NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Beaver Valley Power Station, Unit 1	0 5 0 0 0 3 3 4	1 OF 0 2

TITLE (4) Failure of Containment (CNMT) Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											N/A	0 5 0 0 0
0 7	0 4	8 4	8 4	–	0 0 7	–	0 0	0 8	0 1	8 4	N/A	0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Robert J. Druga, Chief Engineer	AREA CODE 4 1 2 — 6 4 3 - 1 2 6 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	K M	I S V X	M 1 2 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/4/84, at 2138 hours, during normal operations, the Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve failed shut. This caused a loss of cooling water flow to the CNMT Air Recirculation Cooling Coils and to the CNMT Instrument Air Compressors. The loss of cooling water to the Air Recirculation Cooling Coils resulted in increasing CNMT temperatures. Subsequent attempts to restore cooling water were unsuccessful and at 2238 hours, CNMT temperature reached 105.08 degrees. Technical Specification 3.6.1.5 requires the CNMT temperature to be less than 105 degrees. Station Management then elected to reduce power to effect valve repairs and to reduce CNMT temperature. At 2315 hours, the Instrument Air to CNMT Instrument Air Isolation Valve was opened to supply CNMT with Instrument Air. This was done due to the loss of cooling to the CNMT Instrument Air Compressors. At 2318 hours, CNMT temperature reached 106.38 degrees. A manual shutdown to Hot Standby was commenced. The cause for the Chilled Water System Isolation Valve failing shut was due to a failed pneumatic valve diaphragm. This diaphragm was replaced. There were no safety implications to the public because the reactor was placed in a safe, controlled shutdown condition and the River Water System was operable at all times as an additional source of cooling if necessary.

IE22 1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Beaver Valley Power Station, Unit 1	0 5 0 0 0 3 3 4	8 4	— 0 0 7	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 7/4/84, at 2138 hours, during normal operations at 100% reactor power, the Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve [TV-CC-110D] failed shut. This caused a loss of cooling water flow to the Containment Air Recirculation Cooling Coils and the Containment Air Compressors. The loss of cooling water to the Containment Air Recirculation Cooling Coils resulted in increasing Containment temperatures. Subsequent attempts to open the isolation valve [TV-CC-110D] were unsuccessful and at 2238 hours, Containment temperature reached 105.08 degrees. Technical Specification 3.6.1.5 requires the Containment temperature to be less than 105 degrees. Station management then elected to reduce power to effect valve repairs and to attempt to reduce Containment heat load. At 2311 hours, a Containment entry was made to investigate the failure of [TV-CC-110D]. At 2315 hours, the Instrument Air to Containment Instrument Air Isolation Valve [IA-90] was opened to supply Containment with Instrument Air. This was done due to the loss of cooling water to the Containment Air Compressors. At 2318 hours, Containment temperature reached 106.38 degrees. A controlled manual shutdown to Hot Standby was commenced due to the increasing Containment temperatures. On 7/5/84, two additional Containment entries were made to investigate the failure of [TV- CC-110D]. It was determined that [TV-CC-110D] failed shut due to a failed pneumatic valve diaphragm. An additional Containment entry was made on 7/5/84 to erect scaffolding to effect repairs on [TV-CC-110D]. On 7/7/84 at 2000 hours, the valve operating diaphragm and air regulator on [TV-CC-110D] were replaced. No other corrective actions are planned.

There were no safety implications to the public because the reactor was placed in a safe shutdown condition and the River Water System was operable at all times as an additional source of cooling if necessary.

The Containment Recirculation Cooling Coils Chilled Water System Outlet Isolation Valve [TV-CC-110D] is a Masoneillan Trip Valve, Model No. 38-20761. This is the first reported failure of this valve.

Duquesne Light

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August 1, 1984
NDISS1:2130

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
LER 84-007

Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region 1
Park Avenue
King of Prussia, PA 19406

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-007, 10 CFR 50.73.a.2.i, "Completion of Nuclear Plant Shutdown required by Technical Specifications."

Very truly yours,

Wm. S. Lacey
Station Superintendent

Attachment

cc: Director of Management & Program Analysis
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C. A. Roteck, Ohio Edison

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