

ACRS-2961

PDR 11/30/95

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January 12-13, 1995

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A. Reports, Letters, and Memoranda

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ACRS Report to Congress on the NRC Safety Research Program (Letters to the Honorable Albert Gore, Jr., President of the United States Senate, and the Honorable Newt Gingrich, Speaker of the United States House of Representatives, from T. S. Kress, ACRS Chairman, dated January 24, 1995)

Proposed Amendments to 10 CFR Part 26, "Fitness for Duty Programs" (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 18, 1995)

Proposed NRC Generic Letter Regarding NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

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Proposed NRC Generic Letter Regarding Guidance on Developing Acceptable Inservice Testing Programs (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

Proposed Generic Letter Regarding Pressure Locking and Thermal Binding of Gate Valves (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

- B. Reconciliation of ACRS Comments and Recommendations
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MINUTES OF THE 417TH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
JANUARY 12-13, 1995
ROCKVILLE, MARYLAND

The 417th meeting of the Advisory Committee on Reactor Safeguards was held at Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on January 12-13, 1995. The purpose of this meeting was to discuss and take appropriate action on the items listed in the attached agenda. The meeting was open to public attendance. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, N.W., Washington, D.C. 20005.]

ATTENDEES

ACRS Members: Dr. Thomas S. Kress (Chairman), Dr. Robert L. Seale (Vice-Chairman), Mr. James C. Carroll, Dr. Ivan Catton, Mr. William J. Lindblad, Mr. Carlyle Michelson, Dr. Dana A. Powers, Dr. William J. Shack, and Mr. Charles J. Wylie. [For a list of other attendees, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Thomas S. Kress, Committee Chairman, opened the meeting at 8:30 a.m. and reviewed the schedule for the meeting.

The staff to the Advisory Committee on Nuclear Waste introduced themselves to the ACRS members.

Dr. Kress announced that the National Academy of Sciences Committee on the Use of Digital Instrumentation and Control will meet on January 31 - February 1, 1995, in Washington, D.C. Dr. Kress announced that he will address the NAS Committee during the first day of the meeting. Also, Dr. Shack is expected to attend.

II. GENERAL ELECTRIC NUCLEAR ENERGY (GENE) TEST AND ANALYSIS PROGRAM FOR THE SIMPLIFIED BOILING WATER REACTOR (SBWR) DESIGN
(Open)

[Note: Mr. Paul Boehnert was the Designated Federal Official for this portion of the meeting.]

Dr. Ivan Catton, cognizant Member, introduced this topic to the Committee. He noted the following points:

- In response to long-standing NRC staff concerns with the GENE test and analysis program (TAP) being conducted in support of the SBWR design certification, GENE prepared a Test and Analysis Program Description (TAPD).
- The TAPD is a "road map" that ensures the validation and verification of the GE computer code TRACG for use in the SBWR design certification.
- The NRC staff has prepared a Draft Safety Evaluation Report (DSER) on the adequacy of the TAPD relative to ensuring the adequacy of the GENE TAP. The staff has concluded that, subject to a few caveats, the GENE TAP is adequate to support the SBWR design certification.
- As a result of a review by the Thermal Hydraulic Phenomena Subcommittee of this matter, two major concerns were identified: provisions are lacking to ensure the proper treatment of phenomena associated with the presence of noncondensable gases in containment, and GENE has not performed an adequate scaling analysis to ensure that its test facilities will provide the necessary data.

NRC Staff Presentation

Mr. Robert Jones, NRR, discussed the review of the GENE TAPD and the results of same as documented in the NRR DSER. The conclusions of the NRR review were:

- GENE's Phenomena Identification and Ranking Table is inadequate at this time; additional consideration of phenomena other than those ranked "High" by GENE is needed.
- Although the TAPD represents a logical approach to defined testing needs, additional work is required in the following areas: scaling analysis, testing scope for both reactor systems and containment tests, demonstration that the range of test conditions covers those of the actual plant design parameters, and additional details of the data usage for qualification of TRACG.
- The staff's final approval as to the acceptability of the TAP will depend on a satisfactory response by GENE to the above issues.

In response to Dr. Catton, Mr. Jones said that neither GENE nor NRC has performed a scaling analysis to ensure that the GENE test facilities will provide the necessary data. In response to a question from Mr. Michelson, Mr. Jones indicated that NRR will evaluate the issue of ensuring adequate core cooling given a break in the reactor water cleanup (RWCU) line outside of containment. Dr. Catton urged NRR to conduct its own analysis of this break. The NRR staff indicated that they will do so.

GENE Presentation

Representatives of GENE made presentations to the Committee on the following topics:

- Meeting Objectives
- SBWR System Response
- Comments on the NRC DSER on the SBWR TAP
- Technical Basis of SBWR Integral System Response.

The key points made by GENE in these presentations included:

- Calculations of limiting accidents for the SBWR design show that adequate core cooling is maintained.
- Regarding the NRC DSER, GENE said that they will provide responses to all requesters for information. This new information will be provided to the staff and ACRS via test reports, a revision of the TAPD, and Supplements to the TRACG Application and Code Qualification Licensing Topical Reports. Issues pertaining to PANTHERS and PANDA test programs will be resolved, as will design issues identified by NRR (stuck-open vacuum breaker event, debris clogging of the passive containment cooling system). Finally, GENE has yet to address the issue of the need for additional Gravity Driven Cooling System (GDCS) related testing.
- Regarding the need for this additional testing noted above, GENE's position is that such testing is not needed. However, it was noted that GENE and Toshiba are exploring the potential for conducting additional testing in the GIRAFFE facility in order to resolve this matter.

Mr. Michelson asked GENE if they have considered the addition of a third valve to the RWCU line, as was done for the Advanced Boiling Water Reactor (ABWR) design, in order to resolve the above-noted concern here. Mr. James Healzer, GENE, indicated that he would

investigate this matter and provide a response. Mr. Carroll asked if the bottom (vessel) drain line can be isolated, given a pipe break. The GENE staff said that they would investigate and respond later.

Mr. Carroll also asked if the isolation condenser can be isolated should a break(s) occur in the heat exchanger tubing. No immediate reply from the GENE staff was available; Mr. Carroll indicated that the Committee will pursue this matter until a satisfactory response is forthcoming.

Committee Caucus

Dr. Catton noted that he has drafted a letter on this matter for the Committee's consideration. He opined that he believes that the TAPD represents a good start, but, more work by GENE is necessary to achieve closure. Three concerns stand out:

1. Scaling has been done after testing commenced, which is backwards from how it should have been performed. A global scaling analysis still needs to be conducted.
2. Modeling of foreign gases (noncondensibles) represents a strong challenge to the TRACG code. Further, the tests being planned by GENE to obtain the necessary data suffer from a lack of adequate instrumentation.
3. On the issue of the addressing of the intermediate cooling phase (GDCS-cooling period), NRR has stated that more testing is necessary; GENE has agreed. Speaking for himself, Dr. Catton noted his ambivalency concerning the need for such testing.

Dr. Catton said that he believes that the Committee should not endorse the DSER until GENE has addressed the open issues noted above.

Follow-up Items

The following items have been identified for follow-up:

- Mr. Michelson asked GENE if they have considered the addition of a third valve to the RWCU line, as was done for the ABWR design, in order to resolve the above-noted concern here. Mr. Heazler of GENE indicated that he would investigate this matter and provide response.
- Mr. Carroll asked if the bottom (vessel) drain line can be isolated, given a pipe break. GENE said that they would investigate and respond later.

- Mr. Carroll asked if GENE has the capability to isolate the isolation condenser should a rupture(s) occur in the heat exchange tubes.

Note: These items have been passed along to GENE for response.

Conclusion

The Committee issued a report to James M. Taylor, Executive Director for Operations, dated January 19, 1995, on this matter.

III. STATUS OF THE AP600 DESIGN CERTIFICATION REVIEW (Open)

[Note: Mr. Noel Dudley was the Designated Federal Official for this portion of the meeting.]

Mr. William Lindblad, Chairman of the Westinghouse Standard Plant Design Subcommittee, introduced the subject by stating that the staff had issued the DSER on the AP600 nuclear power plant design and that the Subcommittee had met on January 11, 1995. He stated that the applicant needed to submit substantial information and that the Subcommittee would proceed with the technical review, but without a firm schedule on completing that review.

Mr. Thomas J. Kenyon, NRR, provided a summary of the staff's review of the AP600 design. The DSER, issued in November 1994, identifies about 1100 open items and seven global issues that affect multiple aspects of the design application. The global issues identified in the DSER are the regulatory treatment of non-safety-related systems; passive system reliability; determination of an appropriate source term; review of the Westinghouse design certification testing program; inspections, tests, analysis and acceptance criteria (ITAAC); combined operating license action items; and a proposed reduction in proprietary information associated with the AP600 design review.

Mr. Kenyon discussed the expedited review schedule. According to this expedited schedule, the Final Safety Evaluation Report would be presented to the ACRS in May 1996. An ACRS letter would be needed in August 1996 to support a Final Design Approval (FDA) date of September 1996. Staff approval of the Design Certification Document and issuance of the proposed rule is scheduled for December 1996.

Mr. Brian A. McIntyre, Westinghouse Electric Corporation, outlined the Westinghouse response to the DSER. Westinghouse is working with the staff to resolve the open items and the seven global issues. Westinghouse believes that the global issues can be resolved in time to support the FDA date of September 1996.

The Committee discussed the Westinghouse focused PRA, the source term, the number of ITAACs, and combined operating license issues.

Conclusion

This briefing was for information only. The Westinghouse Standard Plant Design Subcommittee plans to proceed with the AP600 design review, but made no commitment to a fixed ACRS review schedule. The ACRS review schedule depends primarily on the timely availability of additional updated information both from Westinghouse and the NRC staff.

IV. CANDU 3U APPLICATION ACCEPTANCE REVIEW (Open)

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

Dr. Gail Marcus, NRR, briefed the Committee regarding the CANDU 3U application acceptance review. The Atomic Energy of Canada Limited Technologies (AECLT) submitted its application for final design approval and design certification (DC) under 10 CFR Part 52. The contents of the application were provided in the form of a CANDU 3U Safety Analysis Report. The application acknowledged that certain information was missing such as the inspections, tests, analyses, and acceptance criteria, technical specifications, severe accident mitigative design alternatives, and the failure modes and effects analyses. The required PRAs (Levels II and III), and the design acceptance criteria were also not included in the application.

The NRC staff has performed its acceptance review of the CANDU 3U application and on December 15, 1994, the staff issued its letter to the AECLT stating that a significant amount of information is missing, or in a form that would cause the staff to expend a great deal more resources to complete the DC review than previously anticipated.

Previously, on March 24, 1994, the staff issued SECY-94-079, "Schedule and Resource Estimates for CANDU 3 Design Certification Review." This SECY included estimates of 105 full-time equivalent (FTE) employees and \$2.2 million for NRR, and 23 FTE and \$18 million for RES. However, these estimates were based on how closely the application complies with the information required by the Standard Review Plan, and the results of the ongoing agency reevaluation of the NRC fee structure related to research needed to support the CANDU 3 DC.

Conclusion

The Committee took no action based on this information briefing.

V. MEETING WITH THE DIRECTOR OF THE NRC OFFICE OF NUCLEAR REGULATORY RESEARCH (Open)

[Note: Mr. Dean Houston was the Designated Federal Official for this portion of the meeting.]

Dr. Kress welcomed Mr. Eric Beckjord, Director of the Office of Nuclear Regulatory Research (RES) and expressed the Committee's appreciation for his willingness to meet with them to discuss items of mutual interest.

Mr. Beckjord discussed the accomplishments of the RES since 1986. Specifically, he described the history and status of the following programs:

- Reactor Aging and License Renewal
- Reactor Accident Analysis: Severe Accidents
- Advanced LWR Confirmatory Research
- Risk Assessment
- Human Reliability
- Digital Instrumentation and Control
- Seismic Research
- Plant Performance
- Safety Issue Resolution and Regulatory Improvement
- International Research Cooperation

During this discussion, Dr. Catton offered to provide RES a copy of his trip report on the 4th International Symposium on Fire Safety Science, June 12-17, 1994. [This report was provided by memorandum dated January 17, 1995.]

Mr. Beckjord addressed the topics that had been requested by the Committee. These were:

- Potential Research for the CANDU 3 Design
- Overview of the NRC Code Development
- Overview of Steam-Explosion Research

With regard to the CANDU 3 design, he discussed a proposed confirmatory research plan, the NSRRC review of this plan, and the estimated cost for the plan (about \$18 million). He also indicated that the CANDU 3 design certification may be taking a back seat to the CANDU 9 design since Korea has ordered one.

Mr. Beckjord presented a status report of the following codes being developed by RES: RELAP5, TRAC-P, RAMONA, MELCOR, SCDAP/RELAP5, CONTAIN, and VICTORIA. He briefly discussed COMMIX and HMS codes.

During the discussion on steam explosion research, he informed the Committee that RES was planning a meeting of Steam Explosion Experts in June 1995. The Committee asked that it not be scheduled on the same dates as the ACRS Meeting, June 8-10, 1995, and RES agreed to schedule it for the following week, tentatively June 15-16, 1995.

In closing, he discussed the prospects for NRC research during the next 10 years. He indicated that an RES Paper describing this plan will be available soon and he will provide ACRS with a copy [This paper was provided to ACRS by memorandum dated January 23, 1995]. He took this opportunity to give the Committee an assessment of some past ACRS reports. Specifically, he made remarks about the reports on ECCS (world class); piping supports, hangers and snubbers (not as good), and ROSA tests for the AP600 (paradox). He stated that if the ROSA tests were not needed, why is it that the myriad of questions that have come forth now could only be answered by doing the ROSA tests?

Dr. Kress and Mr. Lindblad thanked him for his presentation and complimented him for his long and illustrative career with industry and NRC.

Conclusion

This was an informational briefing only. No Committee action was taken on this matter. The Committee concurred on the dates proposed by RES for a meeting of the Steam Explosion Experts in June 1995.

VI. TURBINE OVERSPEED STUDY (Open)

[Note: Mr. Douglas Coe was the Designated Federal Official for this portion of the meeting.]

Mr. James C. Carroll, Chairman of the Plant Operations Subcommittee, introduced this topic to the Committee.

NRC Staff Presentation

Dr. Harold Ornstein, Office for Analysis and Evaluation of Operational Data (AEOD), discussed AEOD Special Study S94-02 "Turbine-Generator Overspeed Protection Systems at U.S. Light-Water Reactors," dated September 30, 1994, which he authored. Important topics covered were:

Description of the Salem Unit 2 turbine overspeed failure of November 9, 1991.

Method and findings of the AEOD turbine overspeed study regarding turbine overspeed protection systems (TOPS), including licensee complacency, unrecognized common-cause vulnerabilities, poor testing practices, insufficient manufacturer guidance, and inaccurate probabilistic assumptions.

Turbine overspeed events/precursors at St. Lucie, Comanche Peak, Big Rock Point, Diablo Canyon, Byron, Vandellos (Spain), Narora (India), and Fermi 2 Nuclear Power Plants.

Current NRC requirements, and the significance of plant vulnerabilities associated with the fires, explosions, flooding, and vibration caused by turbine overspeed and turbine missile ejection events.

NRC staff initiative to study turbine building hazards.

The Committee asked various questions related to: the use of fire brigades to fight turbine building fires, the potential impact of generator overvoltage resulting from turbine overspeed or exciter failure, safety hazards associated with flooding incidental to turbine missile ejection, foreign experience, risk analysis of turbine building hazards, and methods for fighting hydrogen fires.

Mr. Carroll asked the staff for a copy of NRR's user need request to RES regarding the identification of potential failure modes of turbine rotors using integral blade/rotor design. Additionally, he asked for further details on the staff's plan to evaluate the impact on turbine failures through the Individual Plant Examination of External Events review process.

Conclusion

The Committee took no action based on this information briefing.

VII. ANNUAL ACRS REPORT TO CONGRESS (Open)

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed a draft annual ACRS report to Congress on the NRC Safety Research Program.

Conclusion

The Committee provided reports on this matter to the Honorable Albert Gore, Jr., President of the United States Senate, and Newt Gingrich, Speaker of the United States House of Representatives, from T.S. Kress, ACRS Chairman, dated January 24, 1995

VIII. EXECUTIVE SESSION (Open)

[Note: Dr. John Larkins was the Designated Federal Official for this portion of the meeting.]

A. Reports, Letters and Memoranda

General Electric Nuclear Energy Test and Analysis Program for the Simplified Boiling Water Reactor Design (Report to James M. Taylor, Executive Director for Operations, from T. S. Kress, ACRS Chairman, dated January 19, 1995)

ACRS Report to Congress on the NRC Safety Research Program (Letters to the Honorable Albert Gore, Jr., President of the United States Senate, and the Honorable Newt Gingrich, Speaker of the United States House of Representatives, from T. S. Kress, ACRS Chairman, dated January 24, 1995)

Proposed Amendments to 10 CFR Part 26, "Fitness for Duty Programs" (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 18, 1995)

Proposed NRC Generic Letter Regarding NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

Proposed NRC Generic Letter Regarding Guidance on Developing Acceptable Inservice Testing Programs (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

Proposed Generic Letter Regarding Pressure Locking and Thermal Binding of Gate Valves (Memorandum to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995)

B. Reconciliation of ACRS Comments and Recommendations

[Note: Mr. Sam Duraiswamy was the Designated Federal Official for this portion of the meeting.]

The Committee discussed the response from the NRC Executive Director for Operations to ACRS comments and recommendations included in a recent ACRS report:

EDO letter dated December 28, 1994, responding to the ACRS report dated November 11, 1994, concerning NRC Test and Analysis Programs in Support of AP600 and SBWR Advanced Light Water Reactor Passive Plant Design Certification Reviews.

The Committee decided that it was satisfied with the EDO's response.

C. Report on the Meeting of the Planning and Procedures Subcommittee Held on January 11, 1995 (Open)

The Committee heard a report from Dr. Kress on the Planning and Procedures Subcommittee meeting held on January 11, 1995. The following items were discussed:

Status of Member Nominations - Dr. Kress noted that the ACRS staff has narrowed the original 440 applications down to about 25 to be reviewed by the Commission's Screening Panel. The Screening Panel has discussed the proposed rating criteria for evaluating the candidates and expects to meet next week to rank the candidates and propose a slate of nominees for Commission consideration in February 1995.

The Committee recommended that the Screening Panel be given a list of nominees recommended by the Committee along with a brief statement why each nomination would be of significant value to the Committee.

Status of the Fellows Program - Dr. Kress announced that both OGC and OP refused the employment condition requests by the candidate selected for the ACRS Fellows position. The candidate was informed verbally and by letter. If the candidate does not accept the existing offer of employment, the ACRS staff is preparing to reinitiate the solicitation process to recruit an ACRS Senior Fellow to fill the existing vacancy. The staff is looking at applications submitted in response to previous vacancy announcements for a senior fellow, and also the applications for ACRS membership to identify potential fellows.

Members' Handbook - Dr. Kress stated that copies of the final draft of the Committee Members' Handbook have been forwarded to the EDO, OGC, OP, ADM and the IG for comment. To date, the staff has received comments from the IG. Comments from the OP and other offices are expected within the next two weeks. A reconciliation of comments from the NRC staff offices will be conducted by the Planning and Procedures Subcommittee and if needed, unresolved policy issues will be brought to the full Committee for discussion.

Performance Measures for the NRC Advisory Committees - Dr. Kress stated that, in response to a request from OMB and the NRC Advisory Committee Management Officer, dated August 15, 1994, a draft Performance Measures for the ACRS and ACNW was prepared by the staff and provided to the Committee members last month for consideration. The members were reminded that comments should be submitted to the staff during the meeting. Following this meeting, the staff will finalize the draft and submit the Performance Measures to the Commission.

Annual ACRS Report to Congress - Dr. Kress reported that the Committee's request to eliminate the Annual ACRS Report to Congress was not approved by the 103rd Congress. In the proposed legislative package for the 104th Congress, NRC requested that it be changed to the Biennial ACRS Report to Congress. Until Congress makes a change, the Annual ACRS Report to Congress will continue to be issued.

Travel Requests by Members - The Committee approved funding support for Dr. Catton to attend the Structural Mechanics in Reactor Technology Conference and Post Conference Seminar on Current Issues in PRA, August 13-23, 1995, in Porto Alegre, Brazil. The Committee also approved funding support for Dr. Kress to attend the International Meeting on Fission Product Release and Transport, May 21-26, 1995, in Cesme, Turkey.

D. Members' Issues (Closed)

The Committee discussed possible responses to several statements published in the Semiannual Report of the Office of the Inspector General, April 1 - September 30, 1994. The Committee agreed that a response should not be from the Committee. A letter from individual ACRS members was forwarded to the Commission with specific comments and requesting Commission action on the recommendations.

E. Future ACRS Activities

The Committee decided to review the proposed NRC Generic Letter Regarding Pressure Locking and Thermal Binding of Gate Valves following reconciliation of public comments by the staff. The Committee has no objection to the NRC staff proposal to issue the proposed rule for public comment. [A memorandum was issued to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995.]

The Committee decided to review the proposed Amendments to 10 CFR Part 26, "Fitness for Duty Programs" after the staff has reconciled the public comments. The Committee has no objec-

tion to the NRC staff proposal to issue the proposed rule for public comment. [A memorandum was issued to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 18, 1995.]

The Committee decided not to review the proposed NRC Generic Letter regarding NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities. [A memorandum was sent to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995.]

The Committee decided not to review the proposed NRC Generic Letter regarding Guidance on Developing Acceptable Inservice Testing Programs. [A memorandum was sent to James M. Taylor, Executive Director for Operations, from John T. Larkins, Executive Director, dated January 19, 1995.]

During the discussion of the status of shutdown nuclear power plants, it was noted that Indian Point 3 nuclear power plant is scheduled to restart soon. Several ACRS members indicated that they would like to invite the licensee to brief the Committee after the restart activities are completed, perhaps in April or May 1995.

F. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 418th ACRS Meeting, February 9-10, 1995.

The 417th ACRS meeting was adjourned at 6:15 p.m. on Friday, January 13, 1995.

information concerning individuals associated with the nominations and proposals. These matters are exempt under 5 U.S.C. 552b(c) (4) and (8) of the Government in the Sunshine Act.

Dated: December 22, 1994.

Linda Allen-Benton,
Deputy Director, HRM.

[FR Doc. 94-31906 Filed 12-27-94; 8:45 am]
BILLING CODE 7550-01-02

NUCLEAR REGULATORY COMMISSION

Nuclear Safety Research Review Committee; Meeting of Materials and Engineering Subcommittee

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of meeting.

The NSRRC Materials and Engineering Subcommittee will hold a meeting on January 24, 1995 in Room T-2B1, Two White Flint North (TWFN) Building, 11545 Rockville Pike, Rockville, MD.

The entire meeting will be open to public attendance.

The review to be conducted by this newly formed Subcommittee will address seismic, containment structure, and materials issues of prominent current interest. The agenda will be as follows:

January 24

- 8:30-9:00 Introductory remarks (Chairman; Members: Director, Division of Engineering Technology, Office of Nuclear Regulatory Research)
- 9:00-10:30 Seismic issues (seismic hazard methodology; new ASME code criteria for seismic design of piping)
- 10:45-11:15 Subcommittee discussion (seismic issues)
- 11:15-12:00 Containment corrosion, inspection, and safety margins
- 1:15-2:45 Materials issues (pressurized thermal shock; cracking of BWR core shroud materials)
- 3:00-5:00 Subcommittee discussion (issues other than seismic)

The Subcommittee will report to the full Committee on the facts and analyses discussed at the meeting.

A detailed agenda will be made available at the meeting.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Subcommittee. Questions may be asked only by members of the Committee and the staff. Persons desiring to make oral statements should notify the Nuclear Regulatory Commission staff member named below as far in advance as is

practicable so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee may exchange preliminary views regarding matters to be considered during the balance of the meeting. The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff regarding this review.

Further information regarding topics to be discussed, the scheduling of sessions, whether the meeting has been canceled or rescheduled, and the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefore can be obtained by a prepaid telephone call to Mr. George Sege (telephone 301/415-6593) between 8:00 a.m. and 4:30 p.m. (EST). Persons planning to attend this meeting are urged to contact the above named individual one or two days before the scheduled meeting to be advised of any changes in schedule, etc., that may have occurred.

Dated: December 20, 1994.

George Sege,

Technical Assistant to the Director, Office of Nuclear Regulatory Research.

[FR Doc. 94-31856 Filed 12-27-94; 8:45 am]
BILLING CODE 7550-01-02

Advisory Committee on Reactor Safeguards; Meeting Agenda

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on January 12-14, 1995, in Conference Room T2B3, 11545 Rockville Pike, Rockville, Maryland.

Thursday, January 12, 1995

8:30 a.m.-8:45 a.m.: *Opening Remarks by the ACRS Chairman* (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 a.m.-10:45 a.m.: *General Electric Nuclear Energy (GENE) Simplified Boiling Water Reactor (SBWR) Test and Analysis Programs* (Open/Closed)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and GENE regarding the results of the NRC staff evaluation of the test and analysis programs being conducted by GENE in support of the certification of the SBWR passive plant design.

A portion of this session may be closed to discuss GENE proprietary information applicable to this matter.

11:00 a.m.-12:00 Noon: *Status of AP600 Design Certification Review* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the status of the AP600 design certification review.

Representatives of Westinghouse will participate, as appropriate.

2:00 p.m.-3:00 p.m.: *CANDU3 Application Acceptance Review* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding staff findings resulting from its CANDU3 application acceptance review.

Representatives of the Atomic Energy of Canada Limited Technologies (AECLT) will participate, as appropriate.

3:00 p.m.-4:00 p.m.: *Report of the Planning and Procedures Subcommittee* (Open/Closed)—The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business and internal organizational and personnel matters relating to the ACRS staff members.

A portion of this session may be closed to discuss matters that relate solely to internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

4:15 p.m.-4:45 p.m.: *Future ACRS Activities* (Open)—The Committee will discuss topics proposed for consideration during future ACRS meetings.

4:45 p.m.-5:00 p.m.: *Reconciliation of ACRS Comments and Recommendations* (Open)—The Committee will discuss responses from the NRC Executive Director for Operations to ACRS comments and recommendations included in recent ACRS reports.

5:00 p.m.-6:30 p.m.: *Preparation of ACRS Reports* (Open)—The Committee will discuss proposed ACRS reports on certain matters considered during this meeting, including a possible report on Performance-Based Regulation.

Friday, January 13, 1995

8:30 a.m.-8:35 a.m.: *Opening Remarks by the ACRS Chairman* (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 a.m.-10:45 a.m.: *Meeting with the Director of the NRC Office of Nuclear Regulatory Research* (Open)—

The Committee will hear presentations by and hold discussions with the Director of the NRC Office of Nuclear Regulatory Research regarding items of mutual interest.

11:00 a.m.-12:00 Noon: Turbine Overspeed Study (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the results of the Turbine Overspeed Study performed by the NRC Office for Analysis and Evaluation of Operational Data (AEOD).

1:00 p.m.-6:30 p.m.: Preparation of ACRS Reports (Open)—The Committee will discuss proposed ACRS reports on certain matters considered during this meeting.

Saturday, January 14, 1995

8:30 a.m.-11:00 a.m.: Preparation of ACRS Reports (Open)—The Committee will continue its discussion of proposed ACRS reports on certain matters considered during this meeting.

11:15 a.m.-11:45 a.m.: New Research Needs (Open)—The Committee will discuss new research needs, if any, identified during this meeting.

11:45 a.m.-12:00 Noon: Miscellaneous (Open)—The Committee will discuss miscellaneous matters related to the conduct of Committee activities and complete discussions of topics that were not completed during previous meetings as time and availability of information permit.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Register on October 5, 1994 (59 FR 54780). In accordance with these procedures, oral or written statements may be presented by members of the public; electronic recordings will be permitted only during the open portions of the meeting, and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify the ACRS Executive Director, Dr. John T. Larkins, at least five days before the meeting if possible, so that appropriate arrangements can be made to allow the necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set aside for this purpose may be obtained by contacting the ACRS Executive Director prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check

with the ACRS Executive Director if such rescheduling would result in major inconvenience.

I have determined in accordance with Subsection 10(d) Public Law 92-463 that it is necessary to close portions of this meeting noted above to discuss proprietary information per 5 U.S.C. 552b(c)(4); information that involves the internal personnel rules and practices of this Advisory Committee per 5 U.S.C. 552b(c)(2); and to discuss information the release of which would constitute a clearly unwarranted invasion of personal privacy per 5 U.S.C. 552(c)(6).

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the ACRS Executive Director, Dr. John T. Larkins (telephone 301-415-7361), between 7:30 A.M. and 4:15 P.M. EST.

Proposed ACRS Meeting Dates for CY 1995—The proposed ACRS meeting dates for CY 1995 are provided below.

ACRS meeting No.	1995 ACRS meeting dates
418	February 9-11, 1995.
419	March 9-11, 1995.
420	April 6-8, 1995.
421	May 4-6, 1995.
422	June 8-10, 1995.
423	July 13-15, 1995.
424	August 10-12, 1995.
425	September 7-9, 1995.
426	October 5-7, 1995.
427	November 2-4, 1995.
428	December 7-8, 1995.

Dated December 21, 1994.

Andrew L. Eckers,
Advisory Committee Management Officer.
[FR Doc. 94-31954 Filed 12-27-94; 6:05 am]
BILLING CODE 7880-04-0

Docket Nos. 80-351; 90-325 and 80-334; 80-400

Carolina Power & Light Co.; (H.B. Robinson Steam Electric Plant, Unit No. 2, Brunswick Steam Electric Plant, Units 1 and 2; Shearon Harris Nuclear Power Plant, Unit 1); Exemption

Carolina Power & Light Company (CP&L or the licensee) is the holder of Facility Operating License Nos. DPR-71 and DPR-62 for Brunswick Steam Electric Plant, Units 1 and 2, (Brunswick) located in Brunswick County, North Carolina; Facility Operating License No. DPR-23 for H.B. Robinson Steam Electric Plant, Unit No. 2, (Robinson) located in Darlington

County, South Carolina; and Facility Operating License No. NPP-63 for Shearon Harris Nuclear Power Plant, Unit 1, (Harris) located in Wake and Chatham Counties, North Carolina. The licenses authorize the operation of the above facilities.

The licenses provide, among other things, that the licensee is subject to all rules, regulations and Orders of the Nuclear Regulatory Commission (the Commission or NRC) now or hereafter in effect. The facilities consist of two boiling water reactors located at the licensee's Brunswick site, and two pressurized water reactors, one located at the Robinson site and one at the Harris site.

Paragraph (a) of Section 73.55, "Requirements for physical protection of licensed activities in nuclear power reactors against radiological sabotage," of Title 10 of the Code of Federal Regulations (10 CFR) states, in part, that "the licensee shall establish and maintain an onsite physical protection system and security organization that will have as its objective to provide high assurance that activities involving special nuclear material are not inimical to the common defense and security and do not constitute an unreasonable risk to the public health and safety."

Paragraph (1) of Section 73.55(d), "Access Requirement," specifies that "The licensee shall control all points of personnel and vehicle access into a protected area." Section 73.55(d)(5) requires that "A numbered picture badge identification system shall be used for all individuals who are authorized access to protected areas without escort." Section 73.55(d)(5) also states that an individual not employed by the licensee (i.e., a contractor) may be authorized access to protected areas without escort provided the individual "receives a picture badge upon entrance into the protected area which must be returned upon exit from the protected area . . ."

The licensee proposed to implement an alternative unescorted access control system that would eliminate the need to issue and retrieve badges at each entrance/exit location and would allow all individuals with unescorted access to keep their badge with them when departing the site.

An exemption from certain requirements of 10 CFR 73.55(d)(5) is required to allow contractors who have unescorted access to take their badges offsite instead of returning them when exiting the site. By letters dated July 29, 1994, and December 3, 1994, and December 5, 1994, the licensee requested an exemption from certain



APPENDIX II

UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

December 30, 1994

SCHEDULE AND OUTLINE FOR DISCUSSION
417TH ACRS MEETING
January 12-14, 1995

Thursday, January 12, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

1) 8:30 - 8:45⁴⁰ A.M.

[Opening Remarks by the ACRS Chairman (Open)
1.1) Opening Statement (TSK/SD)
1.2) Items of Current Interest (TSK/JTL/SD)
1.3) Priorities for Preparation of ACRS Reports (TSK/SD)

2) 8:45⁴⁰ - 10:45⁵⁰ A.M.

[General Electric Nuclear Energy (GENE) Simplified Boiling Water Reactor (SBWR) Test and Analysis Programs (Open/Closed) (IC/PAB)
2.1) Remarks by the Subcommittee Chairman
2.2) Briefing by and discussions with representatives of the NRC staff and GENE regarding the results of the NRC staff evaluation of the test and analysis programs being conducted by GENE in support of the certification of the SBWR passive plant design.

~~A portion of this session may be closed to discuss GENE proprietary information applicable to this matter.~~

10:45⁵⁰ - 11:00⁰⁵ A.M.

BREAK

3) 11:00⁰⁵ - 11:45^{11:45} NOON

[Status of AP600 Design Certification Review (Open) (WJL/NFD)
3.1) Remarks by the Subcommittee Chairman
3.2) Briefing by and discussions with representatives of the NRC staff regarding the status of the AP600 design certification review.

Representatives of Westinghouse will participate, as appropriate.

11:45⁰⁵ - 12:00⁰⁵ P.M.

LUNCH

[= Transcribed portion of the meeting

4) 2:00⁰⁵ - 3:00 P.M.

CANDU3 Application Acceptance Review (Open)
(RLS/MME)

- 4.1) Remarks by the Subcommittee Chairman
4.2) Briefing by and discussions with representatives of the NRC staff regarding staff findings resulting from its CANDU3 application acceptance review

Representatives of the Atomic Energy of Canada Limited Technologies will participate, as appropriate.

5) 3:00²⁵ - 4:00²⁵ P.M.

Report of the Planning and Procedures Subcommittee (Open/Closed) (TSK/JTL)

Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS staff members.

(Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.)

3:30 4:00
~~4:00~~ - ~~4:15~~ P.M.

BREAK

6) 4:15⁰⁰ - 4:45⁵⁰ P.M.

Future ACRS Activities (TSK/SD) (Open)

Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings

7) 4:45^{2:45} - 5:00^{2:50} P.M.

Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et al./SD, et al.)

Discussion of responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports

8) 5:00^{4:50} - 6:30⁴⁵ P.M.

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 8.1) GENE test and analysis programs for the SBWR Design (IC/PAB)
8.2) Annual Report to Congress on the NRC Safety Research Program (RLS/SD/MME)

4:50 - 6:45 P.M. Closed

8.3) Performance-Based Regulation (Tentative) (DAP/JCC/MDH)

Friday, January 13, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

- 9) 8:30 - 8:35 A.M. Opening Remarks by the ACRS Chairman (Open)
(TSK/SD)
- 10) 8:35 - 10:³⁵~~45~~ A.M. Meeting with the Director of the NRC Office of Nuclear Regulatory Research (RES) (Open)
(TSK/MDH)
- 10.1) Remarks by the ACRS Chairman
- 10.2) Briefing by and discussions with Mr. Beckjord, RES Director, regarding items of mutual interest including the following:
- Mr. Beckjord's reflections on the NRC research program
 - Potential research issues for the CANDU 3 design
 - Overview of the NRC Code developmentment
 - Overview of the steam explosion research
- 10:³⁵~~45~~ - 11:00 A.M. BREAK
- 11) 11:00 - 12:⁰⁵~~00~~ Noon Turbine Overspeed Study (Open) (JCC/DHC)
- 11.1) Remarks by the Subcommittee Chairman
- 11.2) Briefing by and discussions with representatives of the NRC staff regarding the results of the study performed by the NRC Office for Analysis and Evaluation of Operational Data (AEOD)
- 12:⁰⁵~~00~~ - 1:²⁰~~00~~ P.M. LUNCH
- 12) 1:²⁰~~00~~ - 6:¹⁰~~30~~ P.M. Preparation of ACRS Reports (Open/Closed)
- ~~2:45~~ - ~~3:00~~ P.M. BREAK Discussion of proposed ACRS Reports on:
- ~~3:00~~ ~~3:15~~
- 12.1) GENE Test and Analysis Programs for the SBWR Design (IC/PAB)
- 12.2) Annual Report to Congress on the NRC Safety Research Program (RLS/SD/MME)
- 12.3) Performance-Based Regulation (DAP/JCC/MDH) (Tentative)
- 3:25-6:10 closed

~~Cancelled~~

Saturday, January 14, 1995, Conference Room 2B3, Two White Flint North, Rockville, Maryland

- 13) 8:30 - 11:00 A.M. Preparation of ACRS Reports (Open)
 Discussion of the Proposed ACRS reports listed under Item 12
- 11:00 - 11:15 A.M. BREAK
- 14) 11:15 - 11:45 A.M. New Research Needs (Open) (TSK/SD)
 Discussion of new research needs, if any, identified during this meeting
- 15) 11:45 - 12:00 NOON Miscellaneous (Open) (TSK/JTL)
 Complete discussion of matters considered during this meeting and matters considered but not completed during previous meetings as time and availability of information permit

- NOTE:
- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
 - Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

417TH ACRS MEETING
JANUARY 12-14, 1995

NRC STAFF

E. Beckjord	RES
R. Borchardt	NRR
W. Burton	NRR
K. Campe	NRR
F. Coffman	RES
T. Collins	NRR
N. Costanzi	RES
K. Coyne	NRR
D. Crutchfield	NRR
A. Drozd	NRR
F. Eltawila	RES
J. Han	RES
W. Hodges	RES
G. Hubbard	NRR
R. Jones	NRR
E. Jordan	AEOD
J. Kindrick	NRR
R. Landry	NRR
G. Lanik	AEOD
A. Levin	NRR
G. Marcus	NRR
C. McCracken	NRR
R. Meyer	RES
J. Murphy	RES
S. Nimil	NRR
H. Ornstein	AEOD
A. Ramey-Smith	RES
M. Razaque	NRR
J. Rosenthal	AEOD
E. Rossi	NRR
D. Scaletti	NRR
T. Speis	RES
J. Sun	RES
M. Taylor	OEDO
E. Throm	NRR
J. Tsao	NRR
M. Vagins	RES
J. H. Wilson	NRR
J. N. Wilson	NRR
N. Zuber	ACRS Consultant

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

M. Barron	NUS/LIS
R. Buchholz	GE
A. Carson	SERCH Licensing, Bechtel
T. Cook	DOE
E. Frech	R&R Ventures
J. Healzer	GE
T. Kenyon	Westinghouse
J. Leatherman	GE
B. McIntyre	Westinghouse
T. McIntyre	GE
J. Quinn	GE
L. Rib	AECLT
T. Schulz	Westinghouse
B. Shiralkar	GE
J. Trotler	Polestar

APPENDIX IV: FUTURE AGENDA

The Committee agreed to consider the following during the 418th ACRS Meeting, February 9-10, 1995:

Emergency Procedure Guidelines (EPGs) for BWR Core Power Stability/ATWS - The Committee will hear presentations by and hold discussions with representatives of the NRC staff, BWR Owners' Group, and General Electric Nuclear Energy (GENE) regarding the NRC staff Safety Evaluation Report on the modifications to the EPGs to address BWR core power stability/ATWS.

Environmental Qualification Requirements for Digital Instrumentation and Control (I&C) Systems - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the activities of the Office of Nuclear Regulatory Research related to environmental qualification requirements for digital I&C systems. Representatives of the industry will participate, as appropriate.

Analysis of Reactor Water Cleanup (RWCU) System Line-Break Accident for Operating BWRs - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the potential risks associated with an RWCU system pipe break outside of primary containment and the associated staff activities to evaluate this issue.

Proposed Final Revision to Regulatory Guide 1.118, "Periodic Testing of Electric Power and Protection Systems" - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed final revision 3 to Regulatory Guide 1.118, with emphasis on the differing views between the NRC staff and the industry. Representatives of the industry will participate, as appropriate.

Potential Research Issues for CANDU 3U - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding potential research issues for the CANDU3 design. Representatives of the Atomic Energy of Canada Limited Technologies will participate, as appropriate.

Operating Plants vs. Safety Goals - The Committee will discuss the Commission's request for further guidance and insight in determining where the current population of operating plants fall within the safety goals.

Amendment to 10 CFR 50.55a - The Committee will hear presentations by and hold discussions with representatives of the NRC staff and the Nuclear Energy Institute regarding the proposed final amendment to 10 CFR 50.55a to incorporate by reference the 1992 Edition with the 1992 Addenda of both Subsections IWE and IWL, Division 1, Section XI of the ASME Code that deal with inspection of concrete and metal containments.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA

DOCUMENTS

ITEM NO.

2. General Electric Nuclear Energy (GENE) Simplified Boiling Water Reactor (SBWR) Test and Analysis Programs
 1. GE SBWR Test and Analysis Program, Robert C. Jones, Reactor Systems Branch, NRR, dated January 12, 1995 [Viewgraphs]
 2. SBWR Design and Certification Program Review of Test & Analysis Program, dated January 12, 1995 [Viewgraph]
 3. SBWR Design and Certification Program. Program Status Update, R.H. Buchholz, GE Nuclear Energy, dated January 12, 1995 [Viewgraphs]
 4. SBWR System Response. Break Cases Considered, dated January 12, 1994 [Viewgraphs]
 5. GE Comments on Draft SER on SBWR Test & Analysis Program Description (TAPD), B.S. Shiralkar, GE Nuclear Energy, dated January 1995 [Viewgraphs]
 6. Integral System Response - Technical Basis, B.S. Shiralkar, GE Nuclear Energy, dated January 1995 [Viewgraphs]
 - 6a. Report of ACRS Consultants "V.J." Dhir, W. Wulf, N. Zuber - January 10, 1995, Thermal Hydraulics Phenomena Subcommittee Meeting [Handout #2.2]
3. Status of AP600 Design Certification Review
 7. Status of the AP600 Review by the NRC Staff, Thomas Kenyon, NRR, dated January 12, 1995 [Viewgraphs]
 8. AP600 Design Certification Program, Brian A. McIntyre, Westinghouse Electric Corporation, dated January 12, 1995 [Viewgraphs]
4. CANDU 3U Application Acceptance Review
 9. Reactor - Summary Description, 75-00552-SAR-004, Rev. 0, undated
 10. NRR Briefing on Status of the CANDU 3U Design Certification, Gail Marcus, NRR, dated January 12, 1995 [Viewgraphs]
5. Report of the Planning and Procedures Subcommittee
 11. Final Draft Minutes of the Planning and Procedures Subcommittee Meeting - January 11, 1995 [Handout #5.1]

6. Future ACRS Activities
 12. Future ACRS Activities [Handout #6.1]
7. Reconciliation of ACRS Comments and Recommendations
 13. Reconciliation of ACRS Comments and Recommendations [Handout #7.1]
10. Meeting with the Director of the NRC Office of Nuclear Regulatory Research (RES)
 14. Reflections on NRC Research Program, Eric Beckjord, Director, RES, dated January 13, 1995 [Viewgraphs]
11. Turbine Overspeed Study
 15. Turbine Overspeed Protection Systems, H.L. Ornstein, RAB/SPD, dated January 13, 1995 [Viewgraphs]
 16. Memorandum to Edward Jordan, Director, AEOD, from Brian Grimes, Director, Division of Project Support, NRR, dated November 28, 1995, regarding Office for Analysis and Evaluation of Operational Data (AEOD) Special Study AEOD/S94-02, "Turbine-Generator Overspeed Protection Systems at U.S. Light-Water Reactors"

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

2. General Electric Nuclear Energy (GENE) Simplified Boiling Water Reactor (SBWR) Test and Analysis Programs
 1. Table of Contents
 2. Project Status Report, dated January 12, 1995
 3. Memorandum to John Larkins, Executive Director, ACRS, from Dennis Crutchfield, Associate Director for Advanced Reactors and License Renewal, NRR, dated November 25, 1994, regarding Draft Safety Evaluation Report (SER) on the Adequacy of the Technical Approach to the Testing and Analysis Program (TAP) for the Simplified Boiling Water Reactor (SBWR) Design, with Attachment
 4. ACRS Report to Chairman Selin, dated June 10, 1992, regarding Testing and Analysis Programs in Support of the Simplified Boiling Water Reactor Design Certification
 5. Memorandum to Ivan Catton, Chairman, Thermal Hydraulic Phenomena Subcommittee, from Paul Boehnert, Senior Staff Engineer, ACRS, dated September 23, 1994, regarding ACRS Meeting Summary/Minutes of the Thermal Hydraulic Phenomena Subcommittee Meeting, August 24, 1994 - Rockville, Maryland, with Attachment [Official Use Only - Contains GE Nuclear Energy Proprietary Information]
 6. Letter to Patrick Marriott, Manager, GE Nuclear Energy, from Dennis Crutchfield, Associate Director for Advanced Reactors and License Renewal, NRR, dated March 7, 1994, regarding the Simplified Boiling Water Reactor Test Program, with Attachment
 7. Letter to Nicholas Liparulo, Westinghouse Electric Corporation, from Dennis Crutchfield, Associate Director for Advanced Reactors and License Renewal, NRR, dated March 7, 1994, regarding Effect of Testing Program on the Draft Safety Evaluation Report for the AP600

3. Status of AP600 Design Certification Review
 8. Table of Contents
 9. Tentative Agenda
 10. Status Report, dated January 12, 1995
 11. Letter to Nicholas Liparulo, Westinghouse Electric Corporation, from William Russell, Director, NRR, dated November 30, 1994, regarding Draft Safety Evaluation Report for the AP600 Design
 12. Letter to John Larkins, Executive Director, ACRS, from Nicholas Liparulo, Westinghouse Electric Corporation, dated June 16, 1994, regarding AP600 Design Certification Review Schedule

13. Memorandum to NRC Chairman and Commissioners from James Taylor, EDO, dated July 14, 1994, regarding Expedited Review Schedules for the Design Certification Applications, with Attachment
4. CANDU 3U Application Acceptance Review
 14. Table of Contents
 15. Tentative Agenda
 16. Status Report, dated January 12, 1995
 17. Letter to A.D. Hink, President, AECL Technologies, Inc., from Dennis Crutchfield, Associate Director for Advanced Reactors and License Renewal, NRR, dated December 15, 1994, regarding Results of the Acceptance Review for AECL Technologies' Application for Final Design Approval and Design Certification for the CANDU 3U Design, with Attachments
 18. SECY-94-079, Schedule and Resource Estimates for CANDU 3 Design Certification Review, dated March 24, 1994
10. Meeting with the Director of the NRC Office of Nuclear Regulatory Research (RES)
 19. Table of Contents
 20. Tentative Agenda
 21. Status Report, dated January 13, 1995
 22. Memorandum to Eric Beckjord, Director, RES, from John Larkins, Executive Director, ACRS, dated December 20, 1994, regarding ACRS Meeting with Director of RES on January 13, 1995, Rockville, Maryland
 23. Memorandum to the Commission from James Taylor, EDO, dated December 8, 1994, regarding Nuclear Safety Research Review Committee Report Dated December 5, 1994, with Attachment
11. Turbine Overspeed Study
 24. Table of Contents
 25. Agenda
 26. Project Status Report, dated January 13, 1994
 27. Memorandum to William Russell, Director, NRR, from Edward Jordan, Director, AEOD, dated September 10, 1994, regarding AEOD Special Study, "Turbine-Generator Overspeed Protection Systems at U.S. Light-Water Reactors," with Attachment
 28. ACRS Report to James Taylor, EDO, dated July 13, 1994, regarding Some Areas for Potential Staff Consideration for Operating Nuclear Power Plants and the Review of Future Plant Designs Resulting from the ACRS Review of the Evolutionary Light Water Reactors
 29. Memorandum to Thomas Kress, Chairman, ACRS, from James Taylor, EDO, dated September 9, 1994, regarding Some Areas for Potential Staff Consideration for Operating

- Nuclear Power Plants and the Review of Future Plant Designs Resulting from the ACRS Review of the Evolutionary Light Water Reactors, with Attachment
30. ACRS Report to James Taylor, EDO, dated March 11, 1992, regarding Proposed Priority Rankings of Generic Issues: Seventh Group, with Attachment
 31. Memorandum to David Ward, Chairman, ACRS, from James Taylor, EDO, dated April 24, 1992, regarding Proposed Priority Rankings of Generic Issues: Seventh Group, with Attachments
 32. Letter to Douglas Gipson, Senior Vice President, Detroit Edison Company, from Timothy Colburn, Project Manager, NRR, dated December 1, 1994, regarding Fermi 2 - Restart Action Plan Issues Closeout - Safety Evaluation Correction (TAC NOS. M89453, M89747, and M89863), with Attachment