



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
April 2, 1992

Docket No. 50-336

Mr. John F. Opeka  
Executive Vice President, Nuclear  
Connecticut Yankee Atomic Power Company  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: REQUEST FOR RELIEF FROM ASME CODE SECTION XI REQUIREMENTS  
FOR MILLSTONE 2 (TAC NO. M80650)

The enclosed Safety Evaluation (SE) addresses relief from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (Code) which had been determined to be impractical to perform at the Millstone Nuclear Power Station, Unit 2 during the Second Ten-Year Inservice Inspection (ISI) Interval. Your request for relief from the Code-required surface examination for certain examination category B-J welds (Relief No RR-10), and for relief from the Code requirement to examine 21 welds beyond the last normally closed valve in the open-ended portion of the 8-inch diameter containment spray piping system (Relief No. RR-11) was submitted by letter dated June 14, 1991.

Based on the review summarized in the enclosed SE, the staff concludes that, with regard to Relief No. RR-10, the proposed alternative full volumetric examination of the weld and heat affected zone instead of the required inner 1/3 should provide reasonable assurance of the continued structural integrity of certain category B-J welds of the reactor coolant system. There is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner. Therefore, in accordance with 10 CFR 50.55a(g)(6)(i), we find the relief requests may be granted.

With regard to Relief RR-11, the use of approved later editions and addenda of the Code are acceptable alternatives to the required 100% surface examination of Class 2 piping welds in the Containment Spray System and provides a reasonable level of quality and safety. Therefore, pursuant to 10 CFR 50.55a(a)(3)(i), the alternative examinations are approved.

The relief (Relief No. RR-10) pursuant to 10 CFR 50.55a(g)(6)(i) and the approval of alternative examination of Class 2 piping welds (Relief No. RR-11) pursuant to 10 CFR 50.55a(a)(3)(i) are authorized by law and will not endanger

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Mr. John F. Opeka

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life or property or the common defense and security and are otherwise in the public interest giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility.

Sincerely,

/s/

John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
Safety Evaluation

cc w/enclosure:  
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