

November 24, 1995

Mr. L. F. Storz - Senior Vice President
Nuclear Operations
Public Service Electric and Gas Company
PO Box 236
Hancocks Bridge NJ 08038

SUBJECT: NRC INSPECTION NOS. 50-354/95-11

Dear Mr. Storz:

This letter refers to your October 19, 1995 correspondence, in response to our September 19, 1995 letter.

To fully and comprehensively address the July 1995 surveillance violations, we noted that you committed to implement a Technical Specification Surveillance Improvement Program (TSSIP). The TSSIP has a broad charter to address fundamental, long standing weaknesses with surveillance activities. We noted this as a good initiative capable of resolving these surveillance inadequacies; however, we remain concerned with the breadth of your immediate corrective actions for the reference surveillance violations. Recently, on November 13, 1995, the TSSIP identified that additional contacts within the 4160 VAC emergency bus undervoltage (UV) relays may not have been appropriately tested. This resulted in plant staff declaring all the affected UV relays inoperable. This recent problem is similar to that referenced in the violation since the connected contacts are from the same UV relays. Again, this problem may have existed since initial plant licensing and indicated weak or inadequate overlap testing. Thus, it appears that your immediate corrective action in response to the July event may not have been comprehensive or timely. While we laud your efforts to address the surveillance problems generically through the TSSIP, we remain concerned with the adequacy of your immediate corrective action. As such, we request that you revise your response to further address this concern. Your revised response is requested within 30 days of receipt of this letter.

Your cooperation with us is appreciated.

Sincerely,

ORIGINAL SIGNED BY:

Larry E. Nicholson, Chief
Projects Branch 3
Division of Reactor Projects

Docket No. 50-354

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cc:

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F. Thomson, Manager, Licensing and Regulation
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cc w/cy of Licensee's Letter:

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Public Service
Electric and Gas
Company

Louis F. Storz

Public Service Electric and Gas Company, P.O. Box 236, Hancocks Bridge, NJ 08038 609-339-5700

Senior Vice President - Nuclear Operations

OCT 19 1995

LR-N95175

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

REPLY TO A NOTICE OF VIOLATION
MISSED TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENTS
INSPECTION REPORT NO. 50-354/95-11
HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354

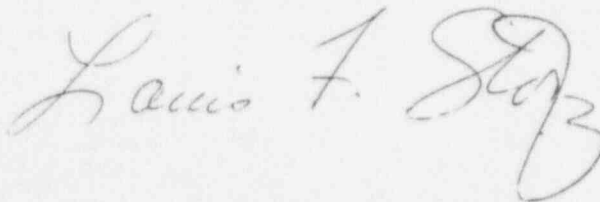
Pursuant to the provisions of 10CFR2.201, this letter submits the response of Public Service Electric and Gas Company to the notice of violation issued to the Hope Creek Generating Station (HCGS) in a letter dated September 19, 1995.

The details concerning these issues are documented in Licensee Event Report (LER) 95-017-00 entitled "Inadequate Testing of Emergency Bus Undervoltage Logic Circuitry - Missed Surveillance Test, Diesel Generators Inoperable," dated August 11, 1995, and LER 95-018-00 entitled "Missed Surveillance Test Due to Procedure and Personnel Error - Automatic Depressurization and High Pressure Coolant Injection Systems Inoperable," dated August 17, 1995.

Also, as requested, this letter provides a comprehensive description of HCGS's planned corrective actions to improve our current Surveillance Requirement Program.

Should you have any questions or comments on this transmittal, do not hesitate to contact us.

Sincerely,



Attachment (1)

~~9510250083 HPP~~

OCT 19 1995

C ✓ Mr. T. T. Martin, Administrator - Region I
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ATTACHMENT

REPLY TO NOTICE OF VIOLATION
MISSED TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENTS
INSPECTION REPORT NO. 50-354/95-11
HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354

LR-N95175

I. INTRODUCTION

Recent performance at Hope Creek has resulted in a significant number of Licensee Event Reports (LERs) relative to missed surveillance activities. As a result of this Technical Specification (TS) surveillance testing performance, the Technical Specification Surveillance Improvement Program (TSSIP) project has been initiated by the General Manager of Hope Creek Generating Station (HCGS). The project will be completed by a multi-disciplinary team consisting of approximately 5 to 6 Nuclear Business Unit (NBU) staff and contract personnel on a full time basis.

In support of this project, a review of the LERs related to missed TS surveillance requirements has been performed. The main contributing factors to the missed TS surveillance requirements are inconsistencies between surveillance procedures or scheduling requirements and the TS requirements, such as:

- * Incorrect periodicity of the surveillance;
- * Incorrect operational condition in which the surveillance should be performed;
- * Omission of a component from a system test; and
- * Incorrect testing methodology.

The charter of the TSSIP project is to compare the TS requirements of section 4.0 (with the exception of section 4.0.5 requirements) to the surveillance procedures to verify that all requirements are met. This comparison will include verification that:

- * Procedures test the entire scope of the TS system (channel, logic system, etc.), including inputs, sensors, indicators, alarms and trip functions as applicable,
- * Numerical values, setpoints, tolerances, calculations, graphs, figures and tables included or referenced in the procedures are consistent with values specified in the TS,

- * Procedure steps, including prerequisites and special conditions (i.e. any operational condition restriction), associated with satisfying TS Surveillance Requirements are correct and identified in the procedure,
- * Procedures satisfy the intent of the surveillance requirement as stated in the TS Bases,
- * Procedure acceptance criteria satisfy the TS surveillance requirements and acceptance criteria have referenced bases, and
- * The computerized surveillance scheduling program correctly identifies the required frequency and Operational Condition for performance.

This program is being closely modelled after the TSSIP effort recently completed at Washington Public Power Supply System (WNP-2). Bases packages will be prepared for each TS Surveillance Requirement. We are confident that this project will resolve deficiencies associated with missed or improperly performed TS surveillance requirements.

II. REPLY TO THE NOTICE OF VIOLATION

1. Description of the Notice of Violation

"Hope Creek technical specification 4.0.2 requires in part, that each surveillance requirement shall be performed within its specified surveillance interval with a maximum allowable extension not to exceed 25 percent of the specified surveillance interval.

Contrary to the above, during the period of July 13 to 20, 1995, certain surveillance requirements were not performed within the specified intervals, including the maximum allowable extension, as stated in the following examples:

- (i) Hope Creek technical specifications 4.8.1.1.2.h.4.a and 6.a require in part, that each diesel generator be demonstrated operable at least once per 18 months during shutdown, by simulating a loss of offsite power and verifying load shedding from the emergency busses. On July 13, 1995, it was determined by the licensee

that surveillance testing omitted complete verification of the circuits associated with the vital bus load shedding in response to the loss of offsite power signals. Specifically, overlap testing between bus loss of power auxiliary relays and the individual load breaker trip circuits had never been adequately demonstrated since initial plant licensing.

- (ii) Hope Creek technical specification 4.3.3.1 requires in part, that each ECCS actuation instrumentation channel shall be demonstrated operable by the performance of a quarterly channel functional test. On July 20, 1995, it was determined by the licensee that various specified ECCS actuation instrumentation channel functional tests were not completed within the specified intervals for the following three examples:
- (1) on June 6, 1995, the condensate storage tank level-low instrument channel functional test for the HPCI system suction swap was improperly credited by the completion of a suppression pool level-high instrument channel calibration;
 - (2) on June 29, 1995, the "C" RHR and "A" core spray pump discharge pressure-high permissive instrument channel functional tests for the automatic depressurization system were improperly credited by the completion of an "A" RHR pump discharge pressure-high permissive channel calibration; and,
 - (3) on June 29, 1995, the "B" and "D" RHR pump discharge pressure-high permissive instrument channel functional tests for the automatic depressurization system were improperly credited by the completion of an "B" core spray pump discharge pressure-high permissive channel calibration."

Discussion for Example i:

2a. Reason for Violation

PSE&G has reviewed the circumstances described by the NRC and concurs with the facts cited in the violation.

On July 13, 1995, discrepancies regarding the logic of Loss Of Power (LOP) circuits were self identified on plant drawings during routine work. In response to discovery of

the drawing discrepancies, a review of surveillance testing was performed. This review revealed that the testing performed to verify TSS 4.8.1.1.2.h.4.a and 6.a was incomplete. These TSS address diesel generator testing and require the simulation of a LOP, and a LOP in conjunction with an Emergency Core Cooling System (ECCS) actuation test signal. The overlap in the surveillance testing did not adequately test all segments of the circuits associated with load shedding in response to a LOP signal. While all the relays in the circuits were cycled during testing, a small number of the relay contacts and their associated functions were not fully tested.

Since operability of the load shedding feature had not been demonstrated, the four diesel generators were declared inoperable in accordance with TSS at 2100 hours on July 13, 1995. Upon declaring the diesel generators inoperable, operating shift personnel reviewed and discussed TSS 3.8.1.2 and 4.0.3. At the time, the operating shift personnel considered TS 4.0.3 to be governing, with TS 3.8.1.2 to be invoked following the twenty-four hour grace period allowed by TS 4.0.3. Both TS were entered on the Limiting Condition for Operation (LCO) log.

Following discussions among operating shift personnel and various department representatives, at approximately 0830 hours on July 14, 1995, it was determined that the governing TS was 3.8.1.2, and not 4.0.3. With four diesel generators inoperable, the minimum A.C. power sources required to be operable by TS 3.8.1.2 were considered to be functional but not operable in accordance with TSS. The associated action statement requires, in part, the suspension of core alterations, suspension of handling of irradiated fuel in the secondary containment, suspension of operations with a potential for draining the reactor vessel, and suspension of crane operations over the spent fuel storage pool when fuel assemblies are stored therein. When the LCO form was updated to reflect the correct governing TS, the required actions were not captured. Compliance with the required actions was verified although controls were not established.

The "C" diesel generator was declared operable at 1300 hours on July 16, 1995. The "A" diesel generator was declared operable at 1130 hours on July 17, 1995. The return of the "A" diesel generator to operable condition restored the required minimum A.C. power sources of two as required by TSS in Operational Condition 4.

As reported in LER 95-017-00, dated August 11, 1995, the cause of this event was procedural inadequacy. A

contributing cause was inadequate review of operating experience feedback of previous similar occurrences at other facilities. The test procedures did not provide sufficient overlap to ensure that TS requirements were met for the complete circuit. Failure to test segments of these circuits is attributed to inadequate technical input and review during the development of these test procedures. The error was not identified during subsequent reviews of the procedures since those reviews focused on the specific changes in that revision.

3a. Corrective Steps That Have Been Taken and Results Achieved

- a. Test procedures were prepared and previously untested segments of the circuits were tested satisfactorily.
- b. This event was reviewed with Licensed Operating personnel with focus on the applicability of TS 4.0.3 and the requirement to implement positive controls when directed by TSSs. Personnel involved have been counseled regarding their inadequate actions associated with the missed surveillance.
- c. The existing process for reviewing operating experience feedback (OEF) for applicability to Hope Creek was completed by an independent consultant. Recommendations from this review have been evaluated for implementation based upon the value added to the OEF process. Those recommendations that will improve the quality of the OEF process will be implemented by February 28, 1996. These recommendations will improve OEF coordination between NBU departments, and result in a more timely review of OEF.
- d. The Technical Specification Surveillance Improvement Program (TSSIP) was initiated.

4a. Corrective Steps that Will Be Taken to Avoid Further Violations

- a. Problems identified during implementation of the TSSIP will be appropriately communicated to the responsible department so that corrective actions can be taken. These corrective actions will prevent further violations. The TSSIP will be completed by 12/31/96.

5a. Date When Full Compliance Will Be Achieved

Full compliance was achieved upon proper completion of the following surveillance requirements on July 18, 1995:

- 1) Surveillance requirement 4.8.1.1.2.h.4.a, simulating a loss of offsite power by itself and verifying loss of power is detected and de-energization of the emergency busses and load shedding from the emergency busses, and
- 2) Surveillance requirement 4.8.1.1.2.h.6.a, simulating a loss of offsite power in conjunction with an ECCS actuation test signal and verifying loss of power is detected and de-energization of the emergency busses and load shedding from the emergency busses.

Discussion of Example ii:

2b. Reason for Violation

PSE&G has reviewed the circumstances described by the NRC and concurs with the facts cited in the violation, with the exception of the third example as discussed on page seven (7) of this response.

- (1) On June 5, 1995, with the plant in Operational Condition 1, credit was improperly taken for TS 4.3.3.1-1.3.c, functional testing of the HPCI Condensate Storage Tank (CST) low level suction transfer instrumentation channels, based on the performance of a single channel calibration. As discussed in LER 95-018-00, dated August 17, 1995, the credited functional test includes both CST level suction transfer instrument channels. The channel calibration procedure includes only one of these two instrument channels but allowed crediting of the functional test. The TS 4.3.3.1-1.3.c test interval was exceeded on July 6, 1995. HPCI was functional but inoperable per TS 4.3.3.1-1.3.c. TS 3.5.1.c Action Statement requires that with the HPCI System inoperable, the system be returned to operable status within fourteen days or be in at least Hot Shutdown within the following twelve hours. For reasons not related to this issue, the plant entered Operational Condition 4 (Cold Shutdown) on July 7, 1995. Operational Condition 3 (Hot Shutdown) was inadvertently entered on July 8-9, 1995, as reported in LER 95-016-01, dated October 2, 1995. The reactor steam dome pressure reached was less than 200 psig. HPCI is only required to be operable in

Operational Condition 3 when steam dome pressure is greater than 200 psig. The functional test of the missed channel was successfully performed on July 21, 1995, prior to entering Operational Condition 3 and reaching a steam dome pressure of 200 psig.

This inappropriately credited surveillance test was self identified during a review of functional test work history initiated following a similar occurrence on July 20, 1995, as discussed on page 7.

- (2) On November 13, 1991 and again on May 22, 1993, credit was improperly taken for functional testing of the Residual Heat Removal (RHR) "A" and "C" Pump Discharge Pressure (Automatic Depressurization System (ADS) Permissive) instrument channels based on the performance of a single channel calibration. The I&C Supervisor improperly allowed crediting of the functional test. ADS was functional but inoperable per TC 4.3.3.1-1.4.e until the next functional tests of the missed channels, performed successfully on December 12, 1991 and June 16, 1993.

The ADS circuits affected by these occurrences are part of the ADS initiation logic. Two initiation signals and one permissive signal are used to actuate ADS. These signals are reactor vessel low water level, high drywell pressure, and RHR and/or Core Spray (CS) pumps running. These permissive pump running signal indicates that RHR and/or CS is available to provide reactor vessel makeup water. In both of the above occurrences, two of the three pump running signals were functional but inoperable per TSs.

These inappropriately credited surveillance tests were self identified during a review of functional test work history initiated following a similar occurrence on July 20, 1995, as discussed in the next example.

- (3) On July 20, 1995, with the plant in Operational Condition 4 (Cold Shutdown), it was self identified during closure of surveillance test documentation that on June 28, 1995 an instrumentation functional test had been improperly credited as complete based on the performance of a channel calibration. The functional test included three instrument channel tests while the channel calibration includes only one of the three instrument channels required by the functional test. The surveillance would have been overdue on July 20, 1995. On July 8, 1995, for reasons not related to this

issue, the plant entered Operational Condition 4. Operational Condition 3 was inadvertently entered on July 8-9, 1995, as reported in LER 95-016-01, dated October 2, 1995. The reactor steam dome pressure reached was less than 100 psig. ADS is only required to be operable in Operational Condition 3 when steam dome pressure is greater than 100 psig. Because ADS is not required to be operable in Operational Condition 4 or in Operational Condition 3 when the steam dome pressure is below 100 psig, the RHR Low Pressure Coolant Injection (LPCI) Mode Pump Discharge Pressure - High permissive for ADS is also not required to be operable. The improperly credited surveillance requirement was completed on July 21, 1995, prior to changing to Operational Condition 3 and reaching a steam dome pressure of 100 psig. Therefore, the improper credit of performed testing did not result in a violation of the HCGS TS Surveillance Requirements.

The cause of these events is procedural inadequacy and personnel error. Maintenance supervisory personnel together with the faulty functional test procedures improperly allowed crediting of a multi-channel functional test based on performance of a single channel calibration.

3b. Corrective Steps that Have Been Taken and Results Achieved

- a. As an interim corrective action, until procedural errors were corrected, a memo was immediately sent to all I&C Supervisors identifying the twelve functional tests which cannot be credited by completion of a single calibration.
- b. The HPCI functional test that was improperly credited on June 5, 1995 was satisfactorily performed on July 21, 1995 prior to reactor startup.
- c. All channel calibration procedures have been revised to include explicit directions regarding the crediting of the functional test based on performance of the channel calibration.
- d. For multiple channel functional tests, the planning and scheduling organization has added a precautionary note to the associated recurring task work orders to indicate that channel calibrations of multiple channels are required to credit the completion of the functional test.

4b. Corrective Steps that Will Be Taken to Avoid Further Violations

- a. Problems identified during implementation of the TSSIP will be appropriately communicated to the responsible department so that corrective actions can be taken. These corrective actions will prevent further violations. The TSSIP will be completed by 12/31/96.

5b. Date When Full Compliance Will Be Achieved

Full compliance was achieved upon proper completion of:

- 1) Surveillance requirement 4.3.3.1-1.3.c, functional testing of the HPCI Condensate Storage Tank (CST) low level suction transfer, on July 21, 1995, and
- 2) Surveillance requirement 4.3.3.1-1.4.e, functional testing of the RHR "A" and "C" Pump Discharge Pressure ADS Permissive, on December 12, 1991 and June 16, 1993.