

## NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20666

Apirl 2, 1992

MEMORANDUM TO: Robert C. Pierson, Director

Advanced Reactors Project Directorate

Division of Advanced Reactors

FROM:

Scott Newberry, Chief

Instrumentation and Control Systems Branch

Division of Systems Technology

SUBJECT:

SICH COMMENTS ON THE GE ABOR DRAFT ITAACS

Enclosed is a set of comments on the draft ITAACs that are being developed on I&C topics. We request that these comments be forwarded to GE for their consideration in the organing development of the ITAACs. We have had several telephone conversations and meetings with GE on these topics and expect several more before complete resolution of these issues. Because the resolution of these issues will be documented in the final ITAACs we do not request a formal response from GE to this set of comments. As GE provides additional ITAACs for NRC review, we expect that we will have additional comments. The impact on review schedules as a result of GE's submittal schedule has been discussed with GE and is documented in the GE/NRC monthly meeting minutes. If you have any questions, please call Jim Stewart (304-2824). These comments do not contain any GE proprietary information.

Scott Newberry, Chief

Instrumentation and Control Systems Branch

Division of Systems Technology

Enclosure: SICB TTAAC Comments

cc: w/enclosure:

A. Thadani

C. Poslusney

T. Boyce

C. Goodman

C920409004Dame

#### SICB ITAAC COMMENTS 1/16/92

These comments are provided to document past converse lons and facilitate future discussions on the development of the ITAACs that are related to the ABWR I&C design.

In general, we agree with GE that those items which are usually referred to as Appendix B type items (traditional quality inspection of the art of the craft such as soldering, welding, painting, wire lug crimes, etc) do not need to be described in the ITAACs. These items will still be inspected under Part 50. Any items which we believe need to be clarified now to minimize disagreements later (softwaru, emi, mutiplexing, etc.) will be recommended for ITAACs.

We have not identified any IFC specific topics that should be included as interface requirements. There will be some interface systems which will have the expected IEC portion included. We believe that these are system specific and will have no significant differences from the IEC systems available for review now.

There is still a need to describe the process which applies to the modification of the tier 2 commutments.

#### PROTUTYPING

These comments relate only to the I&C aspects of the design. The I&C input to the control room man-machine interface design/tests/prototypes should be included as part of that issue.

The tarm "prototype" has been used in several different ways. Using the October 21, 1992 version of the "Scroll" as a reference point we identified several points that have at times been referred to by various parties, including the NRC, as prototypes. There are probably other points than the ones listed below that could be referred to as prototype points but we think this will address the issue. The first, which on the original I&C Scroll occurred prior to design certification, is a demonstration of the basic technology or an example of a possible implamentation. An example of this is the GE NUMAC line as a demonstration that microprocessors can be used to perform a function like neutron monitoring. We have taken this into consideration and will mention it in the final SER. We do not believe that there is any need to put any acceptance criteria into the ITAACs for this part. The design process description, which includes the Scroll, should be adequate. The October 24, 1992 Scroll has removed the NIMAC references. We believe that there should be a block prior to the design pertification line that includes a demonstration of the basic technologies to be employed.

The second place that is identified would be during the selection of the equipment vendors. There may be some prototyping to demonstrate capabilities or compatibility with the rest of the design. We do not believe that this would need to be included as an item in the ITAACs.

The third place (probably many places) would include the development prototyping. For example, this may include breadboarding, testing specific components and testing different software languages to implement the neutron monitoring algorithms. We do not believe that this type of prototyping needs to be described in the ITAACs.

The fourth point could be either the prototype development and testing blocks identified on the scroll or the point identified on the scroll as "perform hardware/software integration testing" (block 54 on the scroll). The staff expects that this is an appropriate point to audit and will be used as part of our safety determination and, therefore, needs to be included in the ITAACs. The staff is flexible on whether this goes into system or generic PTAAC. The items that should be included include: the development of the test and evaluation procedures as the design commitment, inspections to verify that the procedures are being used correctly as the inspection activity, and the completion of the testing, documentation of the results, and the feedback of the results to the dosign process as the acceptance criteria. We believe that this confirmation point would serve two primary purposes. The first would be a verification that the design process is being implemented as certified and that the previous requirements have been met. The second would be to establish the more detailed acceptance criteria for the step described below. We believe that this step should have, as a minimum, one full channel of the SSLC using hardware and software similar to the intended final product.

The fifth point (not usually referred to as prototype) would be a final factory acceptance test (block 64 on the scroll). We believe that this should be in the TTAAC and should be similar to the step above except that the test will be with the final software and hardware which is to be installed in the plant and would include all channels. The acceptance criteria could be similar to the general criteria.

The last point (also not prototype) is the final inspection (block 73 on the scroll) prior to fuel load as installed in the plant. This will be in the ITALC. We consider that this is part of the original construction ITALC intent and is not a new suggestion. The acceptance criteria would be a successful demonstration that all the original criteria in the design certification have been met.

Comments on the GENERIC SOFTWARE ITAAC:

In general, we are not distinguishing between the design acceptance criteria and the original confirmatory inspection criteria.

In referring to item 1 of the generic software ITAAC(1/17/92), the plans themselves should be included as part of an early hold/conformance point described in the scroll. The description of the SMP and V&V plans should be expanded similar to the CMP that we have previously discussed.

Under item 1 add Software Safety Analysis Plan, Timing analysis, and! Hardware/Software Integration Plan. Add "safety" to the list of attributes for the acceptance criteria. For item 2, add Software design safety/hazard analysis.

Delete item number three referring down to tier two.

The plans should specifically describe a method for maintaining testing statistics (metrics) for later use.

In general, the approximate "level of detail" for all of the topics should be equivalent to the configuration management plan.

In addition to the items mentioned above we believe that it will be necessary to describe the selection process for the software design methodology. The criteria for selection of a formal method, object oriented design or particular languages should be described in the ITAAC. This section may be a place for including the requirement that the design method and the tools to be used are proven technology. If formal methods are to be used, the symbol definitions, syntax, semantics, objects, etc must be documented and specifically included (perhaps by reference) in all of the design and configuration control documents.

#### ADDITIONAL COMMENTS

We have received additional comments on the configuration management plan from our contractors and other NRC staff members. We believe that the following items should be added to the CMP part of the software ITAAC. The comments are referenced to the version that was submitted by GE on January 17, 1992.

Item 2 of the CMF should use the definition of IEEE 1042-1987, section 2.1.3 for executable/non-executable software.

Describe how subcontractors are to be included in the CMP.

Add an additional item 11 which puts the definition and control process documents for walkthroughs/inspections/audits/reviews, SQA, V&V plane, design procedures, code format standards, etc., under configuration management control.

We suggest adding an item which requires the definition and control of the various baselines (functional baseline, allocated baseline, developmental configuration baseline, product baseline).

For item 5, add a specific requirement for physical and functional configuration audits/reviews. (Ref IEEE 1042-1987, App. A, 3.4)

Related to the ITAAC process we are providing comments on the GE NUMAC Software Configuration Management Plan.

The staff did a short review of the NUMAC SCMP (Rev 1, March 12,1991) using the draft ITAAC CMF, IEEE 838-1983, and IEEE 1042-1987. The intent of this effort was to simulate an NRC audit of the SCMP as it may occur during the performance of the ITAACs when the final vendors are selected. This is meant as an example only and due to its brevity is not intended as a final

conclusion as would be found in a safety evaluation report. Several items discussed in IEEE 828 may also be discussed in GE software management plans and QA documents which we did not review for this task. In addition, the IEEE standard includes some items which may not be or regulatory concern.

In general, the SCMP appears to provide reasonable guidance to the GENE staff for managing software configuration with a few exceptions. The SCMP follows the recommended format of IFEE 828 except in the area of managing interface requirements (IEEE 828 section 3.0). The SCMP should either include this section, provide justification for the omission or refer to specific product interface requirements in another document.

The introduction and scope are general, as expected for a product line document. The responsible organizations would have to be specifically identified for a specific SCMP.

"FDI" should be included in the list of definitions and acronyms. Section 2.3 of the SCMP does not mention configuration management of related software development, test, and support tools.

Though not required, several items from IEEE 828 should be addressed in section 2.4 of the NUMAC plan. These include the document release process, the structure of the configuration control board, the release and acceptance of software products, SCM audits, problem reporting, testing prior to CM, and QA prior to CM.

The control procedures for associated special software products, such as nonreleased software, off-the-shelf software, user-furnished software,; and in-house support software was not discussed. The control of tools to manage the software configuration appears to be ambiguous.

The control of subcontractors needs to be expanded. The method to determine the SCM capability of the subcontractor and the method to monitor their performance is not discussed in the SCMP.

Physical safeguards were not discussed. Off-site backup was not discussed. Retention periods were not discussed.

In conclusion, it appears that a few revisions or references to other documents would be needed to demonstrate that the GENE SCMP is in complete conformance with IEEE 828.

#### SSIC ITAAC

Though we have not seen the ESIC ITAAC yet, there are a few items that should be included which we wish to identify at this time. Some of these may apply to other ITAACs in addition to SSIC or may be in the generic ITAACs.

1. The testing of the SSLC at the prototype confirmation points should include loss of power. This includes both short (a second or less), intermediate (a few minut a), and long (8 hours) interruptions in power. The acceptance criteria would be that the failure and recovery states of the SSLC

output is achieved. An analysis and confirmatory test should be included for the recovery situation where, after reinitialization, the input data has changed enough to require the final actuated device to change position. The rate of change to the new position (or block and alarm) and information to the operator should be considered.

#### EMI/SWC ITAAC

Though we have not seen this ITAAC draft yet we have a few comments to offer fir discussion. We believe that this ITAAC has four primary functions.

- 1. The equipment should be designed with EMI/SWC considerations in mind. The purchase specifications should specify the minimum levels that the equipment should be designed to withstand and the maximum levels that are acceptable for emissions. Because the methods used are dependent on the equipment and there are different methods to achieve a successful design, we believe that this is appropriate for ITAACs.
- ?. The equipment should be tested in a controlled environment by a lab or at the factory to establish the tested envelope. The key features of your 461/462 commitments could be used for this part of the ITAAC.
- 3. The equipment should be installed using shielding and grounding techniques appropriate for the equipment. The GE commitment to IEEE 1050 can be used to extract the key attributes. Because this is a lengthy standard, we will attempt to work with GE to keep the ITAAC to a reasonable length.
- 4. The FMI fields should be tested with the equipment installed in the plant. The site testing will include radiated and conducted emissions and will be measured over a period of time. The analysis section for this step should verify that the tested environment is enveloped by the factory testing.

#### SETPOINT METHODOLOGY

The setpoint methodology ITAAC should be modeled on ISA -867.04, buch part 1 and part 2. J.L Leong is listed as the GE representative for the standard committee.

In addition to the ISA standard we suggest the additional 4 items:

- 1) Guidance should be provided regarding the use of the SRSS in cascading situations such as multiple instrument channels or multiple modules in an instrument loop. For example, if the SRSS has been performed on each of three channels inputting to a multiple module, can the output of that module have an SRSS performed on the error components that have already undergone an SRSS. The 95/95 rule should take precedence in this matter.
- 2) In section 8.1, additional caution should be expressed in the use of the single sided setpoint error calculation. This should be used as a last resort to avoid going into the process area and, above all, the 95/95 rule must be maintained.

- 3) In section 6.2.6.2, further discussion should be provided regarding the use of a calibration tolerance. The upper calibration tolerance band should leave enough margin between it and the allowable value to allow for the normal uncertainty between the setpoint and the allowable value.
- 4) As before, section 6.1 should contain further discussion on the merits of 95% probability and 95% confidence level. This discussion would help in alleviating setpoint problems that will be encountered in the future.



# GE Nuclear Energy

ICE C	ABWR
TO GLEN KELLY NRC	Date
This page plus 21 page(s)  From ACC DUN CON  Phone (408) 925— 6.947	Mail Code 754 175 Curtner Avenue San Jose, CA 95125 FAX (408) 925-1193 -
	FAX (408) 925-1193 or (408) 925-1687  plana multi-157
Message An updated punch Pleace provide agres " unc contacts" and of	to all the
This revision refle results of ar Man 915205510000 18 CENS by CENDL Buchke	- 50 missel

page in recent discussino with you, Visha end Raftery The mon-DSCR I tams in the OSER Item Column are now munhored for easter reterence (eg n/a in page 1) Other agaitins, charges of note I-1 15 2 plicant acting: Resolver o RWCU so Offer success added 2-4 updated to show 2% on exacting assumption part insights adod after 0-22 Human Factors silters updates to reflect 3/28 discussions (-6 sime 0-184 moteral sulmitted 4/2/92 Site 2 cceptability ( under consequence analysis) accepted by Bob Palla

page isi

If you or other " NAC antacts" need to esiscuss those stone, they should call the " 45" contact"

Please note that in many Cleses the next actor is unc. We und like to have estimates to the unknum date within the most beek.

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NRC Contact 1 n e Next Date Action

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item Section	- Subtask	CE Contact	6	l i				
Fault Tree Update								
0-1	RPS Reliability	Xelly Raftery	٧	*	GΣ	4/15	Provide RPS (Lult tree to NRC thru Fox	
niz	Update other fault trees	Reily Raftery	7	*	Œ	4/30	Provide balance of fault trees to NRC	Some additional functions to be automated
Initiating Evente								
\$-2	Defend IORV frequency	Kelly Raftery	*	B	GE		Use 0.1 is requantification	Staff requires use 6.1 event/year. GE accepts - Isean Resolved
					CE	The	2. Reflect 0.1 in SSAR table and text	
C-1	IORV success criteria	Kelly Raftery			Œ	TBD		NGC clarification sent 4/1.
5-1 n	Defend one trip per year	Kelly Duncan	*	20 1			none	NRC now agrees 1 trip/year as reasonable - Item Resolved

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4/3/92

Comments

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DSER SSAR item Section	Title - Sobtask	NRC Contact GE Londred	1 2 22	20 21	* * * * *	Next	Date	Action	Comments
02	Evaluate support system failure on plant trips	Kelly Raftery	*			NRG	4/16 4/1/64	Review draft material	GE has provided (Jan. 27) draft material (HVAC failure) and list of other support system to be addressed. GE agreed to stell request to add has of single AC or single DC bus
0.5	Analyze interfacing LOCA	Kelly Visweswaren	e			Œ		Document piping upgrade. Teack Separately	
04	Outside containment LOCA	Kelly Frederick				NRC	4/B	Review SSAR	GF request NRC review SSAR 19F 23.3. Issue is addressed
1-1	Confirm site loss of AC, other site specific parameters	Kelly Raftery		٠		В		Applicant action Item Resolved	

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Success Cariteria							
CS	ATWS Success Criteria to prevent core damage	Kelly Buchaolz	 •	NRC	?	Review submittal	Agreed SSAR treatment is adequate at San Jose 1/92 meeting. Documentation set-mitted 4/2/32
n3a	ATWS RHR Success Criteria	Keliy Buchholz	 • 1	NRC	2	Review submittal	Agreed SSA2 treatment is adequate at San Jose 1/92 meeting. Documentation submitted 4/2/92
n4a	RWCU as Dift? Success (ACRS)	Kelly Paradiso		Œ	4/6	Calrify with NRC X (.953	Cally Parodiso
Faihure Data							
C-3	Justify common cause lailure data	Xelly Frederick				See item 0-5	

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DSER SSAR item Section	Title - Subtask	NRC Contact GE Contact	P I a n	A n a I	Review	Next Actor	Date	Action	Comments	
0.5	Justify train-level common cause approach is adequate	Kelly Raftery				Œ	3/24	Discuss which components to include with NRG	For item C-3 and 0-5, NRC suggested requantify PKA without addressing CCF, then requantify CCF as sensitivity study as part of input to RAP (Item 1- 15)	
€4	justify test/maintenance data analysis	Kelly Raftery				Œ		Include new values in requantification: test and maintenance unavailability	Action plan discussed with NRC 5/24: T/M unavailabilities will be increased for utility flexibility. ECCS, RCIC RHP unavailability will be taken as 2%. Item Resolved	
<>6	Justify RIfR, HPCF pump failure probability	Kelly Raftery				Œ	4/15	Provide justification		
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		ABWR Tasks to wrap up PRA										
	SSAR Section	Title - Subtask	NRC Contact GE Contact	Plan	A	200	Next Actor	Date	Action	Comments		
O-21A		List systems not part of certified design, significant assumptions, assumed reliability	Palla Duncan									
Level 1	PRA and	Subsequent Related Stud	See									
S-11	e.	Fire water credit correction	Kelly Visweswaren				n	В	GE approach conservative. Fire water will be in RAP. No other action needed -liem Resolved			
re Gips		Requantify level 1	Kelly				Œ	4/15	Tabular results to NRC:			

O-18A Level I uncertainty

Kelly Raftery

Raftery

CF AT 5/15

6/30

Sebmit uncertainty anaiyess

CDF by initiator and

accident class. Will not include item 0-2 (supports system trips)

SSAR text deaft to NRC

Results stop at CDF, will not be propagated through anaiysis

OHI

Initiating Event uncertainty

PRA results (base

case)

amalysis.

Kelly

Address as part of item O-18A

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4/3/92

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1-15	PRA input to Reliability Assurance	Kelly Raftery	٠		Œ		Define icput	General agreement reached on ow to proceed
n72	PRA input to ITAAC	n Duncan		žnej:				
0-22	PRA as a design took	Eelly Puncas	*	8	CE	4/15	Provide 90% complete deaft	
n7b	PRA Insights	Kelly Duncan			NRC	,	Outline of management closure. June 25 staff deadline	New item, added in 3/23 seceting. Narrative relating how PRA was used, including how several DSER items were addressed

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DSER SSAR item Section	Title - Subtask	NRC Contact GE Contact	P 1 2 0	83	8 8 8 W	Next Actor	Dave	Action	Comments	
0-20	Shutdown risk. Four subtasks:	Kelly? Visweswaren	¢			GE	3/24	Further discussion		
	ABWR design features					GE	4/30			
	2. Review of past abatdown events					CE	4/30			
	3. Decay heat removal rehability					Œ	6/30			
	4. Other tasks					Œ	5/30			
Other Analyses										
O-i2	Fire PRA	Kelly Raftery	c	¢	*	NRC	4/15	Provide results of review to GE	No major items expected	
1.9	Internal Flood	Kelly Visweswaren	8						Approach	
	Four subtasks:	***************************************							outlined 3/24 San Jose	
	1) Turbine Bldg	Ehlert								
	2) Control Bldg	Miller				1				
	3) Reactor Bldg	Mazwell Eblert								
	4) Evens Trees	Cokcek								
C × Confirmato	ry item		-			e = stayte	ed			
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DSER	Section	Title - Subtask	NRC Contact GE Contact	P	A n z l	K v i e w	Next Actor	Date	Action	Comments
Fivenan	Factors is	PRA								
C-5 O-7 O-8 O-9 O-10 I-2 I-3 I-4 I-5 I-6 I-7		These items are addressed by the following job subtasks:								Plan (Subtasks A,B, C, D) based on NRC/GE phone call 3 Mr./ch 92 and discussions 23 March.
		A) Provide a list of references/general methods used to determine all failure probabilities. Reference to GESSAR PRA is acceptable.	Beckaer Frederick	٠			CE			
		B) Conduct sensitivity study	Kelly Raftery	•			GE.	6/1	Sensitivity study results	Increase sensitivity range if hard to document source of number
	onfirmator						= staste	ed		

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Comments

C) Develop Kelly descriptions for most Dencan significant actions for input to control room design ie, provide time available for operator action & signals that the operator relies on. Provides input to control room design & EPGS.

D) Provide more specific references for operator actions identified as significant based on sensitivity study. References should include table and page numbers in references.

Asternatively, use a wider range in performing the sensitivity study.

Kelly v GE

Preliminary judgement of significant seems documented in 18F.Z.

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#### Containment Event Trees

GE will take credit for 84 CF Stoff extremates extra Kelly went in Level 1 credit for vent in Buschhody requantification Class E interval event and scamic requestions 0.13 Treatment of drywell -wetwell bypass in CFTs. This item is addressed by two cab taske-Tracked Kndrick A) Design basis separately by sensitivity studies Sagena Kudrick/Sazena Sec O-18. Palla/ B) Determine if Rackend CETs need to change Kudrick Uncertainty Buchholz Analysis See item O-17A 0-178 Modify CET for Palla under Backend severe accident phen. Buchholz

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Analysis

	SSAR Section	Title - Subtask	NRC Contact CE Contact	3	e y	Date	Action	Comments
					160			

Beckend Analyses								
C-6	Flashing during venting	P:::a Bachholz	£4		NRC	?	Review submittal	Analysis documented in 4/2/92 submittal.
0-14	Justify aspects of OPPD (Rupture Disk Sequint)	Palla Buchholz	c	*	Œ		Provide documentation indicating results of sensitivity and uncertainty analyses in regard to the OPPD and giving the basis for the setpoint.	GE agreed to raise rupture disk setpoint
O-15	Net risk of passive florder system	Palla Buchkotz	c		Œ	4/30	Provide documentation indicating results of sensitivity and uncertainty analyses in regard to the passive flouder.	
O-16	CCI- coolability	_	-			-		

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DSER item	SSAR Section	Title - Subtask	NRC Contact CE Contact	i a n	A n a	R e v i e w	Next Actor	Date	Action	Comments
		- Containment integrity	Palla Buchholz	•	3		Œ	4/16	Take credit for assumed factor of safety in calculation. Submit to NRC.	CE has shown staff early results of bounding analysis - work not complete but results appear to support no containment failure
		- Source terms	Palla Buchholz	¢		-	+			See uncertainty analysis
O- 17A		Modely CETs for DCH, FCI, etc.	Palla Buchholz							DCH and FCI modes are included as early containment failure modes in the current event trees. OCI and Pool bypast will be added if the uncertainty analysis warrants it.
0-18		Uncertainty analysis		- Committee	-	-	-			

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item Section	Title - Subtask	NRC Contact GE Contact	1 20	13	e se	Next Actor	Dr.te	Action	Comments
	- Survey Merature	Palla Buchhok	•	c	*	NRC	?	Review submittal	Discussed at 1/22 meeting in Bethesda. Documented in 4/2/92 submittal.
	Screen events for applicability to ABWR	Palla Buchhob	¢	ε	٠	NRC	2	Review submittal	Discussed at 1/22 meeting in Bethesia. Documented in 4/2/92 submittal.
	sensitivity studies	Palla Be: hholz	*	*		Œ	4/10	Complete sensitivity studies and document (some will be missing)	List of sensitivity studies discussed at 1/22 meeting in Bethesda. About 50% of studies complete.
	Detailed DCH Uncertainty analysis	Palia Buchholz	c	۳	*	Œ	4/16	Incorporate NRC comments. Document study and results.	Essentially done. Discussed at 1/22 meeting in Bethesda and in 2/26 telecon. NRC had only minor comments.

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AR DSER SSAR Tiske NRC Contact I m r Next Date Action Commenia item Section - Seebtask CE Contact w Actor Detailed Palla 4/30 Complete study -CF Discussed of OCI/Coolability Base laborate 1/22 meeting in incorporating NRC

5.9

Rupture disk Palla operation before 24 Buchhola hours not cost failure

Uncertainty analysis

23

COMPRESS

NRC agrees rupture disk operation is not containment fadure !tcm Resolved

Betherda and in 2/26 telecon

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4/3/92

DSER SSAR item Sections	Title Subtrak	NRC Contact GE Contact	P # 3 B	Ana	R c v · · · · w	Next Actor	Date	Action	Comments
S-12	ATWS treatment by NRC	Palia Bechholz						n	NRC agrees GE apparach conservative, no other impacts. Item Resolved
n17a Chapter 22	Severe Accident Closure	Kudrick Buckbolz	c	ε		NRC	7	Write section  Inform CF if more info needed beyond that identified in 2/29 submittal.	
n48a	Accident management	Paila Bochholz	2	e	¥	NRC		Provide response to CE submittal	
Seionic Analysis B	leyond Design Base								
8-5	Fuel assembly capacity	Kelly Liu	¥			Œ		Update SSAR	NRC, GE have agreed to 1.2g Item resolved
\$-6	Flat-bottom tank capacity	Ketly Liu	٧			Œ		Margins Analysis	GE will probably reduce value
S-7	Diesel generator capacity	Kelly Liu	٧			CE f		Margins Analysis	GE will probably reduce value
\$-8	Electrical equipment capacity	Kelly Liu	٧			Œ		Margins Analysis	GE will probably reduce value
C = Confirmatory S = Staff correction O = Outstanding	itcan					$\varepsilon = cos$	ussed w	erbally but not documente	

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n = not applicable

NER tem	SSAR Section	Title Subtask	NRC Contact	n n i	A n a l	e v i e w	Next Actor	Elake	Action	Comments
1-14	menterens, mi phenomen	Confirm electrical equipment expacities in site specific PRA	Kelly Duncan	٧			Œ		Masgins analysis	Site specific PRA not required. Capacities used in margins analysis will be imput to ITAAC.
O-21B		Seismi capacities for systems not in certified design	Palla Duncan	٧			Œ		part of margins	
S-19	п	Staff will use LLNL.	Keily Duncan				- 21		25	Seismic PRA not remired. Item
n202	n	How to treat acismic hazard uncertainties Confirm assumed	Kelly Duncan Kelly	-			o. n		TS	Not required.  Item resolved  Probably cover by PRA input to
1-10		seismic capacities and incorporate in design specs	Duncan							Seismic PRA not
1-11		Modify seismic PRA to account for soil failures	Kelly Duncan				- n		19	may address by site licensing requirement. Item Resolved

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4/3/52

#### ABWR Tasks to bemp up PRA

AR Title tion - Subtask	NRC Contact CF. Contact	P l a n	A to a 1	R e v i e w	Next Actor	Date	Action	Comments
equipment capa	cities Duncan	*			Œ		Margins analysis	Site specific PRA not required. Capacities used in margins analysis will be appet to FFAAC.
systems not in	Duncan	٧			Œ		part of margins assessment	
Staff will use I I hazard curve	NL Kelly Duncan			-	n ·		п	Seismic PRA not required. Item resolved
		-	-		D		a	Not required. Item resolved
scismic capaciti	es Duncan	v			Œ			Probably cover by PRA input to ITAAC
					п		n	Seismic PRA not required. NRC may address by site licensing requirement.  Item Resolved
	Confirm electric equipment capaziti systems not in certified design Staff will use I I hazard curve  How to treat se hazard uncerta  Confirm assum seismic capaciti and incorporate design specs  Modify seismic to account for se	Confirm electrical Kelly equipment capacities Duncan is site specific PRA  Seismic capacities for Palla systems not in Duncan certified design  Staff will use LLNL Kelly hazard curve Duncan  Confirm assumed Kelly Duncan  Confirm assumed Kelly science capacities and incorporate in design specs  Modify seismic FRA Kelly to account for soil Duncan	Confirm electrical Kelly vequipment capacities for Palla vertified design  Staff will use I LNL Relly bazard curve Duncan  How to treat seismic Kelly Duncan  Confirm assumed Kelly Duncan  Confirm assumed Kelly Duncan  Confirm assumed Kelly Duncan  Confirm assumed Kelly Duncan  Modify seismic FRA Kelly bo account for soil Duncan	Confirm electrical Relly vegainment capacities for systems not in certified design  Staff will use I LNL Relly bazard curve Duncan  How to treat seismic hazard uncertainties Duncan  Confirm assumed Relly Duncan  Confirm assumed Relly Duncan  Confirm assumed Relly Duncan  Confirm assumed Relly Duncan  Modify seismic FRA Relly Duncan	Conferm electrical Kelly vequipment capacities for Palla vertified design  Staff will use H.N. Kelly Duncan  How to freat seismic Relly Duncan  Confirm assumed Kelly Duncan  Modify seismic FRA Kelly Duncan	Confirm electrical Kelly v GE equipment capacities in site specific PRA  Seismic capacities for Palla v GE systems not in certified design  Staff will use LLNL Kelly Duncan  How to treat seismic Kelly Duncan  Confirm assumed Kelly v GE  seismic capacities Duncan  Confirm assumed Kelly v GE  seismic capacities and incorporate in design specs  Modify seismic FRA Kelly — — — n  to account for soil Duncan	Confirm electrical equipment capacities is site specific PRA  Seismic capacities for systems not in certified design  Staff will use LLNL Relly Duncan  How to freat seismic hazard curve Duncan  Confirm assumed seismic capacities and incorporate in design specs  Modify seismic FRA Relly Duncan  The Next Date Next Date of the Nex	Seisanic capacities for systems not in certified design  Staff will use LLNL hazard curve  How to treat seismic hazard in confirm assumed seismic capacities  Confirm assumed seismic and in certified design  Kelly  Duncan  Kelly  Duncan  Kelly  Duncan  Kelly  CE  Margins analysis  Falla  V  GE  part of margins assessment  n  n  n  n  n  n  n  design specs  Modify seismic FRA  Kelly  Duncan  Confirm assumed seismic capacities and incorporate in design specs  Modify seismic FRA  Kelly  Duncan  N  CE  Margins analysis  FE  Duncan  GE  Date Action  OE  Margins analysis  FE  Duncan  CE  Duncan  Duncan  n  n  n  n  n  n  n  n  n  n  n  design specs

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DSER SSAR	Title - Sestitate	NRG Contact CE Contact	Pi	Anal	現 さ マ 注 ぐ ※	Next	Date	Action	Comments
I-12	Plant walkdown by applicant	Kelly Duncan				79	£3.	\$\tag{2}	Ce agrees to plant walkdown. Per EPRI NP-6041 Item resolved
I-15	Site specific seizmic PRA by applicant	Kelly Duncan				п	n	5	Seismic PRA not required. NRC may address by site licensing requirement. Item Resolved
O-19	Address potential for penetration, isolation valve failure during scissoic event	Palis Knecki	٧			NRC	3	Evaluate proposal. More discussion needed nearing resolution.	Proposed approach provided to NRC Dec. 91, Jan. 92
S-3	Correct the treatment of firewater in Science Class II CET	Kelly Vishu	٧			Œ		Include correction in margins analysis	GE will correct
n Ma	Margins Amalysis	Kelly				NRC	4/5	Clarify rules, describe HCLP calculation	

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P \* DSER page number - no issue number

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4/3/92

A DSER SSAR NRC Contact 2 n e Next Date Action Title Comments - Salvask stem Section GE Contact 9 Actor

1-8

Site specific design Kelly verification: external Duncan

floods, transportation

bazards

NRC 6/92

"Grandchild" of SECY 016 due about lune. Classic design analysis alone may be sufficient.

Setrefores (Ceneral)

112ib

Be sure PRA "requirements" and insights get into "Interface write !!"

Duncan/Fox need discussion.

G = Confirmatory item

5 = Staff correction

O = Outstanding item

1 = Interface requirement

P = DSER page number - no issue number

n = Plot a DSFR item

s = starte of

v = discussed verixily but not documented

t = complete with docomented agreement

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4/3/92

DSER SSAR Title item Section - Smbrask P A R

NRC Consact i n e Next Liete Action
GE Consact 2 e v Actus

Comments

Солвокринос Аналукія

ngia

No NRC feedback on Pails site acceptability Careway from weather standpoint MRC ? Reflect agreement in SER Palia/Careway discussed 4/2. NRC agrees. It m Resolved

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