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Dave Morey Vice President Farley Project

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December 1, 1995

Docket No.: 50-348

10 CFR 50.73

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U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

> Joseph M. Farley Nuclear Plant – Unit 1 Licensee Event Report No. 95-010-00 Actuation of Engineered Safety Feature Equipment <u>Due to Loss of Main Feedwater</u>

Ladies and Gentlemen:

Joseph M. Farley Nuclear Plant Licensee Event Report No. 95-010-00 is being submitted in accordance with 10 CFR 50.73(a)(2)(iv). If you have any questions, please advise.

Respectfully submitted,

more

Dave Morey

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Enclosure

cc: Mr. S. D. Ebneter, Region II Administrator Mr. B. L. Siegel, NRR Senior Project Manager Mr. T. M. Ross, FNP Resident Inspector

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At 1905, on November 5, 1995, with Unit 1 in Mode 1 operating at 28 percent power, an automatic actuation of engineered safety feature (ESF) [JE] pumps occurred when both motor driven auxiliary feedwater (MDAFW) pumps [BA] auto started due to trip conditions on both steam generator feedwater pumps (SGFP). This occurred when the operating 'B' SGFP tripped on low iube oil pressure, during attempts to return lube oil system to its normal operating configuration. The 'A' SGFP had been taken out of service to have an overspeed test performed and was unavailable at time of the trip. The 'B' SGFP trip, concurrent with the 'A' SGFP being off-service, resulted in a trip condition on both SGFPs and the automatic actuation of the MDAFW pumps [BA]. These actions were followed by a manual trip of the main turbine. The reactor was stabilized at one percent power.

The primary cause of this event was the improper adjustment of the SGFP's lube oil pressure regulating system due to inadequate procedural guidance. The lube oil pressure regulating system was properly adjusted, restarted and the system functioned as designed. A contributing cause to this event was a perceived need to return the SGFP lube oil system to its normal operating configuration.

NRČ FORM 366A (4-95)	U.S.NUCLEAR REGULATORY COMMISSION	APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/96									
LICENSEE EVENT REPO	DRT (LER) FION	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH T& F33]. U.S. NUCLEAR REGULATORY COMMISSION. WASHINGTON DC 20555-001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0164), OFFICE OF MANAGEMENT AND BUDGET. WASHINGTON DC 2053.									
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Plant and System Identification

Westinghouse - Pressurized Water Reactor Energy Industry Identification System codes are identified in the text as [XX].

Description of Event

At 1905, on November 5, 1995, with Unit 1 in Mode 1 operating at 28 percent power, an automatic actuation of engineered safety feature (ESF) [JE] pumps occurred when both motor driven auxiliary feedwater (MDAFW) pumps [BA] auto started due to trip conditions on both steam generator feedwater pumps (SGFP). This occurred when the operating 'B' SGFP tripped on low lube oil pressure, during attempts to return lube oil system to its normal operating configuration. The 'A' SGFP had been taken out of service to have an overspeed test performed and was unavailable at time of the trip. The 'B' SGFP trip, concurrent with the 'A' SGFP being off-service, resulted in a trip condition on both SGFPs and the automatic actuation of the MDAFW pumps [BA]. These actions were followed by a manual trip of the main turbine. The reactor was stabilized at one percent power.

On November 2, 1995, following a refueling outage, it was reported, while putting the 'B' SGFP in service, that 'B' SGFP lube oil pressure was low. Adjustments on the SGFP's lube oil pressure regulating system were made as a result of inspections and testing. Following the adjustments, the lube oil system functioned properly. The SGFP lube oil pressure was monitored to ensure lube oil pressure stayed in the expected normal operating range. On November 5, 1995, the SGFP lube oil pressure began to unexpectedly decrease and the emergency lube oil pump was started. Subsequently, the unexpected decrease in pressure was determined to have been caused by the improper adjustment of the SGFP's lube oil pressure regulating system. However, once the emergency lube oil pump was started, the lube oil pressure increased to a pressure higher than the acceptable range. In an effort to reduce the lube oil pressure, the number 2 main lube oil pump was started and the emergency lube oil pump was secured. It was considered that the number 2 main lube oil pump would maintain the lube oil pressure within the normal operating range. Therefore, the number 1 main lube oil pump was secured to return the lube oil system to it's normal operating configuration. After securing the number 1 main lube oil pump, the lube oil pressure to the SGFP decreased to less than 10 psig, thus causing the SGFP to trip. The emergency lube oil pump was manually started prior to reaching its autosiart setpoint. However, the emergency lube oil pump did not reach running speed prior to the SGFP reaching its trip setpoint.

NRC FORM 366A	U.S.NUCLEAR REGULATORY COMMISSION							
LICENSEE EVENT REPO TEXT CONTINUAT	RT (LER) ION	ESTIMA REPOR LICENS COMMI AND R REGUL TO THE	ATED I ATORY TED L BING P ENTS P ECORI ATORY E PAPI NAGEN	BURDEN PE INFORMATK ESSONS LE ROCESS AN REGARDING F XS MANAGEI COMMISSIC ERWORK RE AENT AND BL	R REI ON CLARNED D FED BURDE MEINT ON, W	SPONSE TO OLLECTION D ARE INCO BACK TO I EN ESTIMATE BRANCH (T- ASHINGTON ION PROJEC WASHINGT	COMPLY V REQUEST: 1 XPORATED NDUSTRY TO THE INP 8 F339, U.S. DC 205564 CT (3150-010 ON DC 2050	VITH THIS 50.0 HRS INTO THE PRIVARD DRMATION NUCLEAR 1001, AND 6), OFFICE
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Cause of Event								
The primary cause of this event w regulating system due to inadequate p	as the improper adjustmen procedural guidance.	t of t	the	SGFP's	lub	e oil p	ressure	
A contributing cause to this event w normal operating configuration.	vas a perceived need to retu	arn the	e SC	GFP lub	e oi	l systen	n to its	
Safety Assessment								
This event is reportable since the M actuation is reportable under 10CFR5	DAFW pumps are an Engin 50.73(a)(2)(iv).	neered	i Sa	fety Fea	ture	e (ESF)	whose	
All systems operated as design 4.								
This event would not have been more	e severe if had occurred unde	er diff	eren	t operat	ing	conditio	ons.	
Corrective Action								
The SGFP lube oil pressure regulating adjusted.	g systems on both Unit 1 and	d Unit	2 h	ave beer	n pr	operly		
Proper procedural guidance has been	enhanced to include lessons	learn	ed fi	rom this	eve	ent.		
This event has been discussed with th	e appropriate personnel.							
Additional Information								
A four-hour notification was made to 10CFR50.72.	to the NRC at 2146 hours	or No	over	nber 5,	199	95 pursi	uant to	
No similar LERs have been reported	by Farley Nuclear Plant.							
The unit returned to power operation	at 0251 on November 8, 19	95.						

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