#### AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311
Unit Name Salem # 2
Date August 10,1984
Telephone609-935-6000
Extension 4455

Month _	July 1984					
Day Avera	ge Daily Power L e-NET)	evel	Day	Average Daily (MWe-NET)	Power	Level
1 9	95		17	0		
2	98		18	0		
3	75		19	0		
4	99		20	0		
5 _ 3	94		21	0		
6	0		22	0		
7	0		23	0		
8	0		24	156		
9	0		25	447		
10	0		26	0		
11.	0		27	0		
12	0		28	0		
13	0		29	0		
14	0		30	0		
15	0		31	0		
16	0					
Pg. 8,1-7	R1					

8408200276 840731 PDR ADOCK 05000311 R PDR JE24

# OPERATING DATA REPORT

			Telep	hone	ugust 10,1984 935-6000
Comp	pleted by L. K. Miller		Exten	sion	4455
Open	rating Status				
1. 2. 3. 4. 5. 6. 7. 8.	Reporting Period Licensed Thermal Power (MWt)	s MWe) $\frac{11}{110}$	11 62 15 49		since Last
9.	Power Level to Which Restricted,	if any	(Net MWe) _	1	N/A
10.	Reasons for Restrictions, if any		N/	'A	
		This Mon	nth Year	to Date	e <u>Cumulative</u>
12. 13. 14. 15. 16. 17. 18. 19. 20. 21. 22.	Hours in Reporting Period No. of Hrs. Reactor was Critical Reactor Reserve Shutdown Hrs. Hours Generator On-Line Unit Reserve Shutdown Hours Gross Thermal Energy Generated (MWH) Gross Elec. Energy Generated (MWH) Net Elec. Energy Generated (MWH) Unit Service Factor Unit Availability Factor Unit Capacity Factor (using MDC Net) Unit Capacity Factor (using DER Net) Unit Forced Outage Rate Shutdowns scheduled over next 6 1 N/A	17.4 17.4 14.6 14.5 82.6	61 20 19	3111 055.1 442.9 899.4 0 43345 38100 04389 37.2 37.2 33.7 33.4 62.8 and du	54.2 54.2 45.0 44.6 36.5
	If shutdown at end of Report Per 8-8-84 Units in Test Status (Prior to Company)				artup:
	Initial Cr Initial Ele Commercial	iticalit	Fore 6/3 9/1	00/80 1/80 24/80	Achieved 8/2/80 6/3/81 10/13/81

### UNIT SHUTDOWN AND POWER REDUCTIONS REPORT MONTH July 1984

Docket No. 50-311 Unit Name Salem No. 2 August 10,1984 Date Telephone 609-935-6000 Extension 4455

Completed by L.K. Miller

No.	Date	Туре	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
84-214	7-5	P	459.5	A	1	-	SF	PIPEXX	Nuclear High Pressure Piping Core Cooling
84-216		P	10.3	В	5	- 101	CG	INSTRU	Feedwater Regulating Boiler Level Control Valve
84-218	7-25	P	154.7	A	3		CJ	VALEX	Power Operated Relief and Safety/ Relief Valves Reactor

F: Forced S: Scheduled 2 Reason A-Equipment Failure-explain B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & Licensing Exam F-Administrative G-Operational Error-explain H-Other-explain

3 Method 1-Manual 2-Manual Scram. 3-Automatic Scram. tion of Data 4-Continuation of Entry Sheets Previous Outage 5-Load Reduction 9-Other

4 Exhibit G Instructions Salem as for Preparafor Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Source

MAJOR PLANT MODIFICATIONS REPORT MONTH July 1984 DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1984
COMPLETED BY: L.K. Miller

TELEPHONE: 609/935-6000 X4455

\*DCR NO. PRINCIPLE SYSTEM

SUBJECT

NONE

MAJOR PLANT MODIFICATIONS REPORT MONTH July 1984 DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1984
COMPLETED BY: L.K. Miller
TELEPHONE: 609/339-4455

DCR NO. 10CFR 50.59

SAFETY EVALUATION

NONE

# PSE&G SALEM GENERATING STATION SAFETY RELATED WORK ORDER LOG

## SALEM UNIT NO. 2

WO NO DEPT	UNIT	EQUIPMENT IDENTIFICATION	
9900132-2 MD	2	100' EL. AIRLOCK DOOR FAILURE DESCRIPTION: CORRECTIVE ACTION:	BROKEN ROLLER ON INNER DOOR REPLACED BEARING
9900127-6 MD	2	#24 SW TRAVELING SCREEN FAILURE DESCRIPTION: CORRECTIVE ACTION:	REMOVE #24 SW SCREEN AND REPLACE WITH SPARE REPLACED SCREEN
922427 OD	2	#24 RHR SUMP FAILURE DESCRIPTION: CORRECTIVE ACTION:	SUMP PUMP DOES NOT AUTO START CALIBRATED AND INSTALLED NEW LEVEL DEVICE
942982 OD	2	#22 RHR SUMP FAILURE DESCRIPTION:	OHA SUMP OVERFLOW WAS ILLUMINATED AND SUMP HAD BEEN PUMPED WITH A PORTABLE PUMP
		CORRECTIVE ACTION:	INSTALLED NEW PUMP AND NEW MAGNETROL UNIT

### SALEM UNIT NO. 2 OPERATIONS SUMMARY REPORT JULY 1984

Unit No. 2 began the month operating at full power. It remained at full power until July 5 when a controlled shutdown to Mode 5 was commenced at 0822 hours to effect repairs to a Charging Pump suction line which had developed a leak. As the charging pump suction line was being repaired, the three Pressurizer Safety Valves were replaced with the Unit 1 Pressurizer Safety Valves. The valves were replaced due to evidence of leakage causing the high airborne activity in Containment. Wyle Lab's inspection results identified that cracks had formed on the eductor of each of the three valves. Subsequently, the Pressurizer Safety Relief Valves from Unit 1 (that were placed in Unit 2) have also been sent to Wyle Laboratories for inspection. The original three (3) Pressurizer Safety Valves were installed and tested on July 21, 1984 upon receipt from Wyle Laboratories. Plant heatup commenced on July 22, 1984 at 1548 hours. On July 24, 1984 at 0609 hours the reactor was critical and at 1311 hours the Unit was on line. On July 25, 1984 at 1318 hours with the Unit at 66% power the Unit experienced a Reactor Trip/Safety Injection. The Pressurizer PORV stop valve, 2PR6, was opened at completion of the test of the POPS system. When the valve was opened, the reactor began experiencing a sudden pressure drop. The operator immediately initiated a close signal to valve 2PR6. When reactor pressure dropped to 1865 psig the Reactor tripped and Safety Injection occurred at 1765 psig due to valve 2PR6 not closing for 4.5 minutes. The apparent flow path for the pressure drop was through valve 2PR47. The reactor was brought to Mode 5 pending the results of investigations and repair and testing of applicable systems/components. The Unit remained in Mode 5 for the remainder of the period.

REFUELING INFORMATION
DOCKET NO.: 50-311

COMP	LETED	BY:	L.K.	Miller	UNIT NAME: DATE: TELEPHONE: EXTENSION:	Salem 2 August 10, 1984 609/935-6000 4455
Mont	h	July	1984			
1.	Refue	eling	info	rmation has chan	ged from last mo	onth:
2.	Sched	duled	date	for next refuel	ing: January 5	, 1985
3.	Sched	duled	date	for restart fol	lowing refueling	g: March 17, 1985
4.	A)			nical Specificat s be required? YES NOT DETERMINED	NO	other license 8/1/84
	В)					
5.	Sched	duled	date	(s) for submitti December 1984 (		ensing action:
6.	Impo	rtant	lice	nsing considerat	ions associated	with refueling:
	=					
7.	A) B)	Inco	re	Assemblies: Fuel Storage		193 72
8.	Prese	ent 1	icense	ed spent fuel st	orage capacity:	1170
	Futur	re sp	ent f	uel storage capa	city:	1170
9.	to s	pent		efueling that ca pool assuming th ity:		March 2004

8-1-7.R4



Public Service Electric and Gas Company P.O. Box E. Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 10, 1984

Director, Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT, REVISION SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of July 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zughra

J. M. Zupko, Jr.

General Manager - Salem Operations

LKM: sbh

cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management Information and Program Control U.S. Nuclear Regulatory Commission Washington, DC 20555

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