

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Beaver Valley Power Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 3 4	PAGE (3) 1 OF 0 1
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TITLE (4)
Inadvertent Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	0 6	8 4	8 4	0 0 8	0 0 0	8 0	1 8	4	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

OPERATING MODE (9) 5

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
20.406(a)(1)(i)	80.38(a)(1)		80.73(a)(2)(v)	73.71(c)
20.406(a)(1)(ii)	80.38(a)(2)		80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.406(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(viii)(A)	
20.406(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(viii)(B)	
20.406(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Robert J. Druga, Chief Engineer	TELEPHONE NUMBER 4 1 2 6 4 3 - 1 2 6 4
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2140 hours, during the performance of Maintenance Surveillance Procedure 26.01 (P-446, Turbine First Stage Pressure Protection Channel III Test), the reactor shutdown bank rods tripped on a pressurizer low pressure trip signal. The trip occurred when the technicians input a test signal for the turbine first stage pressure channel. When this was done, the P-7 interlock (which unblocks the pressurizer low pressure reactor trip above 10 percent power) was activated and caused the reactor trip. Because the plant was in cold shutdown, no action beyond the immediate verification of the trip was necessary.

It was later discovered that, contrary to the initial conditions of MSP 26.01, the test was performed with the reactor trip breakers closed. The Shift Supervisor and I&C personnel have been counseled as to the importance of thoroughly reviewing all procedures prior to their performance. An evaluation of the procedure in question has been made and it was judged that in this case, a procedure change is necessary for clarification.

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PDR ADOCK 05000334
S PDR

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Duquesne Light

Nuclear Division
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Telephone (412) 393-6000

August 1, 1984
NDISS1:2131

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
LER 84-008

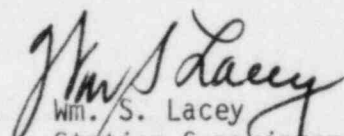
Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region 1
Park Avenue
King of Prussia, PA 19406

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-008, 10 CFR 50.73.a.2.iv, "Engineered Safeguards Features System Actuation."

Very truly yours,


Wm. S. Lacey
Station Superintendent

Attachment

T. E. Murley
July 20, 1984
ND1SS1:2131
Page two

cc: Director of Management & Program Analysis
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Washington, D.C. 20555

C. A. Roteck, Ohio Edison

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