



**BOSTON EDISON**

Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

E. Thomas Boulette, PhD  
Vice President Nuclear Operations  
and Station Director

April 14, 1992  
BECO Ltr. #92-043

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

License No. DPR-35  
Docket No. 50-293

Subject: March 1992 Monthly Report

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

*E. Thomas Boulette*  
E. Thomas Boulette

WJM/bal

Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator, Region 1  
U.S. Nuclear Regulatory Commission  
475 Allendale Rd.  
King of Prussia, PA 19406

Mr. R. B. Eaton  
Div. of Reactor Projects I/II  
Office of NRR - USNRC  
One White Flint North - Mail Stop 14D1  
11555 Rockville Pike  
Rockville, MD 20852

Senior Resident Inspector

160016

9204160261 920331  
PDR ADOCK 05000293  
R PDR

*TERA*  
*1/1*

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293  
UNIT Pilgrim 1  
DATE April 14, 1992  
COMPLETED BY W. Munro  
TELEPHONE (508) 747-8474

MONTH March 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>666</u>	17	<u>667</u>
2	<u>666</u>	18	<u>665</u>
3	<u>667</u>	19	<u>665</u>
4	<u>666</u>	20	<u>665</u>
5	<u>666</u>	21	<u>665</u>
6	<u>616</u>	22	<u>665</u>
7	<u>665</u>	23	<u>665</u>
8	<u>665</u>	24	<u>665</u>
9	<u>665</u>	25	<u>665</u>
10	<u>665</u>	26	<u>109</u>
11	<u>664</u>	27	<u>0</u>
12	<u>665</u>	28	<u>0</u>
13	<u>664</u>	29	<u>0</u>
14	<u>665</u>	30	<u>0</u>
15	<u>665</u>	31	<u>0</u>
16	<u>665</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE Apr ' 14, 1992  
 COMPLETED BY W. MUNRO  
 TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period March 1992
3. Licensed Thermal Power (Mwt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
None

9. Power Level To Which Restricted, If Any (Net MWe) None

10. Reasons For Restrictions, If Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>744.0</u>	<u>2184.0</u>	<u>169272.0</u>
12. Number Of Hours Reactor Was Critical	<u>614.6</u>	<u>2054.6</u>	<u>100415.2</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>610.6</u>	<u>2050.6</u>	<u>96553.5</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1203408.0</u>	<u>4039992.0</u>	<u>167744472.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>415950.0</u>	<u>1397500.0</u>	<u>56603914.0</u>
18. Net Electrical Energy Generated (MWH)	<u>400576.0</u>	<u>1345773.0</u>	<u>54396721.0</u>
19. Unit Service Factor	<u>82.1</u>	<u>93.9</u>	<u>57.0</u>
20. Unit Availability Factor	<u>82.1</u>	<u>93.9</u>	<u>57.0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>80.4</u>	<u>92.0</u>	<u>48.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>82.2</u>	<u>94.1</u>	<u>49.1</u>
23. Unit Forced Outage Rate	<u>17.9</u>	<u>6.1</u>	<u>12.5</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			

25. If Shut Down At End Of Report Period, Estimated Date of Startup 4/9/92

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

Operational Summary for March 1992

The unit started the reporting period at approximately 100 percent core thermal power (CTP) and maintained that level until March 25 with the exception of a brief downpower on March 6 to perform turbine testing and control rod exercising. On March 25 while performing post work testing following replacement of the RCIC EGR unit, erratic indication of the RCIC Inboard Steam Isolation Valve MO1301-16 was noted. The valve was declared inoperable and the RCIC Outboard Steam Isolation Valve MO1301-17 was closed and deenergized. The decision was made to shut the plant down to investigate and repair the MO1301-16 valve. On March 26 the turbine was tripped and the main generator was taken off the grid at 1033 hours. At 1438 hours the reactor was manually shutdown. There were two Engineered Safety Feature (ESF) actuations on March 26 and one on March 27 during the controlled shutdown. High water level spiking occurred on the reactor water level instruments which caused two of the Group 1 isolations. The other Group 1 isolation was caused when the Main Steam Isolation Valves were opened causing a pressure transient resulting in a level increase. A root cause analysis team was formed to investigate the cause of the spiking. The unit ended the reporting period in cold shutdown with a forced outage in progress making repairs that could not be performed on line. Minor power reductions were initiated on March 13 and 20 to perform control rod exercises.

Safety Relief Valve Challenges  
Month of March 1992

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

## REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 3, 1993
3. Scheduled date for restart following next refueling: June 8, 1993
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage and consisted of 168 assemblies.
7. (a) There are 580 fuel assemblies in the core.  
(b) There are 1489 fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.  
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

Month March 1992

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

---

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
---------------	------------------	--------------------	--------------	--------------------	--	---------------------------

---

No Major Safety Related Maintenance was completed during the reporting period. Per the guidelines in CDW: 3.02-03 Major Safety Related Maintenance is reported only after the work has been completed and the Maintenance Request is closed.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293  
 NAME Pilgrim 1  
 DATE April 14, 1992  
 COMPLETED BY W. Munro  
 TELEPHONE (508) 747-8474

REPORT MONTH March 1992

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
03	3/26/92	F	133.4	A	1	92-003-00	BN	ISV	Failure of MOV 1301-16 valve. Forced outage.

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)