

LICENSEE EVENT REPORT (LER)

Updated report: Initial report dated April 17, 1984

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9 1	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
0 3	2 3	8 4	8 4	0 0 1	0 1	0 8	0 9	8 4	N/A		
									DOCKET NUMBER(S) 0 5 0 0 0		
									N/A		
									0 5 0 0 0		

OPERATING MODE (8) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 1 5	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)	
NAME Gerald W. Bergan (X-529)	TELEPHONE NUMBER AREA CODE: 8 1 1 5 8 1 1 5 9 4 2 1 - 2 9 1 2 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPPDS	
D										

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)		
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO		MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a reactor startup subsequent to a refueling outage, with Reactor heat up and pressurization under way, reactor water level decreased to the low level alarm point. With the low-flow feedwater regulating valve wide open in response to the low water level, the first reactor feed pump was started. Excessive cold water entering the reactor caused a high neutron flux reactor scram.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Dresden Nuclear Power Station Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4 -	0 0 1 -	0 1	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 306A's) (17)

Following a refueling outage, with Unit 3 reactor in the startup mode, reactor power had increased to the heating range with control rods being notched out to maintain proper heat up rate. A second Nuclear Station Operator (NSO) was assisting in maintaining the appropriate heat up rate by increasing the turbine pressure set. One condensate/booster pump was on and supplying coolant to the reactor vessel. At 310 psig reactor pressure and with two turbine steam bypass valves open, it was noticed by the NSO pulling control rods that the reactor water level was decreasing and the low flow feedwater regulating valve was wide open. After verifying the condensate pump had not tripped, the NSO attempted to open a main feedwater isolation valve. While attempting to open the main feedwater isolation valve, water level continued to drop to the low level alarm point of 20 inches. The NSO then started a first reactor feed pump to recover the loss of coolant inventory. This action introduced excessive cold water in the vessel via a wide open low flow feedwater regulating valve. The reactor scrammed on high neutron flux conditions at 15 percent APRM power while the reactor mode switch was in the startup position. All appropriate protective systems performed as designed.

Coolant loss was compounded by the fact that two bypass valves were open. Surveillances, DOS 250-5, Automatic Blowdown System at Low Pressure and Rated Pressure, and DOS 2300-3, HPCI System Pump Test, require some bypass steam flow as a prerequisite to avoid any reactor pressure and flux transients. The two bypass valves remained open after testing, resulting in a drop in water level which the condensate/booster pump was unable to restore.

Corrective action consisted of submitting permanent revisions to procedures DOS 250-5 and DOS 2300-3, reminding the NSO to close turbine bypass valves when testing is complete.

SUPPLEMENT TO DVR

DVR NO.			
STA	UNIT,	YEAR	NO.
D - 12	- 3	- 84	- 9

PART 1	TITLE OF EVENT	OCCURRED	
	Reactor Scram	<u>3/23/84</u>	<u>1308</u>
		DATE	TIME
REASON FOR SUPPLEMENTAL REPORT			
Update original report to include a cause for this event.			
PART 2			
ACCEPTANCE BY STATION REVIEW	<u>J. Brunner</u>	<u>J.A. Cusla</u>	_____
DATE	<u>8/13/84</u>	<u>8/13/84</u>	_____
SUPPLEMENTAL REPORT APPROVED AND AUTHORIZED FOR DISTRIBUTION	<u>Douglas D. Ott</u>		<u>8/14/84</u>
	STATION SUPERINTENDENT		DATE



Commonwealth Edison

Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

August 9, 1984

DJS Ltr #84-789

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Updated Licensee Event Report #84-001-1, Docket #050-249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv). This report specifies a cause for this event. The initial report, dated April 17, 1984, contained no event cause due to a misunderstanding in the use of the new Licensee Event Report form.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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