GERS-2943

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DESIGNATED ORIGINAL

rtified By EMB



APPENDICES

I. Federal Register Notice
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IV. Future Agenda and Subcommittee Activities
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Issued: October 5, 1994

MINUTES OF THE 412TH MEETING OF THE ADVISORY COMMITTEE ON REACTOR SAFEGUARDS AUGUST 4-5, 1994 ROCKVILLE, MARYLAND

The 412th meeting of the Advisory Committee on Reactor Safeguards was held at NRC Commissioners' Hearing Room, One White Flint North Building, Rockville, Maryland, on August 4-5, 1994. The purpose of this meeting was to discuss and take appropriate action on the items listed in the attached agenda. The meeting was open to public attendance, except portions that dealt with classified information or matters of a personal nature. There were no written statements nor requests for time to make oral statements from members of the public regarding the meeting.

A transcript of selected portions of the meeting was kept and is available in the NRC Public Document Room at the Gelman Building, 2120 L Street, N.W., Washington, D.C. [Copies of the transcript are available for purchase from Ann Riley & Associates, Ltd., 1612 K Street, N.W., Washington, D.C. 20006.]

ATTENDEES

ACRS Members: Dr. Thomas S. Kress (Chairman), Mr. William Lindblad (Vice-Chairman), Mr. James C. Carroll, Dr. Ivan Catton, Mr. Peter R. Davis, Mr. Carlyle Michelson, Dr. Dana A. Powers, Dr. William J. Shack, and Mr. Charles J. Wylie. [For a list of other attendes, see Appendix III.]

I. CHAIRMAN'S REPORT (Open)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

Dr. Thomas S. Kress, Committee Chairman, opened the meeting at 8:30 a.m. and reviswed the schedule for the meeting.

Dr. Kress reported that the General Electric Nuclear Energy application for certification of the Advanced Boiling Water Reactor design has been approved by Mr. William Russell. The ASEA Brown Boveri - Combustion Engineering application for certification of the System 80+ Standard Plant design was also approved.

Mr. Carroll stated that, subsequent to the ACRS final report, Mr. Bill Borchardt, Office of Nuclear Reactor Regulation (NRR), sent a letter to the Committee identifying five additional items of potential significance regarding the System 80+ Standard Plant design. Mr. Carroll noted that he conclude, along with Dr. Kress, that none of these items were of substance and there was no need for further Committee review.

II. STEAM GENERATOR TUBE INTEGRITY MATTERS (Open)

[Note: Mr. Noel Dudley was the Designated Federal Official for this portion of the meeting.]

Dr. William Shack, Chairman of the Materials and Metallurgy Subcommittee, summarized the discussions held on August 3, 1994, with representatives of the NRC staff and the Nuclear Energy Institute (NEI) staff, as well as the author of a differing professional opinion. The discussions concerned a proposed Generic Letter on an interim tube plugging criteria and a draft of an Advance Notice of Proposed Rulemaking (ANPR) on Steam Generator Tube Integrity.

Dr. Shack noted that the value for core damage frequency used in the differing professional opinion had been updated by the NRC staff using different assumptions. The updated values indicated a lower core damage frequency, which appeared reasonable.

Dr. Shack noted that the present radiological release analysis provides reasonable assurance that 10 CFR Part 100 limits would not be exceeded. He indicated that the basis for the present iodine spiking estimates does not appear to be well developed, and that the NRC staff plans to perform additional work on quantifying the spiking.

Dr. Shack commented that the industry supports the proposed Generic Letter based on the expected economic savings and lower personnel exposures.

In response to a discussion initiated by Mr. Carroll, Dr. Brian Sheron, NRR, stated that the mitigating affects of leak before break assumptions on the possible tube-support plate displacement during a steam line break would be examined further and reported to the Committee.

Mr. Carroll questioned the acceptability of a through-wall crack in a primary coolant boundary. Mr. Lindblad compared the primary coolant boundary integrity requirements for heavy-wall and short-wall structures. The ASME code allows heavy-wall boundaries to contain cracks or linear indications that are not expected to leak between inspections. On the other hand, the proposed Generic Letter allows leakage from through-wall cracks in short-walled structures. Dr. Shack observed that allowing leaks is not good engineering practice. However, he believed that the staff made a fairly convincing case that the risk associated with through-wall cracks in tubes at the core-support-plate intersection is small.

Dr. Catton believed that a preliminary analysis of the impact of a steam line break on steam generator tubes could determine whether a problem existed and whether a research program was needed. Mr.

Timothy Reed, NRR, stated that such an analysis existed and that he would provide the report to the Committee.

Dr. Kress noted that reliance on a correlation requires a good database. However, he believed that the database supporting the empirical voltage correlations is weak. Mr. Carroll noted that the proposed Generic Letter requires a tube pulling program, which would continually expand the database. Dr. Shack also added that the program would ensure the correlations remained valid as the tubes aged.

In regard to the staff's presentation on the ANPR, Mr. Carroll stated that he was discouraged that the staff does not fully understand what constitutes performance-based regulation. Mr. Davis and Dr. Catton agreed that the ANPR is very prescriptive though the staff considers the approach to be performance-based.

Conclusion

The Committee completed a report on this matter during the meeting. Subsequently, it was brought to the attention of the Committee that the staff has provided incorrect information regarding the off-site dose calculations. The staff has submitted revised calculations to the ACRS for consideration. Consequently, the ACRS Chairman, in consultation with the Subcommittee Chairman, decided to defer issuance of the report until after the Committee had discussed the revised calculations during the September ACRS meeting.

III. DOD/DOE NAVAL REACTORS DESIGN (Closed)

[Note: Mr. Paul Boehnert was the Designated Federal Official for this portion of the meeting. Minutes for this portion of the meeting are classified and maintained separately.]

The Committee heard presentations by and held discussions with representatives of the NRC staff and Department of Defense (DOD)/Department of Energy (DOE) regarding the latest nuclear plant propulsion design being installed in submarines of the Seawolf class. A meeting of the Naval Reactors Subcommittee was held to review this matter on July 20, 1994. In addition, members of the Committee toured the SEAWOLF construction site at the General Dynamics Shipyard in Groton, Connecticut, and the Bettis Atomic Power Laboratory in conjunction with this review on July 19 and August 2, 1994, respectively.

Conclusion

The Committee issued a classified report to Chairman Selin dated August 10, 1994, on this matter.

IV. REACTOR TRIP EVENT AT SALEM NUCLEAR POWER PLANT, UNIT 1 (Open)

[Note: Mr. Dean Houston was the Designated Federal Official for this portion of the meeting.]

Mr. James C. Carroll, Chairman of the Plant Operations Subcommittee, indicated that on April 7, 1994, due to operator error and equipment complications, Salem Nuclear Power Plant, Unit 1, had experienced a reactor trip and multiple automatic safety injections. He stated that an Augmented Inspection Team (AIT) had been formed to study this event and that the AIT report would be the basis for the Committee discussion.

NRC Staff Presentation

Mr. Edward Goodwin, NRR, introduced the subject and the AIT Team Leader, Mr. Robert Summers, Region I. Mr. Goodwin indicated that what started as a simple transient was turned into a challenging event through some work-arounds and human errors.

Mr. Summers, AIT Team Leader, discussed the activities of the AIT in regard to the Salem Unit 1 reactor scram with subsequent safety injection and solid reactor coolant system. He described the configuration of the plant, especially the once-through circulating water system that was vulnerable to river grass clogging, and the sequence of events on April 7, 1994. He discussed the most probable root causes for the overall event, reactor trip, safety injections and unexpected component response to the first safety injection.

Mr. Summers summarized the findings of the AIT as follows:

- No abnormal releases of radiation to the environment occurred during the event.
- The event challenged the reactor coolant system pressure boundary resulting in multiple, successful operations of the pressurizer power operated relief valves and no operations of the pressurizer safety valves.
- Operator errors occurred that complicated the event.
- Management allowed equipment problems to exist that made operations difficult for plant operators.
- Some equipment was degraded by the event, but overall, the plant performed as designed.

- Operator use of emergency procedures was good. However, procedural inadequacies were noted with other operating procedures.
- Licensee's investigations and troubleshooting efforts were good.

Mr. Summers concluded with a discussion of generic issues and generic communications that were identified as a result of this event. One of these was an Information Notice 94-55: Problems With Copes-Vulcan Pressurizer Power-Operated Relief Valves dated August 4, 1994. He agreed to provide copies of this report to the Committee and did so on August 5, 1994.

Mr. Carroll indicated that an equipment problem, vulnerability of the high steam flow instruments, was well known and documented in other notices. He asked if the licensee had a good operating events assessment program. Mr. Rick Villar, PSE&G, stated that operating experience was given much attention by management and that operating events were discussed weekly. [Note: Mr. Villar called on August 8, 1994 to correct part of his statement to the Committee. He had erroneously indicated that LERs were part of this weekly review. He determined later that they were not.]

Mr. Carroll also expressed a concern that the shift supervisor reassigned a reactor operator to another task at a critical time during the event. Mr. Villar indicated that PSE&G was also concerned about this maneuver and that extensive retraining has been performed.

Conclusion

This briefing was for information only. No Committee action was taken on this matter.

V. MEETING WITH THE DIRECTOR OF THE NRC OFFICE OF NUCLEAR REACTOR REGULATION (Open)

[Note: Dr. Medhat El-Zeftawy was the Designated Federal Official for this portion of the meeting.]

Mr. William Russell, Director, NRR, briefed the Committee regarding the following items:

1. Senior Management Meeting Review (SMMR) Process

Mr. Russell stated that the staff requirements memorandum (SRM) of February 14, 1994, requested the NRC staff to describe the process and the criteria used in evaluating plants when identifying those to be placed on the problem

plant list and the role of NRC's various inspections and reviews in this process.

The NRC Executive Director for Operations (EDO), the Regional Administrators, and the headquarters program Office Directors conducted the first periodic senior management meeting (SMM) in April 1986. Since then, SMMs have been held every 6 months. At the January 1987 SMM, management expanded the scope of issues discussed at the meeting to include significant issues at major fuel facilities and material licensees. Also, in June 1991 management broadened the SMM discussions to determine those plants to recognize as good performers.

Each semiannual SMM process involves a comprehensive review of the licensed operating power reactors to identify discussion plants and potential good performers for review at the SMM. The process starts with a screening meeting that the Director of NRR schedules and holds with each Regional Administrator approximately 2 months before each SMM. Each reactor's performance is reviewed, including inspection findings, operating experience, performance indicators (PIs), enforcement history, and corporate and plant management activities, with emphasis on the effectiveness of licensee self-assessment and corrective actions. Except for the PIs data, this information is also used by the regions in implementing the reactor inspection and SALP programs.

The SMM review of discussion plants is led by the regional administrator with input from other senior managers. The managers determine which plants, if any, to place in the following categories:

Category 1: Plants removed from the problem plant list

Category 2: Plants authorized to operate and that NRC will monitor closely

Category 3: Shutdown plants requiring NRC authorization to start up and that the NRC will monitor closely

Plants that are placed in either a Category 2 or 3 status are referred to as being on the NRC's problem plant list.

Not all plants discussed at the SMM are placed on the problem plant list. Typically, a plant may be discussed at more than one SMM before being placed on the list. Exceptions are plants whose performance has dropped quickly or who have experienced an event that reveals basic problems.

As a result of SMM discussions, the EDO issues letters to the problem plants, also may issue trending letters, letters to

recognize good performers, or direct that a diagnostic evaluation be conducted at a plant for which additional information is necessary.

The staff briefs the Commission on the SMM results at a public meeting approximately two weeks after each SMM. In addition, the SMM results are communicated to the NRC's Congressional Oversight Committees.

Mr. Russell stated that significant improvement in operating power reactor performance has been observed since 1985 with a reduction in the average number of reactor trips, safety systems actuations, personnel exposure and others. However, even with this overall improvement, the NRC continues to identify individual plants with marginal performance and significant operational problems.

The NRC's SMM review process has been an effective means by which plants with actual and potential performance problems are identified. Only four plants have returned to the problem plant list after being removed.

Customized Inspection Planning Process (CIPP)

Mr. Russell summarized the CIPP as follows:

- Review of prior inspection results and licensee performance through inspection reports, enforcement actions, SALP, SMM, LERS, PIs, and licensee self assessments.
- Attempts to apply risk to analyze performance (IPE).
- Development of performance assessment/inspection planning tree.
- Multi-disciplined assessment team site visit.
- Final analysis and development of inspection recommendations for the upcoming inspection period.

The benefits of the CIPP are as follows:

- To provide integrated assessment of NRC information on licensee performance. This will be confirmed with onsite inspection, resulting in inspection plan with rationale for future NRC activities.
- Optimize the use of NRC resources by focusing inspection and evaluation activities on sites and areas needing the most attention.

Matches resources with performance.

Mr. Russell cited the Point Beach site visit by a team of NRC personnel from headquarters and Region III during the week of April 18, 1994, as part of a trial program intended to improve the overall NRC inspection process. The purpose of this effort was to integrate the results of the recent inspection efforts at Point Beach, to provide the licensee with the current NRC view of their performance, and to recommend future inspection activities. The team identified, for example, the performance of the control room to be weak, noted instances of poor communications, a lack of familiarity, and limited oversight of plant operations by senior reactor operators.

3. Construction Inspection Program (CIP) Under 10 CFR Part 52

The sequence of the CIP is construction and preoperational testing, fuel loading authorization, startup testing, and then operation.

Under construction and preoperational testing, inspection activities have to be provided first. These inspection activities include inspection, testing, analysis, and acceptance criteria (ITAAC) verification, and program inspections.

The ITAAC verification includes the following items:

- site preparation
- structure fabrication
- system fabrication
- system and component functional testing

The program inspections include the following items:

- licensed operator training
- site security inspections
- radiological controls program inspections
- quality control program inspections
- other inspections

The majority of the ITAAC items will be met at approximately 80 percent construction completion.

Mr. Frank Gillespie, NRR, outlined the ITAAC verification process and cited the high pressure core flooder (HPCF) system ITAAC for the GENE/ABWR design. This ITAAC requires the HPCF system flow to be achieved within 16 seconds of receipt of a simulated initiation signal. Mainly, the ITAAC consists of three elements: the design commitment; inspections, tests, analyses; and acceptance criteria.

The NRC staff's inspection activity during construction is to determine, prior to fuel loading, whether the acceptance criteria associated with the ITAACs have been met. The staff is considering several concepts to verify satisfactory completion of ITAACs such as the bridge concept, sign-as-you-go (SAYGO), independent NRC inspection, and licensee determination that all ITAACs have been met.

Mr. Gillespie stated that by the time many ITAACs would be ready for verification, much construction activity will have taken place, most of which will have been inspectable only as it was performed. Therefore, the NRC will need to conduct inspections throughout plant construction, and the results of these inspections will provide the bases for final ITAAC verification.

Mr. Russell concluded that there still is much work to be performed to satisfy the design ITAAC with the programmatic ITAAC.

4. Organizational Factors Research Program

Mr. Russell indicated that the organizational factors research program is continuing only in the area of a PRA model that can quantify the effects of organizational factor on risk to the public. The model should offer insights on what aspects of organizational factors are most important to risk at a given plant. Currently no specific schedule for completion of this program has been identified.

5. Lessons Learned from the NRC Staff Review of the Evolutionary Designs

Mr. Russell stated that the NRC staff is drafting a response to the ACRS letter of July 13, 1994, addressing potential areas for the evolutionary designs. After the Committee has had an opportunity to review this response, Mr. Russell stated that he is willing to return for further discussion.

Conclusion

During the discussion on the customized inspection planning process, Mr. Russell indicated that NRR is developing a database

for the IPE program. He said that a briefing could be arranged as part of the follow-up on what NRR is doing to implement the use of PRA within the NRC. Mr. Peter Davis, ACRS member, stated that a joint meeting of the ACRS Subcommittees on Individual Plant Examinations and Probabilistic Risk Assessment is scheduled to be held on September 27, 1994, and it would appear that an NRR staff discussion regarding this issue would be in order.

In response to a question from Mr. Michelson, Mr. Russell stated that the NRC staff is drafting a response to the ACRS letter dated July 13, 1994, on Some Areas for Potential Staff Consideration for Operating Nuclear Power Plants and the Review of Future Plant Designs Resulting from the ACRS Review of the Evolutionary Light Water Reactors. Mr. Russell agreed to meet with the Committee to discuss this response if the Committee wishes.

This briefing was for information only. No Committee action was taken.

VI. REPORT ON THE MEETING OF THE PLANNING AND PROCEDURES SUBCOM-MITTEE HELD ON AUGUST 3, 1994 (Open/Closed)

[Note: Dr. John T. Larkins was the Designated Federal Official for this portion of the meeting.]

The Committee heard a report from Dr. Kress on the Planning and Procedures Subcommittee meeting held on August 3, 1994. The following items were discussed:

Status of Member Nomination - Dr. Kress reported that about 190 applications have been received as of July 29, 1994. Additional applications received through August 31st (the last day for submitting applications) will be summarized and provided to the Committee for discussion during the September meeting.

Dr. Kress requested that the members comment on the draft selection criteria that can be used in screening applicants and as guidance for the rating panel in nominating new members. The criteria may be discussed during the next full committee meeting.

Status of the ACRS Fellows Program - Dr. Kress announced that Mr. Richard Sherry has accepted an offer to be an ACRS Fellow. He is expected to start in August. Four additional candidates for the ACRS Fellows Program were interviewed during the August meeting and one will be interviewed during the September meeting. The status of the ACRS Fellows Program Brochure was also discussed.

The ACRS staff agreed to revise the ACRS Fellows Brochure to achieve consistency with the Charter of the Fellows Program Subcommittee.

- ACRS/ACNW Office Reorganization Dr. Kress announced that the reorganization of the ACRS/ACNW Office became effective on July 17, 1994. The reorganization eliminated two supervisory positions, the Assistant Executive Director for Operations and Chief of the Technical Information Branch. This reorganization was initiated in response to the National Performance Review's direction to streamline government and reduce overhead, and also the Executive Director's effort to make the staff more unified.
- Use of Consultants Dr. Kress advised the Committee that, in response to a memorandum from Dr. Larkins, dated June 7, 1994, regarding Mr. Schrock terminating his consulting agreement with GE, Mr. Rothschild stated that OGC has no objection to his participating in matters relating to the ACRS review of the Westinghouse design. However, Mr. Schrock should wait one year from the date of termination of his contract with GE before participating in the review of GE SBWR design. Since the review of the GE SBWR design is likely to be delayed, the Committee believes that the one year wait will not be an issue.
- and Procedures Subcommittee discussed the resolution proposed by the ACRS staff management to individual questions and concerns provided by the members regarding each section of the ACRS Members Handbook and provided guidance for clarification and improvement in several areas.

The ACRS staff plans to revise the ACRS Members Handbook consistent with the recommendations of the Subcommittee and Executive Director. The revised version will be provided to the Committee members for final approval.

VII. EXECUTIVE SESSION (Open/Closed)

[Note: Dr. John Larkins was the Designated Federal Official for this portion of the meeting.]

A. Report and Memorandum

Report on S6W Nuclear Propulsion Plant for Application on a Seawolf-Class Submarine (Classified Report to Chairman Selin from T. S. Kress, ACRS Chairman, dated August 10, 1994)

Proposed Revision 3 to Regulatory Guide 1.118, "Periodic Testing of Electric Power and Protection Systems," (Draft Regulatory Guide DG-1028) (Memorandum for Lawrence C. Shao, Director, Division of Engineering, Office of Nuclear Regulatory Research, from John T. Larkins, Executive Director, dated August 11, 1994) - Consistent with the Committee's decision, Dr. Larkins informed Mr. Shao that the Committee decided to review the proposed final version of the Regulatory Guide 1.118, Revision 3, after reconciliation of public comments.

Proposed Rulemaking Package on Generic Safety Issue 23, "Reactor Coolant Pump Seal Failure" (Memorandum to Joseph A. Murphy, Acting Director, Division of Safety Issue Resolution, Office of Nuclear Regulatory Research, from John T. Larkins, Executive Director, dated August 11, 1994) - Consistent with the Committee's decision, Dr. Larkins informed Mr. Murphy that the Committee decided not to review the subject proposed rulemaking package until the staff has reconciled the public comments. The Committee has no objection to the NRC staff proposal to issue the proposed rule for public comment.

B. Foreign Travel

The Committee approved funding support for Dr. Catton to attend and participate in the Third International Conference on Containment Design and Operation on October 18-21, 1994, in Toronto, Canada. The Committee amended its funding approval of travel expenses for Dr. Davis to attend the same Conference on Containment Design and Operation.

C. Selection of New ACRS Members (Closed)

The Committee discussed the qualifications of candidates nominated for appointment to the ACRS. The Committee reviewed the current list of applicants and identified other potential candidates not on the list. Among these names, the Committee picked six highly qualified applicants and potential applicants so that arrangements could be made for interviewing these candidates during a future meeting.

D. Future ACRS Activities

- The Committee proposed the following items for discussion during the meeting with the NRC Commissioners scheduled on Thursday, September 8, 1994:
 - Status of the ACRS Review of the Evolutionary and Passive Plant Designs
 - Lessons Learned from the ACRS Review of the Evolutionary Plant Designs

- Protective Action Guidelines

- Thermo-Lag Fire Barriers

 NAS-NRC Study and Workshop on Digital Instrumentation and Control Systems

- Selection of New ACRS Members

The ACRS staff was asked to determine what items are of interest to the Commissioners. It is planned to have this meeting in the new ACRS Conference Room.

The Committee recommended that representatives of the Nuclear Energy Institute, formerly NUMARC, be invited to brief the Committee on its reorganization and activities. The Committee is also interested in being briefed by the Director, AEOD, on items of mutual interest, including those AEOD programs that the ACRS could have access to.

E. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 413th ACRS Meeting, September 8-10, 1994, and future Subcommittee meetings.

The 412th ACRS meeting was adjourned at 3:15 p.m. on Friday, August 5, 1994.

sources of information used to determine U.S. monetary, fiscal, trade, and commercial policies. They are also used to deflate the Gross National Product.

Revision

Employment and Training
Administration
Unemployment Compensation for
Former Federal Employees, Handbook
No. 391

1205-0179 One-time

Individuals or households; State or local governments; Federal agencies or employees

Form No.		Average time per response	
S 931 S 931A S 935 S 933 S 934 S 936 S 939	144,000 33,120 144,000 2,360 15,025 7,200 75	5 minutes. 3 minutes. 3 minutes. 3 minutes. 1 hour 45 minutes.	

Federal law (5 U.S.C. 8501-8509) provides unemployment insurance protection to former (or partially unemployed) Federal civilian employees. It is referred to, in abbreviated form, as "UCFE." The forms

contained throughout the UCFE Handbook are used in connection with the provision of this benefit assistance.

Extension

Bureau of Labor Statistics

Labor Market Information (LMI) Cooperative Agreement (CA) 1220-0079; BLS LMI 1A, 1B, 2A, 2B State or local governments

Form No.	Respond- ents	Frequency	Average time per response
LMI 1A, 18 LMI 2A LMI 2B 771 total hours.	55 10 27		6 hrs., 42 mins. 8 hrs., 2 mins. 45 mins.

Cooperative Agreements between BLS and SESAs are entered annually in order to provide the LMI statistics. The agreements provide the basis for administrative planning, financial planning, and monitoring.

Extension

Employment Standards Administration
Establishment Information
1215-0008: WH-45 and WH-45 (Sp.)
On occasion
Businesses or other for profit; Non-profit institutions; Small businesses or organizations
5,000 respondents; 10 minutes per

5,000 respondents; 10 minutes per response; 835 total hours; 2 forms Form WH-45, when completed by employers, is used to determine coverage of various establishments that are being considered for investigation under the FLSA.

Extension

Bureau of Labor Statistics
1220-0045; BLS 9300
State and local governments; Farms,
Businesses or other for profit; Small
business or organizations
280,000 respondents; 59 minutes per
response; 275,000 total hours.
The Annual Survey of Occupational

Injuries and Illnesses is the primary indicator of the Nation's progress in providing every working man and woman safe and healthful working conditions. Survey data are used to evaluate the effectiveness of Federal and State programs and to prioritize scarce resources. (This information collection request (ICR) includes "OSHA Records: Prenotification of Exempt Employer" recordkeeping requirements which was previously included in the ICR approved under OMB Number 1220—0029.)

Extension

Airline Employee Protection Program
(29 CFR Part 220)
1205-0322
On occasion; semi-annually
Businesses or other for-profit
1,240 respondents; 3 average hours per
response; 372 total hours.

Office of the American Workplace

The Airline Deregulation Act (ADA), section 43(d) requires the Secretary of Labor to establish a program to implement the first-right-of-hire provision of the ADA. This program, as outlined in 29 CFR 220 includes two specific information collection requirements: (1) an airline job vacancy list; and (2) a semi-annual report of designated employees hired. Both reports are required by the ADA.

Extension

Mine Safety and Health Administration

Record of Examinations for Hazardous Conditions 1219–0083

Each shift

Businesses and other for profit; Small businesses or organizations 2,170 respondents; 508.5 average hours

per response; 1,103,445 total hours.

Requires operators of surface coal mines and surface facilities to keep records of required examinations result.

records of required examinations results for hazardous conditions. Records consist of the nature and location of any hazardous condition found and the actions taken to abate the hazardous condition.

Signed at Washington, DC this 8th day of July, 1994.

Kenneth A. Mille.

Departmental Clearance Officer. [FR Doc. 94-17975 Filed 7-22-94; 8:45 am] 68.1898 CODE 4616-24-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards; Meeting Agenda

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on August 4-5, 1994, in Conference Room T2B3, 11545 Rockville Pike, Rockville, Maryland.

Thursday, August 4, 1964

8:30 A.M.-8:45 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 A.M.-10:15 A.M.: Steen: Gomerator Tube Integrity Matters (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding a proposed Generic Letter on Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes, and an Advance Nestice of Proposed Rulemaking regarding steem generators surverisence and maintenance. Representatives of the industry will participate, as appropriate.

10:30 A.M.-1:08 P.M.: DOD/DOE Neval Reactors Design (Closed)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and Department of Defense (DOD)/ Department of Energy (DOE) regarding the latest nuclear plant propulsion design being installed in automarines of the Seawolf class.

This portion of the meeting will be closed to discuss classified information applicable to this matter.

2:00 P.M.—3:00 P.M.: Reactor Trip Event at Salem Unit 1 (Open)—The Committee will hear presentations by end hold discussions with representatives of the NRC staff regarding the April 7, 1994 reactor trip event at the Salem Nuclear Plant, Unit 1 and the findings and recommendations of the Augmented Inspection Team (AIT) which investigated this event.

3:00 P.M.—5:15 P.M.: Selection of New ACRS Members (Open/Closed)—The Committee will discuss qualifications of candidates nominented for appointment to the ACRS.

A portion of this sension will be closed to discuss matters the release of which would represent a clearly unwarranted invasion of personal privacy.

5:15 P.M.-6:45 P.M.: Preparation of ACBS
Reports (Open/Closed)—The Committee will
discuss proposed ACRS reports on matters
Considered during this

considered during this meeting.

A portion of this session will be closed to discuss a proposed ACRS report on the DOD/DOE Neval Reactors Design that will involve discussion of classified information.

Friday, August 5, 1994

8:30 A.M.-8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 A.M.-9:45 A.M.: Meeting with the Director of the NRC Office of Nuclear Reactor Regulation (NRT) (Open)—The Committee will meet with Mr. W.T. Russell, Director of NRR, to discuss items of mutual interest.

9 45 A.M.—10:30 A.M.—Report of the Planning and Procedures Subcommittee (Open/Closed)—The Committee will hear a report of the Planning and Procedures

Subcommittee on matters related to the conduct of ACRS business and internal organizational and personnel matters relating to the ACRS staff members.

A portion of this session may be closed to discuss matters that relate solely to internal personnel rules and practices of this advisory Committee, and matters the release of which would represent a clearly unwarranted investors of personal privacy.

10:45 A.M.-11:15 A.M.: Future ACRS
Activities (Open)—The Committee will
discuse topics proposed for consideration
during future ACRS meeting

during future ACRS meeting.

11:15 A.M.-11:30 A.M.: Reconciliation of ACRS Comments and Recommendations (Open).—The Committee will discuss responses from the NRC Executive Director for Operations to recent ACRS comments and recommendations.

11:30 A.M.-12:30 P.M.: Strategic Planning (Open)—The Committee will discuse items of significant importance to the ACRS and Commission.

1:30 P.M.-5:00 P.M.: Preparation of ACRS
Reports (Open/Closed)—The Committee will
discuss proposed ACRS reports on meetings
considered during this meeting.

A portion of this session dealing with the ACRS report on DOD/DOE Naval Reactors design with be closed to discuss classified information.

5:00 P.M.-5:15 P.M.: Activities of the ACRS Subcommittee Individual Members (Open)— The Committee will hear reports on the activities of ACRS Subcommittees and individual members.

5:15 P.M.—5:30 P.M.: Miscellaneous
(Open)—The Committee will discuss
miscellaneous matters related to the conduct
of Committee activities and complete
discussion of topics that were not completed
during previous mostings as time and
avalableity of information petra it.

Procedures for the conduct of and participation in ACRS meetings were published in the Federal Regimer on September 30, 1993 (58 FR 51118). hr accordance with these procedures, oral or written statements may be presented by members of the public, electronic recording will be permitted only during the open portions of the meeting, and questions may be asked only by members of the Committee. its consultants, and staff. Persons desiring to make oral statements should notify the ACRS Executive Director, Dr. John T. Lerkins, as fer in sevance as practicable so that appropriate arrangements can be made to allow the necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during this meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set saide for this purpose may be obtained by contecting the ACRS Executive Director prior to the meeting. In view of the possibility that the schedule for ACRS meeting rasy be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the ACRS Executive Director if such rescheduling would result in major inconvenience.

I have determined in accordance with Subsection 16(d) P.L. 92-463 that it is

necessary to chose portions of this meeting noted above to discuss information that involves the internal personnal rules and practices of this advisory Committee per 5 U.S.C. 552b (c)(2); to discuss information the release of which would represent a clearly unwarranted invasion of personal privacy per 5 U.S.C. 552b(c) (e); and to discuss privileged and classified information per 5 U.S.C. 552b (c)(1).

Further information regarding topics to be discussed, whether the miseting has been cancelled or rescheduled, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefore can be obtained by contacting the ACRS Executive Director, Dr. John T. Larkins (telephone 301-415-7361), between 7:30

A.M. and 4:15 P.M. EST.

Dated: July 19, 1994

Andrew L. Bates.

Advisory Committee Management Officer. [FR Doc. 94–17990 Filed 7–22–94, 8:45 am] BILLING CODE 7569-01-M

Detroit Edison Co.; Notice of Issuance of Amendment to Facility Operating License

[Doctost No. 50-341]

The U.S. Nuclear Regulatory
Commission (Commission) has issued
Amendment No. 101 to Facility
Operating License No. NPF-43 issued to
Detroit Edison Company (the licensee),
which revised the Technical
Specifications for operation of the Fermi
2 facility located in Monroe County,
Michigan. The amendment is effective
as of the date of immance with full
implementation within 45 days.

The amendment modified the Technical Specifications to make changes to job takes including a provision for future title changes, made changes to the onsite review organization (OSRO) membership and certain changes to the overtime controls

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the Federal Register on September 2, 1993 (58 FR 46664). No request for a hearing or petition for leave to intervene was filed following this notice, A requested title change to TS 6.2.2 g was withdrawn by the licensee's letter date 7, 1994 as it was no longer needed.



UNITED STATES NUCLEAR REGULATORY COMMISSION ADVISORY COMMITTEE ON REACTOR SAFEGUARDS WASHINGTON, D. C. 20656

July 25, 1994

SCHEDULE AND OUTLINE FOR DISCUSSION 412TH ACRS MEETING AUGUST 4-5, 1994

Thursday, August 4, 1994, One White Flint North, 11555 Rockville Pike, Rockville, Maryland, Commissioners' Conference Room

1) 8:30 - 8:45 A..M.

Opening Remarks by the ACRS Chairman (Open)

1.1) Opening Statement (TSK/SD)

9:40 2) 8:45 - 10:00 A.M. 1.2) Items of Current Interest (TSK/JTL/SD)
1.3) Priorities for Preparation of ACRS Reports (TSK/SD)

Steam Generator Tube Integrity Matters (Open) (WJS/NFD)

2.1) Remarks by the Subcommittee Chairman

2.2) Briefing by and discussions with representatives of the NRC staff and Nuclear Energy Institute regarding the proposed Generic Letter on Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes, and an Advance Notice of proposed Rulamaking regarding steam generator surveillance and maintenance.

9:50 10 10:00 - 10:15 A.M.

Break

DOD/DOE Naval Reactors Design (Closed) (CM/PAB) 3.1) Remarks by the Subcommittee Chairman

3.2) Briefing by and discussions with representatives of the NRC staff and Department of Defense (DOD)/Department of Energy (DOE) regarding the latest nuclear plant propulsion design being installed in submarines of the Seawolf class

(Note: This portion of the meeting will be closed to discuss classified information applicable to this matter.)

30 20 1:00 - 2:00 P.M. 20 - 30 P.M.

LUNCH

Reactor Trip Event at Salem Nuclear Plant, Unit ! (Open) (JCC/MDH)

4.1) Remarks by the Subcommittee Chairman

4.2) Briefing by and discussions with representatives of the NRC staff regarding the April 7. 1994 reactor trip event at the Salem Nuclear Plant, Unit 1 and the findings and recommendations of the Augmented Inspection Team (AIT)

= Transcribed portion of the meeting

30 4:55 3:00 - 5:15 P.M. 5) (4:00-4:15 Break) 30 45

Selection of New ACRS Members (Open/Closed)

5.1) Remarks by the ACRS Chairman

5.2) Discussion of the qualifications of the candidates nominated for appointment to the ACRS

(Note: A portion of this session will be closed to discuss matters the release of which would constitute a clearly unwarranted invasion of personal privacy.)

4:55 5:15 - 6:45 P.M. 6)

5:30 - 6:45 P.M.

Preparation of ACRS Reports (Open/Closed) Discussion of proposed ACRS reports on: - 6.1) DOD/DOE Naval Reactors Design (Closed) (CM/PAB)

9:40-9:50 A.M. - 6.2) Steam Generator Tube Integrity Matters (Tentative) (WJS/NFD) 4:55-5:30 P.M.

(Note: A portion of this session dealing with the proposed ACRS report on DOD/DOE Naval Reactors design will be closed to discuss classified information.)

Friday, August 5, 1994, One White Flint North, 11555 Rockville Pike, Rockville, Maryland, Commissioners' Conference Room

8:30 - 8:35 A.M. 7)

Opening Remarks by the ACRS Chairman (Open)

8) 8:35 - 9:45 A.M.

Meeting with the Director of the NRC Office of Nuclear Reactor Regulation (NRR) (Open) (TSK/MME) 8.1) Remarks by the ACRS Chairman

8.2) Briefing by and discussions with Mr. W. T. Russell. Director of NRR, on items of mutual interest

11:05 11:30 10-30 A.M.

Report of the Planning and Procedures Subcommittee (Open/Glosed) (TSK/JTL) Report of the Planning ad Procedures Subcommittee on matters

related to the conduct of ACRS business, and organizational and personnel matters relating to ACRS staff members. (Note: A portion of this session may be closed to discuss organizational and personnel matters that relate solely to the internal personnel rules and practices of this advisory Committee, and matters the release of which would constitute a clearly unwarranted invasion of personal privacy.)

11:05 45 10:30 - 10:45 A.M. 11:30 12:10 11:15 A.M

BREAK

Future ACRS Activities (Open) (TSK/SD) Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings. 11) 11:15 - 11:30 A.M.

Reconciliation of ACRS Comments and Recommendations (Open) (TSK, et al., SD, et al.

Discussion of responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports

Postponed 12) 11:30 - 12:30 P.M.

Strategic Planning (Open) (TSK/JTL) Discussion of issues of significant importance to the ACRS and the Commission

- 1:30 P.M.

(3:30-3:45 Break)

LUNCH

Room TBE3

Preparation of ACRS Reports (Open/Closed)

Discussion of proposed ACRS reports on:
1:45-3:00 p.m.-13.1) DOD/DOE Naval Reactors Design (Closed) (CM/PAB)
13.2) Steam Generator Tube Integrity Matters (Tentative) (WJS/NFD)

(Note: A portion of this session dealing with the proposed ACRS report on DOD/DOE Naval Reactors design will be closed to discuss classified information.)

14) 5:00 - 5:15 P.M.

Activities of the ACRS Subcommittees/Individual Members (Open)

Discussion of the activities of the ACRS Subcommittees and individual members

15) 5:15 5:30 P.M.

Miscellaneous (Open) (TSK/JTL) Complete discussion of matters considered during this meeting and matters considered but not completed during previous meetings as time and availability of information permit

NOTE: Presentation time should not exceed 50 percent of the total time allocated for a specific item. The remaining 50 percent of the time is reserved for discussion.

Number of copies of the presentation materials to be provided to the ACRS · 35.

APPENDIX III: MEETING ATTENDEES

412TH ACRS MEETING AUGUST 4-5, 1994

NRC STAFF

Α.	Adams	NRR	F.	Miraglia	NRR
F.	Allenspach	NRR	J.	Monninger	NRR
G.	Bagchi	NRR	J.	Murphy	RES
E.	Bennett	NRR	H.	Pastis	NRR
R.	Capra	NRR	S.	Richards	NRR
D.	Crutchfield	NRR	S.		NRR
J.	Donoghue	NRR	C.	Serpan	RES
R.	Eckenrode	NRR	М.	Slosson	NRR
B.	Elliot	NRR	J.	Sorensen	OCM/KR
M.	Fleishman	OCM/KR	J.	Stone	NRR
E.	Goodwin	NRR	R.	Summers	NRC RI
В.	Grimes	NRR	М.	Taylor	OEDO
J.	Jacobson	NRR	Α.	Thadani	NRR
N.	Kadambi	RES	М.		NRR
L.	Корр	NRR	C.	Thomas	NRR
J.	Kudrick	NRR	S.	Weiss	NRR
10000		NRR	J.	Wermiel	NRR
D.	Lange	NRR	C.	Willis	NRR
H.	Li		G.		NRR
J.	Lyons	NRR			
G.	Marino	RES	J.	Zimmerman	NRR
E.	Marinos	NRR			
Μ.	Mayfield	RES			

ATTENDEES FROM OTHER AGENCIES AND GENERAL PUBLIC

J.	Alf Barbeito Bell	Naval Reactors Bechtel NEI
	Gunter	NIRS
	Ludursky	Bettis
B.	Moore	SNC
R.	Pearson	NSP
J.	Trotyter	EPRI
	Villar	PSE&G
C.	Weltz	EPRI

M. Mendonca NRR

APPENDIX IV: FUTURE AGENDA

413th ACRS Meeting, September 8-10, 1994, Rockville, Maryland. During this meeting, the Committee plans to consider the following:

Proposed Generic Letter on Digital Instrumentation and Control (I&C) Systems Retrofits (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed Generic Letter on Digital I&C Systems Retrofits. Representatives of the industry will participate, as appropriate.

Proposed Generic Letter on Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the revised calculations for radiological consequences of main steamline break associated with degraded steam generator tubes.

Proposed Revisions to Appendix J to 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors" (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff and Nuclear Energy Institute (NEI) regarding the proposed revisions to Appendix J to 10 CFR Part 50.

Meeting with the NRC Commissioners (Open) - The Committee will meet with the NRC Commissioners to discuss items of mutual interest.

Regulatory Analysis Guidelines (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed final version of the Regulatory Analysis Guidelines document. Representatives of the industry will participate, as appropriate.

Report of the Planning and Procedures Subcommittee (Open/Closed) - The Committee will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business and internal organizational and personnel matters relating to the ACRS staff members.

Proposed Final Version of NUREG-1465, "Accident Source Terms for Light-Water Nuclear Power Plants" (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed final version of NUREG-1465. Representatives of the industry will participate.

<u>Vessel Head Penetration Cracking</u> (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff and NEI regarding the issues associated with the vessel head penetration cracking.

Appendix IV 412th ACRS Meeting

Generic Letter on Intergranular Stress Corrosion Cracking of Core Shrouds in BWR Plants (Open) - The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the Generic Letter on Core Shroud Cracking in BWR Plants. Representatives of the industry will participate, as appropriate.

Reconciliation of ACRS Comments and Recommendations (Open) - The Committee will discuss responses from the NRC Executive Director for Operations to ACRS comments and recommendations included in recent ACRS reports.

<u>Selection of New ACRS Members</u> (Open/Closed) - The Committee will discuss qualifications of candidates nominated for appointment to the ACRS.

<u>Subcommittee Activities</u> (Open) - The Committee will hear reports from cognizant Subcommittee Chairmen regarding the activities of Subcommittees.

Miscellaneous (Open) - The Committee will discuss miscellaneous matters related to the conduct of Committee activities and complete discussions of topics that were not completed during previous meetings as time and availability of information permit.

APPENDIX V LIST OF DOCUMENTS PROVIDED TO THE COMMITTEE

[Note: Some documents listed below may have been provided or prepared for Committee use only. These documents must be reviewed prior to release to the public.]

MEETING HANDOUTS

AGENDA ITEM NO.

DOCUMENTS

4 Reactor Trip Event at Salem Nuclear Power Plant, Unit 1
1. ACRS Presentation on Operating Reactor Events, dated

August 4, 1994 [Viewgraphs]

NRC Information Notice 94-55: Problems with Copes-Vulcan Pressurizer Power-Operated Relief Valves, dated August 4,

1994

8 Meeting with the Director of the NRC Office of Nuclear Reactor Regulation (NRR)

3. SECY-94-113, Senior Management Meeting Review Process,

dated April 22, 1994

3a. Memorandum to ACRS Members from Medhat El-Zeftawy, dated August 3, 1994, regarding Meeting with the Director of the NRC Office of Nuclear Reactor Regulation, with enclosures [Handout #8.1]

4. Customized Inspection Planning Process, dated August 1994

[Viewgraphs]

5. Letter to R.E. Link, Vice President, Wisconsin Electric Power Company, from T.O. Martin, Deputy Director, Region III, NRC, dated May 24, 1994, regarding Point Beach Evaluation and Assessment Team Site Visit, April 18-22, 1994, with enclosure

6. Construction Inspection Program, dated August 5, 1994

[Viewgraphs]

9 Report of the Planning and Procedures Subcommittee

7. Final Draft Minutes of Planning & Procedures Subcommittee Meeting, dated August 3, 1994 [Handout #9.1]

10 Future ACRS Activities

8. Memorandum to ACRS Members from Sam Duraiswamy, dated August 1, 1994, regarding Future ACRS Activities - 413th ACRS Meeting, September 8-10, 1994, with enclosure [Handout #10.1]

MEETING NOTEBOOK CONTENTS

TAB

DOCUMENTS

2 Steam Generator Tube Integrity Matters

- 1. Table of Contents
- 2. Tentative Agenda
- 3. Status Report
- 4. Draft NRC Generic Letter 94-xx, Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes, undated
- 5. Memorandum to James M. Taylor, EDO, from Joram Hopenfeld, RES, dated July 13, 1994, regarding Differing Professional Opinion Regarding Voltage-Based Interim Repair Criteria for Steam Generator Tubes
- 6. Draft Advanced Notice of Proposed Rulemaking on Steam Generator Tube Integrity, undated

4 Reactor Trip Event at Salem Nuclear Power Plant, Unit 1

- 7. Table of Contents
- 8. Tentative Agenda
- 9. Status Report
- 10. NRC Augmented Inspection Team (AIT) Report Nos. 50-272/94-80 and 50-311/94-80, dated June 24, 1994 [9407010048]
- 11. Memorandum to Brian K. Grimes, Director, Division of Operating Reactor Support, from Alfred E. Chaffee, Chief, Events Assessment Branch, dated April 18, 1994, regarding Operating Reactors Events Briefing April 13, 1994 Briefing 94-13, with enclosures
- 12. NRC Information Notice 94-36, dated May 24, 1994, regarding Undetected Accumulation of Gas in Reactor Coolant System [9405190137]