



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated November 27, 1991, as supplemented February 12, March 6, and March 10, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

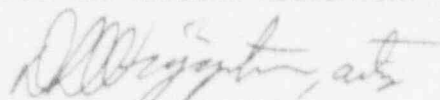
2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 143, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John T. Larkins, Director
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 2, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 143

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise Appendix "A" Technical Specifications as indicated below. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

REMOVE PAGE

2-50

INSERT PAGE

2-50

2.0 LIMITING CONDITIONS FOR OPERATION

2.10 Reactor Core (Continued)

2.10.2 Reactivity Control Systems and Core Physics Parameters Limits

Applicability

Applies to operation of control element assemblies and monitoring of selected core parameters whenever the reactor is in cold or hot shutdown, hot standby, or power operation conditions.

Objective

To ensure (1) adequate shutdown margin following a reactor trip, (2) the MTC is within the limits of the safety analysis, and (3) control element assembly operation is within the limits of the setpoint and safety analysis.

Specification

- (1) Shutdown Margin With $T_{\text{cold}} > 210^{\circ}\text{F}$

Whenever the reactor is in hot shutdown, hot standby or power operation conditions, the shutdown margin shall be $\geq 4.0\% \Delta k/k$. With the shutdown margin $< 4.0\% \Delta k/k$, initiate and continue boration until the required shutdown margin is achieved.

- (2) Shutdown Margin With $T_{\text{cold}} \leq 210^{\circ}\text{F}$

Whenever the reactor is in cold shutdown conditions, the shutdown margin shall be $\geq 3.0\% \Delta l/k$. With the shutdown margin $< 3.0\% \Delta l/k$, initiate and continue boration until the required shutdown margin is achieved.

- (3) Moderator Temperature Coefficient

The moderator temperature coefficient (MTC) shall be:

- Less positive than $+0.2 \times 10^{-4} \Delta \rho/^{\circ}\text{F}$ including uncertainties for power levels at or above 80% of rated power.
- Less positive than $+0.5 \times 10^{-4} \Delta \rho/^{\circ}\text{F}$ including uncertainties for power levels below 80% of rated power.
- More positive than $-3.0 \times 10^{-4} \Delta \rho/^{\circ}\text{F}$ including uncertainties at rated power.

With the moderator temperature coefficient confirmed outside any one of above limits, change reactivity control parameters to bring the extrapolated MTC value within the above limits within 3 hours or be in at least hot shutdown within 6 hours.