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October 6-7, 1994

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CERTIFIED

Certified: November 3, 1994

MINUTES OF THE 414TH MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
OCTOBER 6-7, 1994
ROCKVILLE, MARYLAND

The 414th meeting of the Advisory Committee on Reactor Safeguards was held at Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on October 6-7, 1994. The purpose of this meeting was to discuss and take appropriate action on the items listed in the attached agenda. The meeting was open to public attendance. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Ann Riley & Associates, Ltd., 1612 K Street, N.W., Washington, D.C. 20006.]

ATTENDEES

ACRS Members: Dr. Thomas S. Kress (Chairman), Mr. William J. Lindblad (Vice-Chairman), Mr. James C. Carroll, Dr. Ivan Catton, Mr. Carlyle Michelson, Dr. Dana A. Powers, Dr. William J. Shack, and Mr. Charles J. Wylie. [For a list of other attendees, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Thomas S. Kress, Committee Chairman, opened the meeting at 8:30 a.m. and reviewed the schedule for the meeting.

Dr. Kress announced that the NRC is proposing a two-phased approach to the proposed contract with the National Academy of Sciences (NAS) to develop and carry out a workshop on the use of digital control and protection systems in nuclear power plants. The first phase will be to develop a report on digital instrumentation and control issues as related to their use in nuclear power plants. The second phase would involve a workshop to discuss these issues and suggest a regulatory approach to approving the use of digital systems.

II. NRC TEST PROGRAM IN SUPPORT OF THE AP600 AND SIMPLIFIED BOILING WATER REACTOR DESIGN CERTIFICATIONS (Open)

[Note: Mr. Paul Boehnert was the Designated Federal Official for this portion of the meeting.]

Dr. Ivan Catton, Chairman of the Thermal Hydraulic Phenomena (THP) Subcommittee, introduced this topic to the Committee. He noted that the THP Subcommittee has, for some time, been monitoring the NRC Office of Nuclear Regulatory Research (RES) test and analysis programs underway in support of the advanced light water reactor (ALWR) design certification programs. The Subcommittee has been concerned that there seems to be a lack of integration between the analysis programs (which uses the RELAP5 code) and the testing programs.

A THP Subcommittee meeting was held on August 25-26, 1994, to review the status of the ROSA-V/AP600 and the PUMA/Simplified Boiling Water Reactor (SBWR) test programs. At the conclusion of this meeting, the Subcommittee requested that RES discuss with the ACRS its plans to ensure sufficient integration of these test and analysis programs.

Dr. Catton noted that the briefing on the SBWR PUMA facility was the Subcommittee's first exposure to the details of this test program. He noted that while Dr. M. Ishii, Purdue University, is to oversee the PUMA test program, he has not been included on the team that is to analyze this data. Overall, Dr. Catton indicated that the PUMA test program shows promise of providing sound test data.

NRC Staff Presentation

Mr. M. Wayne Hodges, RES, provided introductory comments and noted that the primary purpose of the discussion is for RES to show how it will integrate the testing and analysis programs being conducted in support of the design certification reviews. Mr. Hodges noted the following points:

- The vendor is responsible for demonstrating to the NRC the safety of its design.
- The central product of ALWR thermal hydraulic (T/H) research is a system computer code assessed against data that can be used with a reasonable confidence to analyze the response of the AP600 and SBWR designs.
- Significant insights have been gained toward understanding AP600 plant transient response based on tests and analyses conducted by RES to date. Among these insights are: results

of RELAP5 analyses that showed that the initial 4th-stage automatic depressurization system valves were too small to allow adequate post-blowdown injection; tests in ROSA-V show both long-term thermal stratification in the cold-leg of the reactor coolant system (RCS), and the occurrence of condensation/oscillation events that influence RCS behavior.

Mr. Hodges indicated that the THP Subcommittee expressed concern with RES's handling of the so-called "top-down" scaling approach. Dr. Catton indicated that RES may have "holes" in the test data, given the lack of an adequate top-down scaling analysis. Mr. Hodges said that RES will conduct such a scaling analysis and if the results of this analysis show that additional testing is needed, it will be performed.

RES ALWR T/H Research Program

Mr. Norman Lauben, RES, provided details regarding the RES program to ensure integration of the ALWR test and analysis efforts. The following key points were noted:

- RES has formulated a comprehensive ALWR Research Plan that is based on the principles of the code scaling, applicability & uncertainty (CSAU) methodology (Figure 1). This approach reflects the fact that code development and assessment is an iterative process; thus, the code's capabilities and data assessment must be continually reviewed and updated.

During discussion, it was noted that the THP Subcommittee plans to meet with RES to discuss the AP600 PIRT in early December. As a result of questions posed by Dr. Powers on this matter, Dr. Catton invited Dr. Powers to attend this upcoming Subcommittee meeting.

- The schedules for development and assessment of the RELAP5 code for support of the AP600 and SBWR certification review were noted. RES will now use the same version of RELAP5 (MOD3.3 to be issued in 1996) for analyses of both advanced reactor designs. Dr. Catton indicated approval with this approach.
- As part of the approach to establish confidence in the code calculations, RES will perform a so-called "top-down" scaling analysis. RES plans to detail this scaling approach in a report due by the end of the year, and suggested a meeting with the THP Subcommittee during January to discuss same.

In response to Dr. Kress, RES has established a list of questions that serve as benchmarks to determine that the code is providing adequate results.

ALWR T/H RESEARCH PLAN

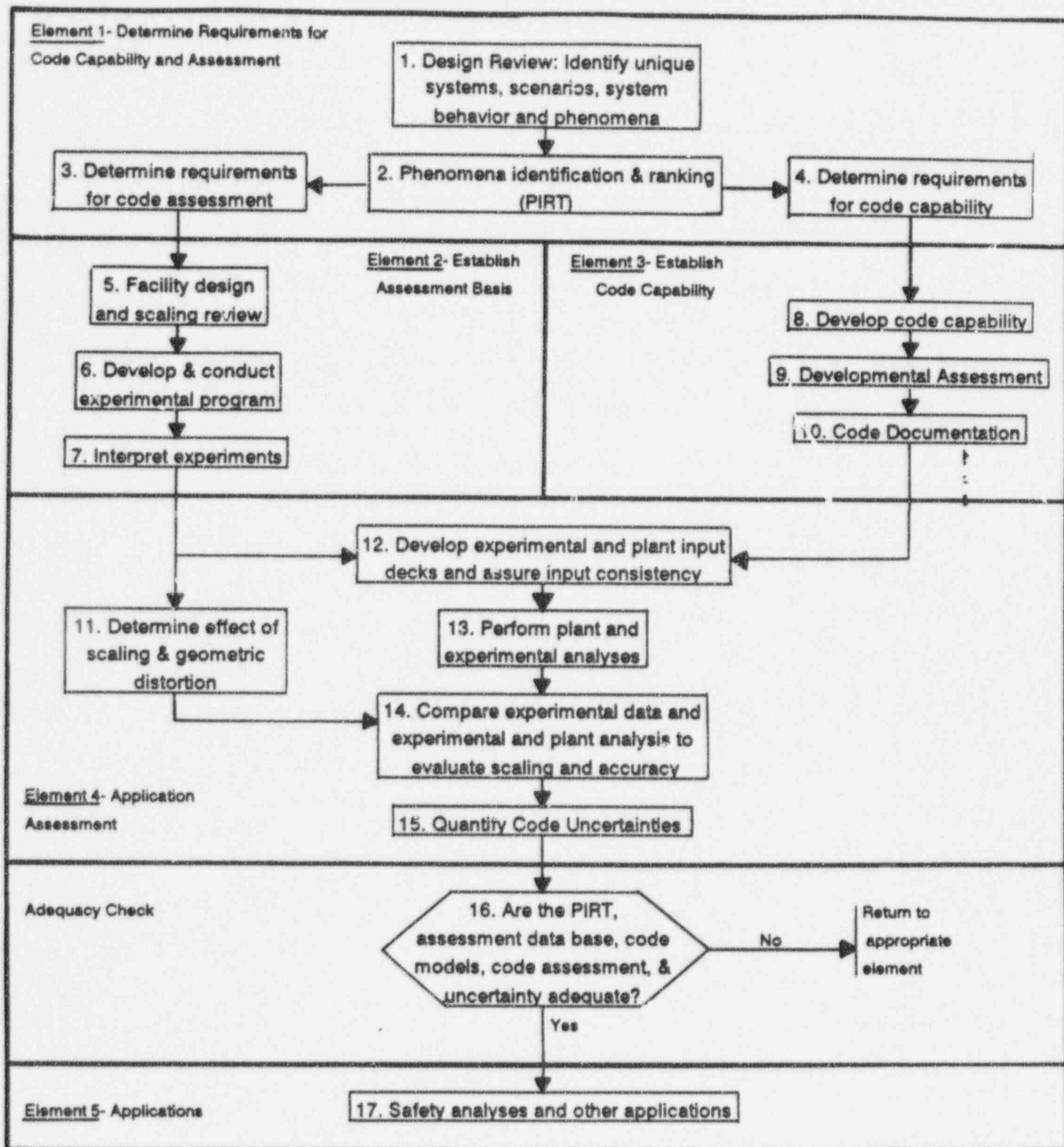
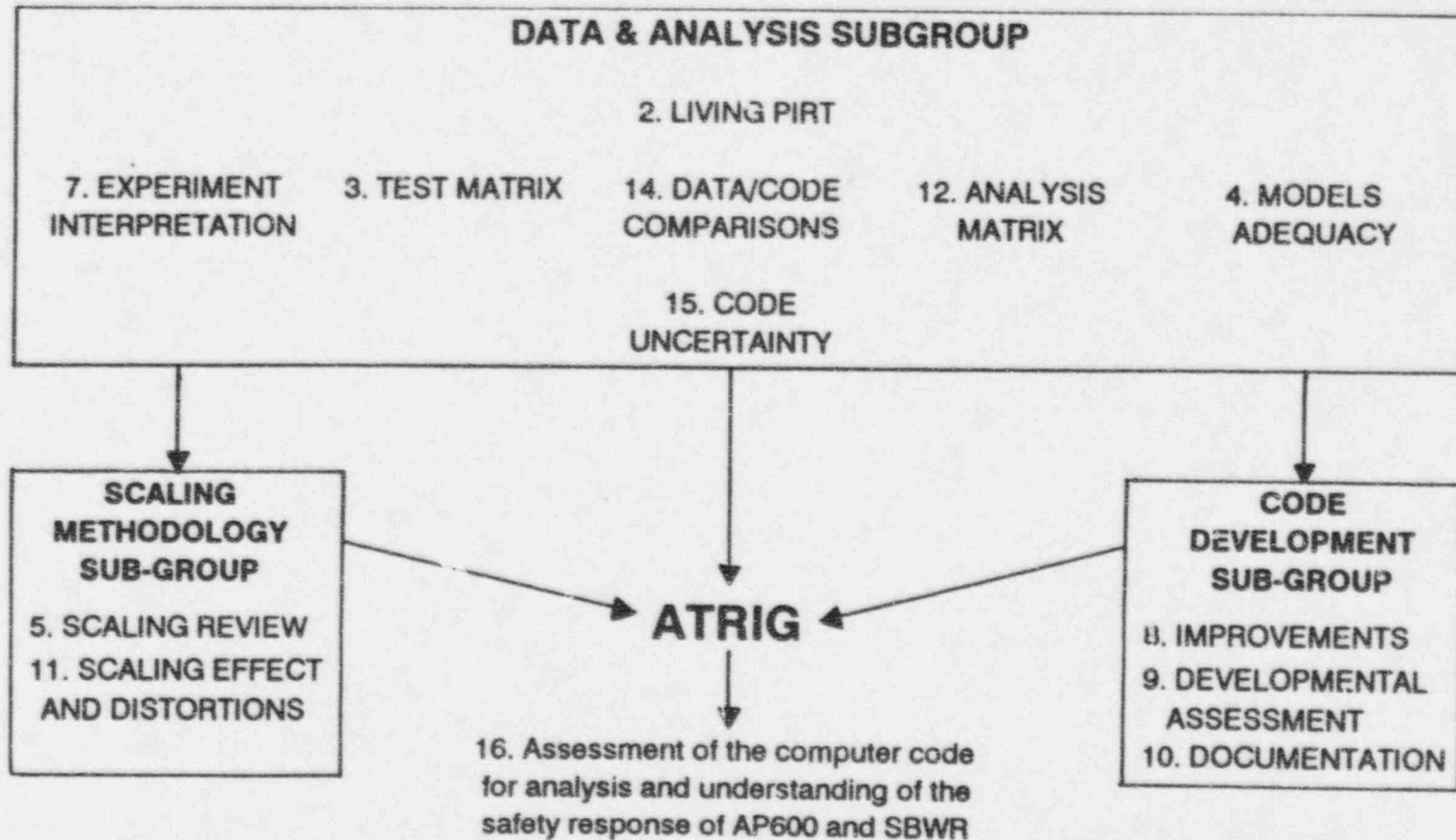


FIGURE 1

**FUNCTIONAL INTEGRATION OF ALWR
ANALYTICAL AND EXPERIMENTAL RESEARCH
THROUGH ALWR T/H RESEARCH INTEGRATION GROUP (ATRIG)**



Note: Numbers correspond to Research Plan steps

FIGURE 2

- To ensure close coordination and integration of the code development and assessment processes, RES has established a Group to be known as the "ATRIG" (ALWR T/H Research Integration Group - Figure 2). The ATRIG will be subdivided into three subgroups: Data and Analysis, Scaling Methodology, and Code Development. The respective tasks of these Subgroups are taken from the Research Plan noted in Figure 1.
- The following topics were discussed: code improvements to the MOD3 version of RELAP5, developmental assessment of same, rationale for testing programs, and the goal of counterpart testing and analyses. Mr. Lauben noted that the draft PIRT report for AP600 modeling will be available by November 1, 1994.

Dr. Catton opined to the effect that RES needs to ensure that the ATRIG has a "strong chair." Mr. Carroll asked if the advice provided to RES by the THP Subcommittee, about these test and analysis programs, has been useful. Mr. Lauben indicated that the Subcommittee has provided some insights that have been of use to RES. He indicated that, in particular, the practice of having the Subcommittee Consultants observe RES's staff meetings has been helpful.

In response to Mr. Carroll, Dr. Catton indicated that the SBWR test and analysis program appears to be on track. Since it is lagging the AP600 program by 6-12 months, it will see greater benefit of the ATRIG's oversight.

Follow-on Efforts

No specific follow-on/action items were identified during this meeting. As noted above, the THP Subcommittee will continue to monitor RES's activities in this area.

Conclusion

The Committee expects to continue its discussion on a draft report on this matter during its November ACRS meeting.

III. PROPOSED REVISION 2 TO REGULATORY GUIDE 1.82, "WATER SOURCES FOR LONG-TERM RECIRCULATION COOLING FOLLOWING A LOSS-OF-COOLANT ACCIDENT" (Open)

[Note: Mr. Douglas Coe was the Designated Federal Official for this portion of the meeting.]

Mr. James C. Carroll, Chairman of the Plant Operations Subcommittee, introduced this subject and noted that the Committee has been briefed by the NRC in January 1993, July 1993, and April 1994.

An NRC staff presentation, summarized below, was made by Mr. Al Serkiz, Engineering Issues Branch, RES, Mr. Richard Lobel, Containment Systems and Severe Accident Branch, Office of Nuclear Reactor Regulation (NRR), and Dr. Richard Barrett, Chief, Containment Systems and Severe Accident Branch, NRR.

The BWR strainer blockage events at the Barsebäck Nuclear Power Plant in Sweden on July 28, 1992 and at the Perry Nuclear Power Plant prompted a study, started in September 1993, of the potential for BWR emergency core cooling system (ECCS) strainer blockage in the U.S., and resulted in publication of draft NUREG/CR-6224 prepared by Science and Engineering Associates (SEA). This study estimated a 0.25 probability that core damage would occur due to loss of ECCS pump net positive suction head (NPSH) following a large LOCA. The staff also is proposing a revision to Regulatory Guide 1.82 regarding BWR ECCS suction strainer performance analysis and is continuing experimental work to refine the debris transport and NPSH head loss models.

The staff's early assessment was that the Barsebäck event was unique to Swedish plants. Then the Perry event in early 1993 revealed the U.S. BWR vulnerability, prompting issuance of NRC Bulletin 93-02 and its supplement 1 to notify licensees of this issue and required interim actions. The staff regards this as a compliance issue, and intends to issue a generic letter by August 1995, requiring specific licensee actions for resolution. The staff expects to seek ACRS and Committee to Review Generic Requirements (CRGR) approval to issue a proposed generic letter for public comment by March 1995.

The staff is continuing its consideration of the ACRS concern regarding the impact of debris to ECCS pump seals and bearings, and has requested that the BWR Owners Group (BWROG) evaluate this issue.

A BWROG presentation, summarized below, was made by Mr. Rocky Sgarro, Pennsylvania Power and Light, and Chairman of the BWROG ECCS Suction Strainer Committee.

All BWR licensees are members of the BWROG Committee for this issue and have prepared a generic safety assessment regarding this issue. The BWROG is looking at Leak-Before-Break criteria, but remains cognizant of the need to address the current regulatory design basis. They are meeting publically with the NRC staff each month to discuss BWROG test results, international working group perspectives, and other related matters.

During the discussion, Committee members raised questions regarding: why it took the staff until September 1993 to initiate the SEA study; the SEA report lack of treatment of the potential for damaged insulation and equipment beyond the immediate zone of destruction; the extent of individual plant differences that may bear on any generic solutions; the extent of actions by BWROG; and the relationship of possible solutions to performance-based regulations.

Conclusion

The Committee issued a report on this matter to Chairman Selin, dated October 14, 1994.

IV. REACTOR VESSEL STRUCTURAL INTEGRITY (Open)

[Note: Mr. Noel Dudley was the Designated Federal Official for this portion of the meeting.]

The Committee heard a presentation by Mr. Jack Strosnider, Jr., NRR, regarding a draft NUREG that summarizes licensee responses to Generic Letter (GL) 92-01, "Reactor Vessel Structural Integrity." He reviewed the objectives of the reactor pressure vessel evaluation program and future actions. Mr. Strosnider described reactor pressure vessel design, upper shelf energy, pressurized thermal shock reference temperature (RT_{pts}), and the reactor vessel integrity data base. From the data base, the projected RT_{pts} for two reactor vessels would exceed the screening criteria before the associated plant operating licenses expired.

The Committee discussed the following issues with the NRC staff: (1) how the upper shelf and RT_{pts} screening criteria were developed, (2) the ability to detect flaws of the size assumed in the derivation of the screening criteria, (3) the difference between reactor vessels that exceeded the RT_{pts} screening criteria and other reactor vessels, and (4) the important parameters that effect fracture toughness, such as neutron fluence.

Conclusion

This briefing was for information only. No Committee action was taken.

V. MEETING WITH THE DIRECTOR OF THE OFFICE FOR ANALYSIS AND EVALUATION OF OPERATIONAL DATA (Open)

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

Mr. Edward Jordan, Director of Office for Analysis and Evaluation of Operational Data (AEOD), stated that the AEOD missions are to:

- provide an independent capability to analyze operational data,
- review, analyze, and evaluate reactor plant and nuclear materials safety experience,
- manage the agency Incident Response and Emergency Response programs,
- manage the agency Technical Training program, and
- manage the agency CRGR function.

The AEOD resources are divided among the Incident Response Division, Safety Program Division, Technical Training Division, and Regional incident response oversight. The mission of the Safety Programs Division is to communicate the important safety lessons drawn from independent analysis and evaluation of experience. Some Safety Programs Division activities performed in 1993 are performance indicators, enhanced nuclear materials program, Rosemount transmitters study, turbine overspeed study, ESF bypass study, temporary repair issue, accident sequence precursor (ASP) results, and system reliability studies.

Mr. Patrick Baranowsky, AEOD, outlined the ASP process. The first step in the ASP approach is to review LERs to identify events that satisfy selection criteria as precursors. The next step is to determine the impact of "elements" of each event on systems and functions that provide protection from core damage. These systems and functions are defined through the use of event sequence models (event tree). A conditional probability of subsequent severe core damage is then estimated for each precursor using event trees that are modified to reflect the systems observed to be degraded or failed during the precursor. The final step is to rank precursors as to significance and identify the attributes of more significant events. Each event identified by screening as a candidate precursor was reviewed independently by two engineers against predefined criteria to determine if such event has possible risk significance and should be analyzed in greater detail.

The system reliability studies consist of:

- Use of operating experience to estimate reliability
- Analyze trends based on experience
- Compare experience with PRA and IPE inputs
- Quantify uncertainty in reliability estimates
- Identify plant specific outliers.

AEOD has initiated eleven system reliability studies and three overview reports.

The Safety Programs Division 1994 activities and future plans includes:

- Rulemaking
- Enhanced ASP
- Expanded Study Scope
- Testing study, and other new studies
- Lisbon initiative, Russia and Ukraine
- Increased communications

Dr. Frank Congel, AEOD, briefed the Committee regarding the Incident Response Division. He stated that the mission of the Incident Response Division is to provide direction and develop policies and guidelines to the NRC programs for the diagnostic evaluation of licensee performance, the investigation of operational incidents, and the immediate response to radiological incidents. Nine incident investigation teams (IIT) were conducted since 1985. Accident review group capability been added in 1993. Action resolution adequacy was initiated in 1994.

Dr. Kenneth Brockman, AEOD, summarized some current and future activities of the Emergency Response Branch. These activities include the following:

- Federal radiological emergency response plan
- International nuclear event scale
- LISBON initiative - Russian priority #3; Ukrainian priority #11
- Emergency response data system
- State outreach
- Plant information books/internet
- Operations center enhancements

Mr. Steven Arndt, AEOD, described the mission of the Technical Training Division. The mission includes the following:

- Coordination with NRC offices and regions in policy development and implementation of formal NRC staff qualification and

training programs and provide technical training to meet these integrated needs.

- Develop new courses and modify existing courses to meet new or changing needs identified by the NRC line organization.
- Manage the technical training center (TTC) facility.
- Provide technical assistance in areas of expertise and provide advice and limited technical training assistance to foreign regulatory counterparts.

Some TTC highlights during FY 1994 are:

- Implementation of Reactor Safety Course
- Acquisition of Shoreham simulator and program modification
- Acquisition of Trojan Simulator
- Classroom implementation of nuclear engineering workstation simulator
- Technical issue training Bulletin-2, "BWR Thermal-Hydraulic Stability"
- Continued improvement of existing curriculum and changes resulting from technical training needs survey (such as improvements to PRA training program)

Mr. Jordan added that the TTC courses are designed to fulfill the requirements in the qualification standards for the technical staff. The PRA courses are relatively well developed, however digital instrumentation and control (I&C) courses need additional development. Mr. Jordan stated that he welcomes any suggestions from the ACRS regarding the current TTC curriculum and offered to brief the Committee (in the future) regarding this matter.

The AEOD databases that are accessible to the ACRS are:

- Accident sequence precursor database
- Common cause failure database
- Sequence coding and search system
- Performance indicator database
- Monthly operating report database
- Nuclear plant reliability data system
- Preliminary notifications, emergency notifications, morning reports
- Plant data books

Following the conclusion of AEOD presentation, the Committee members and staff toured the NRC Operations Center located on the 4th floor of the Two White Flint North Building.

Conclusion

This briefing was for information only. The Committee agreed to schedule a meeting with TTC staff to discuss curriculum.

VI. ROD CONTROL SYSTEM SINGLE FAILURE POTENTIAL (Open)

[Note: Mr. Douglas Coe was the Designated Federal Official for this portion of the meeting.]

Mr. Carroll, Chairman of the Plant Operations Subcommittee, introduced this subject and the participants.

An NRC staff presentation was made by Mr. Robert Jones, Chief, Reactor Systems Branch, NRR, Mr. Hukam Garg, I&C Branch, NRR, and Ms. Margaret Chatterton, Reactor Systems Branch, NRR.

The NRC staff discussed the series of rod control evolutions and malfunction events from May 25, 1993 to June 3, 1993, at Salem Nuclear Power Plant Unit 2 that prompted an NRC augmented inspection team visit on June 5, and activation of the Westinghouse Regulatory Response Group. The malfunctions included an uncontrolled withdrawal of one control rod. A WCAP report was summarized regarding the rod control system failure assessment and corrective action program to ensure that for applicable Westinghouse reactors any similar future failure will cause control rods to always insert (i.e., not withdraw) by changing the timing order of the current inputs to the rod drive mechanisms. This WCAP also recommends a new post-modification surveillance test program. The staff discussed their assessment of this WCAP report, including the applicable regulatory requirements, their conclusion that the issue had low safety significance, and NRC generic communications issued. The staff also discussed their review of a second WCAP report regarding the assessment of asymmetric rod control cluster assembly withdrawal.

During the discussion, the Committee questioned the presenters on the quality of circuit board manufacturing, the interaction between the non-safety control rod system with the safety-related protection systems, and the previous failure modes and effects analysis (FMEA) that did not analyze component failures within circuit cards. Mr. Roger Newton, Westinghouse Regulatory Response Group, was present to answer questions.

Conclusion

The Committee took no action based on this information briefing.

VII. INDIVIDUAL PLANT EXAMINATION (IPE) INSIGHTS PROGRAM (Open)

[Note: Mr. M. Dean Houston was the Designated Federal Official for this portion of the meeting.]

Dr. Thomas Kress, Acting Chairman of the Individual Plant Examinations (IPE) and Probabilistic Risk Assessment Subcommittees, noted that a joint subcommittee meeting had been held on September 27, 1994. He briefly summarized the topics that were discussed: (1) IPE Insights Program, (2) Diesel Generator Problems at H. B. Robinson Nuclear Power Plant, (3) PRA Implementation Plan, and (4) Resolution of Generic Issue by the IPE/IPEEE (IPE External Events) Process. As a follow-up to the subcommittee meeting, he stated that the NRC staff was requested to brief the Committee on the IPE Insights Program.

NRC Staff Presentation

Ms. Mary Drouin, RES, discussed the IPE/IPEEE Programs. Within the Office of Research, there are two programs: (1) Review Program - an individual technical review of each submittal, and (2) Insights Program - an examination of results across classes of plants. The Insights Program included two phases with tasks listed as follows: IPE Insights - overall examination of results [core damage frequency (CDF) and containment performance], plant improvements, and model categorization; and IPEEE Insights. To date, 50 IPE submittals have been examined to establish a correlation between CDF and reactor/containment type, dominant accident sequence, and dominant contributor within each accident sequence. Some general preliminary conclusions were stated as follows:

- CDF for BWRs lower than CDF for PWRs
- Lots of variability among plants
- No single feature appears important to each class of accident.

Other tasks are to be initiated soon. Meaningful insights are expected by the end of CY 1995.

Conclusion

This briefing was for information only. Consistent with the Committee's decision, Dr. Larkins was asked to inform Mr. Murphy that the Committee decided to review the subject proposed rulemaking package during the public comment period. The Committee has no objection to the NRC staff recommendation to issue the proposed rule for public comment.

The Committee requested that two consultant's reports on this matter be provided to the staff for their consideration.

VIII. EXECUTIVE SESSION (Open)

[Note: Dr. John Larkins was the Designated Federal Official for this portion of the meeting.]

A. Report and Memorandum

Potential for BWR ECCS Strainer Blockage Due to LOCA Generated Debris (Report to Chairman Selin from T.S. Kress, ACRS Chairman, dated October 14, 1994)

Proposed Revision to 10 CFR 2.802, Petition for Rulemaking, Addressing Regulatory Improvements (Memorandum to Joseph A. Murphy, Acting Director, Division of Safety Issue Resolution, Office of Nuclear Regulatory Research, from John T. Larkins, Executive Director, dated October 14, 1994)

B. Report on the Meeting of the Planning and Procedures Subcommittee Held on October 5, 1994

The allocated time scheduled for this session was used to discuss proposed ACRS reports. Dr. Kress recommended that the members read the final draft minutes of the Planning and Procedures Subcommittee meeting that was held on October 5, 1994.

C. Reconciliation of ACRS Comments and Recommendations

The allocated time scheduled for this session was used to discuss proposed ACRS reports. Discussion of responses from the NRC Executive Director for Operations to recent ACRS reports will be rescheduled during the November ACRS meeting.

D. Selection of New ACRS Members

The allocated time scheduled for this session was used to discuss proposed ACRS reports. Discussion of qualifications of candidates nominated for appointment to the ACRS will be rescheduled during the November ACRS meeting.

E. Strategic Planning

The allocated time scheduled for this session was used to discuss proposed ACRS reports. Discussion of issues that are important the Committee and the Commission will be rescheduled during the November ACRS meeting.

F. Members Travel

Dr. Catton informed the Committee that he will not attend the Third International Conference on Containment Design and Operation on October 19-22, 1994, in Toronto, Canada, as planned.

The Committee approved funding support for Dr. Seale to attend the Workshop on the Safety of Russian Reactors (prior to the American Nuclear Society Winter Meeting), November 12-13, 1994, in Washington D.C.

G. Future ACRS Activities

Dr. Catton, Chairman of the Thermal Hydraulic Phenomena Subcommittee cancelled the tentatively scheduled meeting for October 28, 1994, so that the BWR core power stability/ATWS briefing could be rescheduled during the November full Committee meeting.

H. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 415th ACRS Meeting, November 3-5, 1994, and future Subcommittee meetings.

The 414th ACRS meeting was adjourned at 5:30 p.m. on Friday, October 7, 1994.

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special accommodations should contact Tom Hall by October 13 at the address indicated below.

An official record of the meeting will be available for public inspection through the Tom Hall, Division of Consumer Affairs, Room N-3647, 200 Constitution Avenue, NW., Washington, DC 20210, telephone 202-219-8615.

For additional information contact: Joanne Goodell, Directorate of Policy, Occupational Safety and Health Administration, Room N-3641, 200 Constitution Avenue, NW., Washington, DC 20210, telephone 202-219-8021.

Signed at Washington, D.C. this 19th day of September, 1994.

Joseph A. Dear,

Assistant Secretary of Labor.

[FR Doc. 94-23723 Filed 9-23-94; 8:45 am]

BILLING CODE 4510-25-01

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (94-0731)]

NASA Advisory Council; Task Force on Shuttle-Mir Rendezvous and Docking Missions; Meeting.

AGENCY: National Aeronautics and Space Administration.

CTION: Notice of meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Public Law 92-463, as amended, the National Aeronautics and Space Administration announces a meeting of the NASA Advisory Council, Task Force on Shuttle-Mir Rendezvous and Docking Missions.

DATES: October 11, 1994, 3 p.m. to 5:30 p.m. and October 12, 1994, 7:30 a.m. to 6:00 p.m.

ADDRESSES: The National Aeronautics and Space Administration, Lyndon B. Johnson Space Center, Building 1, Room 945, Houston, TX 77058.

FOR FURTHER INFORMATION CONTACT: Mr. William L. Vantine, Code M, National Aeronautics and Space Administration, Washington, DC 20546, 202/358-1698.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

Review the upcoming Shuttle-Mir missions from the following perspectives: training, operations, rendezvous and docking.

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: September 20, 1994.

Timothy M. Sullivan,

Advisory Committee Management Officer.

[FR Doc. 94-23734 Filed 9-23-94; 8:45 am]

BILLING CODE 7510-01-02

NASA FAR Supplement (NFS); Availability in Electronic Form

AGENCY: Office of Procurement, Procurement Policy Division, National Aeronautics and Space Administration (NASA).

ACTION: Notice.

SUMMARY: The National Aeronautics and Space Administration is announcing the availability of the NASA FAR Supplement in electronic form in order to satisfy requests for an electronic copy of the publication. With appropriate software, users will be able to search the copy using keywords.

ADDRESSES: Requests for NFS Version 89.16 should be sent by electronic mail addressed to: dbeck@proc.hq.nasa.gov

FOR FURTHER INFORMATION CONTACT:

Dave Beck (202) 358-0482.

SUPPLEMENTARY INFORMATION: Under 42 U.S.C. 2473(c)(1), notice is given that NFS Version 89.16, effective September 30, 1994, is available in WordPerfect 5.1. For as long as we can accommodate requests, a copy will be sent by e-mail, without charge, to anyone sending an e-mail request. We are supplying the NFS by e-mail until we place the NFS on Internet.

The electronic copy contains the text that is used to produce the loose-leaf version of the NFS. The NFS is also published in 48 CFR Chapter 18. Efforts are made to minimize the differences between the loose-leaf version and 48 CFR Chapter 18. However, neither the electronic copy nor the loose-leaf version are a substitute for the Code of Federal Regulations or the Federal Register.

The copy supplied by e-mail will be a compressed file of 964 kilobytes along with shareware (pkunzip.exe, 30 kilobytes) for decompressing the file. When the file is "unzipped" it becomes 122 WordPerfect 5.1 files (plus a README file) totalling 3 megabytes. Persons using WordPerfect 5.1, or software capable of converting from WordPerfect 5.1, should be able to search the text of the 122 files using keywords.

(Caution: When converted to ASCII, some text and most tables are difficult to read. We

will try to improve later versions of the NFS to eliminate this problem.)

Tom Luedtke,

Deputy Associate Administrator for Procurement.

[FR Doc. 94-23772 Filed 9-23-94; 8:45 am]

BILLING CODE 7510-01-02

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting Agenda

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on October 6-8, 1994, in Conference Room T2B3, 11545 Rockville Pike, Rockville, Maryland. The dates for this meeting were published in the Federal Register on Friday, August 22, 1994.

Thursday, October 6, 1994

8:30 A.M.-8:45 A.M.: Opening Remarks by the ACRS Chairman (Open)

The ACRS Chairman will make opening remarks regarding conduct of the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 A.M.-10:45 A.M.: NRC Test Programs in Support of the AP600 and SBWR Design Certification (Open)

The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the management and status of the NRC test programs being conducted at the ROSA-V and PUMA test facilities. Representatives of the industry will participate, as appropriate.

11:00 A.M.-12:30 P.M.: Proposed Revision 2 to Regulatory Guide 1.82, Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident (Open)

The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed Revision 2 to Regulatory Guide 1.82. Representatives of the industry will participate, as appropriate.

1:30 P.M.-2:30 P.M.: Reactor Vessel Structural Integrity (Open)

The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding materials data acquisition associated with reactor vessel structural

integrity. Representatives of the industry will participate, as appropriate.

2:30 P.M.-5:00 P.M.: Meeting With the Director, Office for Analysis and Evaluation of Operational Data (AEOD) (Open)

The Committee will meet with the Director of AEOD to discuss items of mutual interest, including the NRC Technical Training Program.

Friday, October 7, 1994

8:30 A.M.-8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)

The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 A.M.-9:45 A.M.: Rod Control System Single Failure Potential (Open)

The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the rod control system single failure event at Salem Unit 2, the findings of the Augmented Inspection Team (AIT), licensee response to Generic Letter 93-04, and the staff's actions. Representatives of the industry will participate, as appropriate.

10:00 A.M.-11:30 A.M.: IPE Insights Program (Open)

The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the IPE Insights Program.

11:30 A.M.-12:15 P.M.: Report of the PeP Subcommittee (Open/Closed)

The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business and internal organizational and personnel matters relating to the ACRS staff members.

A portion of this session may be closed to discuss matters that relate solely to internal personnel rules and practices of this Advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

1:15 P.M.-1:45 P.M.: Future ACRS Activities (Open)

The Committee will discuss topics proposed for consideration during future ACRS meetings.

1:45 P.M.-2:00 P.M.: Reconciliation of ACRS Comments and Recommendations (Open)

The Committee will discuss responses from the NRC Executive Director for Operations to ACRS comments and recommendations included in recent ACRS reports.

2:00 P.M.-3:00 P.M.: Selection of New ACRS Members (Open/Closed)

The Committee will discuss qualifications of candidates nominated for appointment to the ACRS.

A portion of this session will be closed to discuss matters the release of which would constitute a clearly unwarranted invasion of personal privacy.

3:15 P.M.-4:15 P.M.: Strategic Planning (Open)

The Committee will hold strategic planning discussions related to its future activities.

4:15 P.M.-6:30 P.M.: Preparation of ACRS Reports (Open)

The Committee will discuss proposed ACRS reports on matters considered during this meeting.

Saturday, October 8, 1994

8:30 A.M.-11:00 A.M.: Preparation of ACRS Reports (Open)

The Committee will continue its discussion of proposed ACRS reports on matters considered during this meeting.

11:15 A.M.-11:45 A.M.: New Research Needs (Open)

The Committee will discuss new research needs, if any, identified during this meeting.

11:45 A.M.-12:00 Noon: Miscellaneous (Open)

The Committee will discuss miscellaneous matters related to the conduct of Committee activities and complete discussions of topics that were not completed during previous meetings as time and availability of information permit.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Register on September 30, 1993 (58 FR 51118). In accordance with these procedures, oral or written statements may be presented by members of the public, electronic recordings will be permitted only during the open portions of the meeting, and questions may be asked only by members of the Committee, its consultants, and staff. Persons desiring to make oral statements should notify the ACRS Executive Director, Dr. John T. Larkins, at least five days before the meeting if possible, so that appropriate arrangements can be made to allow the necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding

the time to be set aside for this purpose may be obtained by contacting the ACRS Executive Director prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the ACRS Executive Director if such rescheduling would result in major inconvenience.

I have determined in accordance with Subsection 10(d) P.L. 92-463 that it is necessary to close portions of this meeting noted above to discuss information that involves the internal personnel rules and practices of this advisory Committee per 5 U.S.C. 552(c)(2); and to discuss information the release of which would constitute a clearly unwarranted invasion of personal privacy per 5 U.S.C. 552b(c)(6).

Further information regarding topics to be discussed, whether the meeting has been canceled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the ACRS Executive Director, Dr. John T. Larkins (telephone 301-415-7361), between 7:30 A.M. and 4:15 P.M. EST.

Dated: September 20, 1994.

Andrew L. Bates,
Advisory Committee Management Officer.
[FR Doc. 94-23755 Filed 9-23-94; 8:45 am]
BILLING CODE 7890-01-01

[Docket No. 50-100]

Georgia Institute of Technology; Consideration of Application for Renewal of Facility License

The United States Nuclear Regulatory Commission (the Commission) is considering renewal of Facility License No. R-97, issued to the Georgia Institute of Technology (Georgia Tech or the licensee) for operation of the Georgia Tech Research Reactor located on the Georgia Tech campus in the city of Atlanta, Fulton County, Georgia.

The renewal would extend the expiration date of Facility License No. R-97 for twenty years from date of issuance, in accordance with the licensee's timely application for renewal dated April 19, 1994.

Prior to a decision to renew the license, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

Within thirty days of publication of this notice, the licensee may file a request for a hearing with respect to renewal of the subject facility license

APPENDIX II



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

September 26, 1994

SCHEDULE AND OUTLINE FOR DISCUSSION
414TH ACRS MEETING
OCTOBER 6-8, 1994

Thursday, October 6, 1994, Conference Room 283, Two White Flint North,
Rockville, Maryland

1) 8:30 - 8:45 A.M.

Opening Remarks by the ACRS Chairman (Open)
1.1) Opening Statement (TSK/JTL)
1.2) Items of Current Interest (TSK/JTL)
1.3) Priorities for Preparation of ACRS Reports (TSK/RPS)

30

2) 8:45 - 10:45 A.M.

NRC Test Programs in Support of the AP600 and SBWR Design Certification (Open)
(IC/PAB)
2.1) Remarks by the Subcommittee Chairman
2.2) Briefing by and discussions with representatives of the NRC staff regarding the management and status of the NRC test programs being conducted at the ROSA-V and PUMA test facilities in support of the certification of the Westinghouse AP600 and General Electric Nuclear Energy (GENE) Simplified Boiling Water Reactor (SBWR) designs

30 10:50

10:45 - ~~11:00~~ A.M.

BREAK

10:50 50

3) ~~11:00~~ - 12:30 P.M.

Proposed Revision 2 to Regulatory Guide 1.82, Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident (Open) (JCC/DHC)

3.1) Remarks by the Subcommittee Chairman
3.2) Briefing by and discussions with representatives of the NRC staff regarding the proposed Revision 2 to Regulatory Guide 1.82

Representatives of the industry will participate, as appropriate.

50 40

12:30 - 1:30 P.M.

LUNCH

[= Transcribed portion of meeting

4) 1:30⁴⁰ - 2:30⁵ P.M.

Reactor Vessel Structural Integrity (Open)
(WJS/NFD)

- 4.1) Remarks by the Subcommittee Chairman
4.2) Briefing by and discussions with representatives of the NRC staff regarding a draft NUREG that compiles licensee responses to Generic Letter 92-01, Reactor Vessel Structural Integrity

Representatives of the industry will participate, as appropriate.

5) 2:30⁵ - 5:00²⁵ P.M.
(3:30-3:45 Break)

Meeting with the Director of the Office for Analysis and Evaluation of Operational Data (AEOD) (Open) (TSK/MME)

- 5.1) Remarks by the ACRS Chairman
5.2) Discussion with Mr. Jordan, AEOD Director, on items of mutual interest, including the following
- Overview of AEOD activities in 1993
 - AEOD activities in 1994 and changes from 1993
 - Future plans for AEOD activities
 - Existing and proposed NRC Technical Training programs
 - Accident Sequence Precursor Program (ASP)
 - How the ASP screens events for further detailed analysis, and the role of risk significance in the screening process
 - How ASP analyzes events such as the April 7, 1994 event at Salem Unit 1 that involved significant human error
 - AEOD data bases being maintained and those accessible to the ACRS
 - Status of rulemaking for collection of reliability information from licensees in support of implementation of the Maintenance Rule
 - Status of the Emergency Response Data System (ERDS)

6) 5:00²⁵ - 7:00⁰⁰ P.M.
~~6:00~~

Tour of NRC Operations Center (TWFN 4th Floor)

Friday, October 7, 1994, Conference Room 2B3, Two White Flint North, Rockville, Maryland

7) 8:30 - 8:35 A.M.

Opening Remarks by the ACRS Chairman (Open)
(TSK/JTL)

8) 8:35 - 9:⁴⁰~~45~~ A.M.

Rod Control System Single Failure Potential
(Open) (JCC/DHC)

- 8.1) Remarks by the Subcommittee Chairman
8.2) Briefing by and discussions with representatives of the NRC staff regarding the rod control system single failure event at Salem Unit 2, the findings of the Augmented Inspection Team (AIT), licensees' responses to Generic Letter 93-04, and associated staff actions

Representatives of the industry will participate, as appropriate.

⁴⁰
9:45 - 10:00 A.M.

BREAK

9) 10:00 - 11:¹⁵~~30~~ A.M.

IPE Insights Program (Open) (TSK/MDH)

- 9.1) Remarks by the Subcommittee Chairman
9.2) Briefing by and discussions with representatives of the NRC staff regarding the IPE Insights Program

~~10) 11:30 - 12:15 P.M.~~
Cancelled

Report of the Planning and Procedures Subcommittee (Open/Closed) (TSK/JTL)

Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to ACRS staff members.

(Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.)

12:15 - 1:15 P.M.

LUNCH

11) ~~1:15~~ 3:30 - ~~1:45~~ ^{4:25} P.M.

Future ACRS Activities (Open) (TSK/JTL)

Discussion of the recommendations of the Planning and Procedures Subcommittee regard-

- ing items proposed for consideration by the full Committee during future meetings
- 12) ~~1:45 - 3:00 P.M.~~ ^{Postponed}
- Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et al./RPS., et al.)
Discussion of responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports
- 13) ~~2:00 - 3:00 P.M.~~ ^{Postponed}
- Selection of New ACRS Members (Open/Closed)
- 13.1) Remarks by the ACRS Chairman
13.2) Discussion of qualifications of candidates nominated for appointment to the ACRS
- (Note: A portion of this session may be closed to discuss matters the release of which would constitute a clearly unwarranted invasion of personal privacy.)
- 3:00 - 3:15 P.M. BREAK
- 14) ~~3:15 - 4:15 P.M.~~ ^{Postponed}
- Strategic Planning (Open) (TSK/JTL)
Discussion of issues that are of significant importance to the Committee and the Commission.
- 15) ~~4:15 - 6:30 P.M.~~
11:15 - 12:15 P.M.
1:15 - 3:30 P.M.
4:25 - 5:30 P.M.
- Preparation of ACRS Reports (Open)
Discussion of proposed ACRS report on:
- 15.1) NRC Test Programs in Support of AP600 and SBWR Design Certification (IC/PAB)
15.2) Proposed Revision 2 to Regulatory Guide 1.82 (JCC/DHC)
15.3) IPE Insights Program (TSK/MDH) (tentative)
15.4) NRC Technical Training Program (JCC/DHC) (tentative)

Cancelled Saturday Session

Saturday, October 8, 1994, Conference Room 2B3, Two White Flint North, Rockville, Maryland

- 16) 8:30 - 11:00 A.M. Preparation of ACRS Reports (Open)
Continue discussion of the proposed ACRS reports listed under Item 15
- ~~11:00 - 11:15 A.M. BREAK~~

- 17) 11:15 - 11:45 A.M. New Research Needs (Open) (TSK/RPS)
Discussion of new research needs, if any,
identified during this meeting
- 18) 11:45 - 12:00 Noon Miscellaneous (Open) (TSK/JTL)
Complete discussion of matters considered
during this meeting and matters considered
but not completed during previous meetings
as time and availability of information
permit

- NOTE:
- Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.
 - Number of copies of the presentation materials to be provided to the ACRS - 35.

APPENDIX III: MEETING ATTENDEES

414TH ACRS MEETING
OCTOBER 6-7, 1994

NRC STAFF

R. Architzel	NRR
S. Arndt	AEOD
P. Baranowsky	AEOD
R. Barrett	NRR
D. Bessette	RES
B. Borchardt	NRR
A. Chaffee	NRR
M. Chatterton	NRR
C. Chung	RES
T. Collins	NRR
F. Congel	AEOD
Dahlia	NRR
M. Drouin	RES
D. Ebert	RES
F. Eltawila	RES
H. Garg	NRR
J. Han	RES
D. Hickman	AEOD
W. Hodges	RES
R. Jones	NRR
E. Jordan	AEOD
T. King	RES
N. Lauben	RES
T. Lee	RES
R. Lobel	NRR
S. Malik	RES
M. Malloy	NRR
M. Marto	RES
S. Mays	AEOD
D. McPherson	NRR
R. Meyer	RES
F. Odar	RES
K. Olive	OC
H. Pastis	NRR
M. Payne	NRR
G. Rhee	RES
C. Robinson	INFOSEC
D. Robinson	PMB.
M. Rubin	RES
H. Scott	RES
A. Serkiz	RES
L. Shotkin	RES
D. Solberg	RES
J. Strosnider	NRR
M. Taylor	OEDO
C. Troutman	RES

Appendix III
414th ACRS Meeting

M. Wegner
J. Wilson

AEOD
NRR

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

B. Alpha	Transco Products, Inc.
H. Barbeito	Bechtel
M. Beaumont	Westinghouse
A. Carson	Becheal
M. Cheek	NUS
J. Lake	INEL
S. Modro	INEL
R. Newton	Westinghouse
D. Palmrose	INEL
L. Rib	AECLT
C. Rothberg	INEL
F. Sciacca	SEA
R. Sgarro	PP&L
S. Sloan	INEL
E. Wolbert	Transco Products, Inc.
G. Wu	NEI
F. Zitria	NUS

APPENDIX IV: FUTURE AGENDA

The Committee agreed to consider the following during the 415th ACRS Meeting, November 3-5, 1994:

BWR Core Power Stability/ATWS - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the Final Safety Evaluation Report (FSER) on Emergency Procedure Guidelines to address BWR core power instabilities. Representatives of the industry will participate, as appropriate.

Watts Bar Progress Status - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the status of resolution of issues associated with Watts Bar. Representatives of the industry will participate, as appropriate.

Reconciliation of ACRS Comments and Recommendations - The Committee will discuss responses from the NRC Executive Director for Operations to ACRS comments and recommendations included in recent ACRS reports.

Options with Regard to Revising 10 CFR Part 100, Reactor Site Criteria - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed rulemaking on reactor site criteria.

Selection of New ACRS Members (Closed) - The Committee will discuss qualifications of candidates nominated for appointment to the ACRS.

Strategic Planning - The Committee will hold strategic planning discussions related to its future activities.

APPENDIX V
LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA

DOCUMENTS

ITEM NO.

- 1 Opening Remarks by the ACRS Chairman
 1. Letter to Neil Thompson from Chairman Selin, dated August 22, 1994, regarding National Performance Review, with attachment
 2. Preliminary Agenda of the 22nd Water Reactor Safety Information Meeting, October 24-26, 1994

- 2 NRC Test Programs in Support of the AP600 and SBWR Design Certification
 3. NRC ALWR Thermal-Hydraulic Research Program Overview, M. Wayne Hodges, Division of Systems Research, dated October 6, 1994 [Viewgraphs]
 4. NRC ALWR (AP600) Thermal-Hydraulic Research Program, G. Norman Lauben, Division of Systems Research, dated October 6, 1994 [Viewgraphs]
 - 4a. Note to Paul Boehmert from Norman Lauben, undated, regarding Additional Slides

- 3 Proposed Revision 2 to Regulatory Guide 1.82, Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident
 5. Proposed Revision 2 to RG 1.82 "Water Sources for Long-Term Recirculation Cooling Following a LOCA" and NUREG/CR-6224, A. W. Serkiz, RES/DSIR/EIB, dated October 6, 1994 [Viewgraphs]
 6. Regulatory Resolution of BWR Strainer Blockage Issue, Richard Lobel, NRR, dated October 6, 1994 [Viewgraphs]
 7. Status of BWROG Work on ECCS Suction Strainer Performance Issues, R.R. Sgarro, PP&L, dated October 6, 1994 [Viewgraphs]
 - 7a. The Barseback Incident - A Precursor Challenging Fundamental Safety Principles of LWRs, Preprint of paper submitted to PSAM 94, March 20-25, 1994, Lennart Carlsson et al

- 4 Reactor Vessel Structural Integrity
 8. Reactor Pressure Vessel Status Report, Jack R. Strosnider, NRR, dated October 6, 1994 [Viewgraphs]

- 5 Meeting with the Director of the Office for Analysis and Evaluation of Operational Data (AEOD)
 9. NRC Office for Analysis and Evaluation of Operational Data, undated [Viewgraphs]
 10. Memorandum to William T. Russell, Director, NRR, from Edward Jordan, Director, AEOD, dated September 30, 1994, regarding AEOD Special Study, "Turbine-Generator Overspeed Protection Systems at U.S. Light-Water Reactors," with attachment
- 6 Tour of NRC Operations Center
 11. Operations Center - Technology Overview, Prepared by HFS, Inc., undated [Viewgraphs]
 12. NRC Operations Center - Initial Activation Protective Measures Team, undated [Organizational Chart]
- 8 Rod Control System Single Failure Potential
 13. Generic Letter 93-04 Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies, Margaret S. Chatterton and Hukam C. Garg, NRR, dated October 7, 1994 [Viewgraphs]
- 9 IPE Insights Program
 14. IPE/IPEEE Programs, Mary Drouin, RES, dated October 7, 1994 [Viewgraphs]
- 10 Report of the Planning and Procedures Subcommittee
 15. Final Draft Minutes of Planning and Procedures Subcommittee Meeting on October 5, 1994 [Handout #10.1]
- 11 Future ACRS Activities
 16. Future ACRS Activities [Handout #11.1]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

- 3 Proposed Revision 2 to Regulatory Guide 1.82, Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident
1. Table of Contents
 2. Tentative Agenda
 3. Status Report
 4. Memorandum to Gary M. Holahan, Director, Division of Systems Safety and Analysis, NRR, from Joseph A. Murphy, Acting Director, Division of Safety Issue Resolution, RES, dated August 26, 1994, regarding Review of Draft Regulatory Guide DG-1038, Proposed Revision 2 to Regulatory Guide 1.82, "Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident", with attachment
 5. NUREG/CR-6224, Parametric Study of the Potential for BWR ECCS Strainer Blockage Due to LOCA Generated Debris, Draft Report for Comment, dated August 4, 1994, Science and Engineering Associates, Inc.
- 4 Reactor Vessel Structural Integrity
6. Table of Contents
 7. Tentative Agenda
 8. Status Report
 9. Selected pages of draft NUREG, Revision 6, dated September 20, 1994, regarding Reactor Pressure Vessel Status Report
 10. Generic Letter 92-01, Revision 1, dated March 6, 1992, regarding Reactor Vessel Structural Integrity, 10 CFR 50.54(f) [NUDOCS #9203060147]
- 5 Meeting with the Director of the Office for Analysis and Evaluation of Operational Data (AEOD)
11. Table of Contents
 12. Tentative Agenda
 13. Status Report
 14. Memorandum to E. Jordan, Director, AEOD, from J. Larkins, Executive Director, ACRS, dated September 14, 1994, regarding ACRS Meeting with Director of AEOD on October 6, 1994, Rockville, Maryland
- 8 Rod Control System Single Failure Potential
15. Table of Contents
 16. Agendas
 17. Project Status Report
 18. Generic Letter 93-04, dated June 21, 1993, regarding Rod

- Control System Failure and Withdrawal of Rod Control Cluster Assemblies, 10 CFR 50.54(f) [NUDOCS #9306170326]
19. Letter to Steven E. Miltenberger, Vice President, PSEG, from Marvin W. Hodges, Director, Division of Reactor Safety, dated August 11, 1993, regarding NRC Augmented Inspection Team (AIT) Report Nos. 50-272/93-81 and 50-311/93-81, with attachment

9 IPE Insights Program

20. Table of Contents
21. Tentative Agenda
22. Status Report
23. ACRS Report to Chairman Zech, dated June 9, 1987, regarding Proposed Generic Letter on Individual Plant Examinations for Severe Accident Vulnerabilities
24. ACRS Report to Chairman Zech, dated May, 10, 1988, regarding Proposed Generic Letter on Individual Plant Examinations and the Proposed Integrated Safety Assessment Program II
25. Staff Requirements Memorandum to James M. Taylor, EDO, dated May 24, 1993, regarding SECY-93-118 - Status of the Individual Plant Examination of External Events (IPEEE)
26. SECY-94-134, Status of IPE and IPEEE Insights Program, dated May 20, 1994
27. RES Viewgraphs Used During the ACRS Joint Subcommittee Meeting on Individual Plant Examinations and Probabilistic Risk Assessment, September 27, 1994