



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 24, 1992

Docket No. 50-327

Mr. Dan A. Nauman
Senior Vice President, Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. Nauman:

SUBJECT: TEMPORARY RELIEF REQUEST FROM AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) CODE FOR WELD LEAKAGE ON COMPONENT COOLING SYSTEM (CCS) HEAT EXCHANGERS OB2 AND 2A1, SEQUOYAH NUCLEAR PLANT, UNIT 1 (TAC NO. 82170)

By letter dated December 2, 1991, the Tennessee Valley Authority (TVA), licensee for the Sequoyah Nuclear Plant, informed the NRC of actions taken to address leakage from two socket weld joints located in ASME Code Class 3 piping. The leakage was from 3/4-inch piping leading to a thermal relief valve on the shell side (Essential Raw Water Cooling System) of the OB2 and 2A1 heat exchangers for Unit 1. The actions taken were of a temporary nature to reduce leakage until repairs could be performed that conformed with the ASME code. The relief request was generated under 10 CFR 50.55a(g)(5)(iii) following discussions with the NRC to address the need for structural integrity until the Code Class repairs could be completed. Unit 1 was in the shutdown condition for a scheduled refueling outage.

In the letter of December 2, 1991, the licensee committed to perform the repairs in accordance with the ASME Code prior to startup from the refueling outage. The licensee has informed the NRC staff that these repairs have been completed in accordance with the ASME Code. Therefore, the relief request is no longer needed. This action closes TAC No. 82170.

Sincerely,

David E. LaBarge, Senior Project Manager
Project Directorate II-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

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