damage) when determining appropriate periodic verification testing for each GL 89-10 MOV.

Over the long term, new information on valve performance, such as (1) Generic Letter 95-07, "Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves," and (2) the results of the Electric Power Research Institute's MOV testing program, may need to be addressed to ensure that safety-related MOVs are capable of performing their design-basis functions. The staff will review your response to these issues and other additional information on MOV performance as part of future NRC activities.

Based on the results of our review of your submittal of March 13, 1995 and on the NRC inspection conducted during the week of June 19, 1995, we are closing our review of the GL 89-10 program at Crystal River. The NRC staff concludes

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our review of the GL 89-10 program at Crystal River. The NRC staff concludes that you have implemented a program that meets the intent of GL 89-10 to verify the current design-basis capability of the safety-related MOVs at Crystal River Unit 3.

If you have any questions regarding this letter, please contact me at (404) 331-5596 or Terence Chan of NRR at (301) 415-1750.

Sincerely,

ORIGINAL SIGNED BY

PAUL FREDRICKSON

Paul E. Fredrickson, Chief  
Special Inspection Branch  
Division of Reactor Safety

Docket No. 50-302  
License No. DPR-72

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