

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 7 3	PAGE (3) 1 OF 0 3
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TITLE (4)
Reactor Water Clean-Up Differential Flow Isolation

EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 4	1 5	8 4	8 4	0 2 3	0 1	0 8	0 3	8 4	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

OPERATING MODE (8) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 6: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 1	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)					
	20.406(a)(1)(ii)	80.38(c)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(c)					
	20.406(a)(1)(iii)	80.38(c)(2)	<input type="checkbox"/>	80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.406(a)(1)(iv)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(viii)(A)						
	20.406(a)(1)(v)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(viii)(B)						
20.406(a)(1)(vi)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER
NAME JoAnn M. Shields, Extension #330.	AREA CODE 8 1 1 5	3 1 5 1 7 1 - 1 6 1 7 1 6 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	CIE	IRIV	L121615	N						

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Unit 1 Reactor Water Clean-Up System isolated on April 15, 1984, at 1754 due to high differential flow, attributed to losses through a stuck open relief valve on the shell side of the regenerative heat exchanger. The "A" heat exchanger string was isolated and the system was returned to service 2249 using the "B" heat exchanger string. All actions occurred in accordance with system design. Safe plant conditions were maintained at all times.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) LaSalle County Station Unit 1	DOCKET NUMBER (2) 0500037384	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	023	01	02	OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. EVENT DESCRIPTION

Unit 1 Reactor Water Clean-Up System (CE) isolated on April 15, 1984, at 1754 due to high differential flow. The Reactor was in Startup Mode, at about 1% power.

II. CAUSE

The Reactor Water Clean-Up System (CE) was operating, running through the "A" heat exchanger string and discharging to the condenser. The clean-up regenerative heat exchanger shell side relief valve, 1G33-F340A, lifted and remained stuck open. Isolation of the system occurred as system differential flow reached the trip point of 70 gpm. While investigating the cause of the isolation, the system isolated a second time on high differential flow at 1818. Upon discovery of the lifted relief, the "A" heat exchanger train was isolated, and the system was returned to service at 2249 on April 15, 1984.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The event was of minimal significance as the Reactor Water Clean-Up System isolated according to design. Flow through the relief valve discharged to the Reactor Building equipment drain tank in accordance with design. Safe plant conditions were maintained at all times.

IV. CORRECTIVE ACTION

1. The "A" heat exchanger string was isolated and the system was restarted, running through the "B" heat exchanger string.
2. A Work Request was written to inspect and repair the relief valve. When the valve was removed from the process line, it was verified that no foreign material was in the lines. Disassembly of the valve revealed a cut disc. The disc was replaced and lapped to nozzle per Lonergan vendor manual. The valve was then re-assembled with a new disc, stem and gaskets. The valve relief setpoint was then verified accurate by three bench tests, and the valve was re-installed in process line.

When the heat exchanger string was then pressurized during operation, no leakage was observed either through the relief or at the flange.

Repairs to the relief valve were completed on June 8, 1984. Since the initial event on April 15, 1984, there have been no re-occurrences of this type.

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FACILITY NAME (1) LaSalle County Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 7 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	- 0 2 3	- 0 1	0 3	OF

TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. PREVIOUS EVENTS

A similar event occurred on Unit 2 and is detailed in LER 84-013-00/374.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

JoAnn M. Shields, 815/357-6761, Extension 330.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 3, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-023-01, Docket #050-373 is being submitted to your office to supercede previously submitted Reportable Occurrence Report 84-023-00.

G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/ph

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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11