November 22, 1995

Mr. Harry P. Salmon, Jr.
Resident Manager
New York Power Authority
James A. FitzPatrick Nuclear Power Plant
Post Office Box 41
Lycoming, NY 13093

SUBJECT: JAMES A. FITZPATRICK MOTOR-OPERATED VALVE INSPECTION 95-20

Dear Mr. Salmon:

This letter transmits the results of an inspection performed between October 23-27, 1995, by Mr. D. Dempsey and others at the James A. FitzPatrick Nuclear Power Plant. The inspection was focused on issues important to public health and safety, specifically your program implemented in response to NRC Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance." The preliminary inspection findings were discussed with Mr. D. Lindsey and other members of your staff on October 27, 1995.

The NRC concludes that you have implemented a program that verifies the design-basis capability of the safety-related motor-operated valves (MOVs) at FitzPatrick. Effective management support was evident in the large number of valves dynamically tested, the quality of your implementing procedures and engineering evaluations, and your staff's participation in various industry MOV groups. Notable program attributes included independent verification of design-basis calculations utilizing diverse methodologies, and well-documented program scope evaluations.

We are not determining the acceptability of your long-term plans for periodically verifying the design-basis capability of safety-related MOVs at this time, since the NRC is preparing a generic letter to address this issue. You should review your plans in light of the new generic letter (when issued), and consider any appropriate adjustments. New information on valve performance, such as: (1) GL 95-07 on pressure locking and thermal binding of gate valves; and (2) the results of the Electric Power Research Institute's MOV testing program, also will need to be assessed to ensure that safety-related MOVs remain capable of performing their design-basis functions. The NRC will review your response to these issues and other additional information on MOV performance as part of future staff activities.

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No response to this letter is requested. Your cooperation in these matters is appreciated, and we would be pleased to discuss any questions you have concerning this inspection.

Sincerely,

(original signed by)

Eugene M. Kelly, Chief Systems Engineering Branch Division of Reactor Safety

Docket No. 50-333

Enclosure: Inspection Report No. 50-333/95-20

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