

NARRATIVE REPORT
UNIT 1

July 8th	0107	Load reduction from rated thermal power to 689 GMWe for weekly turbine test (HNP-1-1053).
July 8th	0320	Turbine testing complete. Returning to rated power.
July 11th	2105	Load reduction to approx. 350 GMWe to investigate problems with 1A circ. water pump.
July 12th	0550	1A circ. water pump problems resolved returning to rated power.
July 12th	0905	Power increase discontinued at 720 GMWe to adjust APRM gains.
July 12th	0925	APRM gains set increasing power to 730 GMWe via recirc flow.
July 12th	0935	Power increase discontinued for P-1 run at 730 GMWe.
July 12th	0947	Increasing to rated power via recirc flow.
July 13th	2130	Load reduction via recirc flow to 500 GMWe for weekly turbine testing and rod pattern adjustment.
July 14th	0105	Turbine testing complete.
July 14th	0233	Rod pattern adjustment complete.
July 14th	0509	Increasing power to 495 GMWe via recirc from a xenon depressed power level of 471 GMWe.
July 14th	0517	Power ascension terminated at 496 GMWe.
July 16th	0830	Process computer inop.

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NARRATIVE REPORT
UNIT 1

July 16th	1613	Process computer restored to service.
July 20th	2040	Reducing power via recirc flow to 649 GMWe. 6 fans tripped on "C" cooling tower, 3 fans tripped on "B" cooling tower.
July 20th	2115	"B" cooling tower fans restarted.
July 20th	2117	"C" cooling tower fans restarted.
July 20th	2321	Increasing power via recirc flow.
July 28th	0008	Load reduction to 720 GMWe for weekly turbine testing.
July 28th	0208	Turbine testing complete, increasing power via recirc flow.
July 28th	0850	Load reduction to 700 GMWe for control rod exercise (HNP-1-3939).
July 28th	0909	Control rod exercise complete, increasing load back to rated.

NARRATIVE REPORT
UNIT 2

July 1st

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Recirc Piping Outage still in
progress.

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HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR July 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-2832	06-27-84	Rebuilt CO ₂ discharge valve 1Z43-F007 in cable spread and computer rooms. Ref. DCR 84-093.
84-2826	06-28-84	Rebuilt CO ₂ discharge valve 1X43-F011 in diesel building. Ref. DCR 84-093.
84-2830	06-27-84	Rebuilt CO ₂ discharge valve 1Z43-F011 in cable spread & computer rooms. Ref. DCR 84-093.
82-7165	05-24-83	Fabricated & installed reinforcement steel in panel pads in control bldg., el. 164'-0". Ref. 80-348.
82-6191	11-29-82	Fire alarm system control bldg., el. 164'-0" pull in and lugged cable Z43-P001-M24. Ref. DCR 80-348.
83-7635	11-22-83	CRD vents and drains. Replaced existing C11-F110 A,B type NP8316 E36E, 125 VDC solenoid valves. Ref. DCR 83-32.
83-6027	08-13-83	Replaced inoperable Barton transmitter with Rosemont 1151-RHR "A" Loop flow. Ref. DCR 83-157.
84-2683	05-22-84	Rebuilt CO ₂ discharge valve 1Z43-F007, F008, F011 in cable spread and computer room. Ref. DCR 84-093.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR July 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
83-4935	01-13-84	Pulled cables RP1A05C28, RP1B04C28, RP2A04C28, RP2B05C28, RP1A05C29, RP1B04C29, RP2A04C29, RP2B05C29 on reactor protection system/scram discharge volume. Ref. DCR 82-220.
83-1519	04-18-83	Install flex hose 2T48-00018 on containment purge & vent and hydrogen inerting system. Ref. DCr 82-076.
83-2877	06-08-83	Overlayed pipe to pipe weld 2B31-IRC-12BR-E4 on recirc loop B, riser E. Ref. DCR 83-083.
82-1986	04-19-82	Replaced existing recirc pump B seal water FCV with new model. Ref. DCR 82-54.
83-4126	08-05-83	Lighting in airlock el. 164'-0". Core drilled 2" hole in SSO kitchen & installed rigid steel sleeve, installed ALS and ground cable, receptables and lights in airlock. Ref. DCR 80-348.
84-1234	07-12-84	Installed air piping and pipe supports for automatic operators on the manually operated valves in RWCU valve nest room for reactor water cleanup system. Ref. DCR 83-86.
84-1690	05-14-84	Changed out H ₂ O ₂ analyzer panels, bottle racks; installed cabinets, circuit breakers, tubing, raceway and cables. Ref. DCR 81-165.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR July 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-1681	05-11-84	Diesel generator 2C air start system. Fabricated and installed tube support, stainless steel and copper tubing. Removed existing pressure switches and installed 2 new pressure switches. Ref. DCR 80-98.
83-2641	06-04-83	Revised reactor bldg. nitrogen inerting system isolation valve actuator orientation. Ref. DCR 83-51.
83-1131	05-27-83	RHR system installed pipe support 2E11-RHR-H719. Ref. DCR 82-76.
83-2438	07-08-83	Re-installed main steam safety/relief valve 2B21-F013L body, topworks and solenoid. Ref. DCR 82-53, DCR 83-25.
84-2021	05-22-84	Removed scram discharge volume level switch level element 2C11-N060C, and investigated level element and SDV tap for moisture. Ref. DCR 82-220.
84-0375	03-27-84	Installed improved design stem/disk assembly on outboard MSIV 2B21-F022 A-D and 2B21-F028A-D. Ref. DCR 83-40.
84-0370	03-28-84	Installed improved design stem/disk assembly on inboard MSIV 2B21-F022 A-D and 2B21-F028 A-D. Ref. 83-46.
84-1929	06-09-84	Filled scram discharge header 2C11 (North and south side) with demin water to help lower radiation exposure rates. Ref. DCR 82-257.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR July 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
84-2420	06-09-83	Sealed target rock solenoid valves Q270-SV-F004, F005, F066, F067. Ref. DCR 81-112.
84-1699	05-23-84	RWCU pump seal water system. Flushed new seal water supply line. Ref. DCR 79-96.
84-1421	04-25-84	Temporary shroud wide range level transmitter. Disconnected the alarm function being supplied by the temporary level transmitter. Reconnected after the ATTS tubing work on racks 2H11-P005 and 2H11-P009 was complete. Ref. DCr 81-139.
84-1351	05-08-84	Containment atmosphere control. Installed conduit and supports 2T48-ADV-F307. Ref. DCR 83-168.
84-0826	05-10-84	Completed internal panel wiring on 2H11-P622, 2H11-P601, and 2H11-P602. Installed amber trip lights on valves 2T48-ADV-F307, F308, F309, F318, F319, F320, F324, and F326. Ref. DCR 83-168.
84-1193	03-29-84	Reactor protection system/scram discharge volume. Calibrated level switches 2C11-LS-N660C,D. Ref. DCR 82-220, Rev. 1.
83-0338	05-09-83	Performed all core drills, and all civil & structural work to construct all pads for ATTS local rack installation. Ref. DCR 81-139.

OPERATING DATA REPORT

DOCKET NO. 50-321
 DATE 08-10-84
 COMPLETED BY: P.J. North
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 07-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	5111	75238
12. Number of Hours Reactor was Critical	744.0	4261.8	53786.9
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	744.0	4131.3	50524.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1641541	9315545	106533573
17. Gross Electrical Energy Generated (MWH)	539410	2963540	34421710
18. Net Electrical Energy Generated (MWH)	514488	2820966	32680329
19. Unit Service Factor	100.0	80.8	67.2
20. Unit Availability Factor	100.0	80.8	67.2
21. Unit Capacity Factor (Using MDC Net)	91.9	73.4	57.7
22. Unit Capacity Factor (Using DER Net)	89.0	71.0	55.9
23. Unit Forced Outage Rate	00.0	17.8	18.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling 10-1-84 10 week duration			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
 DATE: 08-10-84
 COMPLETED BY: P.J. North
 TELEPHONE (912) 367-7851

MONTH 07-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	717	17	710
2	720	18	711
3	720	19	709
4	720	20	697
5	718	21	708
6	719	22	703
7	719	23	701
8	713	24	699
9	715	25	696
10	714	26	693
11	689	27	691
12	622	28	690
13	699	29	690
14	507	30	686
15	611	31	685
16	667		

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-366
 DATE 08-10-84
 COMPLETED BY: P. J. North
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 07-84
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	5111	42864
12. Number of Hours Reactor was Critical	0.0	308.2	27379.4
13. Reactor Reserve Shutdown Hours	0.0	0	0
14. Hours Generator On-Line	0.0	308.2	26096.1
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	0	726912	55943167
17. Gross Electrical Energy Generated (MWH)	0	242640	18414420
18. Net Electrical Energy Generated (MWH)	-3414	215860	17504967
19. Unit Service Factor	0.0	6.0	60.9
20. Unit Availability Factor	0.0	6.0	60.9
21. Unit Capacity Factor (Using MDC Net)	-0.6	5.6	54.6
22. Unit Capacity Factor (Using DER Net)	-0.6	5.4	52.1
23. Unit Forced Outage Rate	0.0	0.0	13.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Recirc piping outage still in progress.			

25. If Shutdown at End of Report Period, Estimated Date of Startup: 8/19/84

26. Units in Test Status (Prior to Commercial Operation):
Forecast
Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366
 DATE: 08-10-84
 COMPLETED BY: P. J. North
 TELEPHONE (912) 367-7851

MONTH 07-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-7	17	-4
2	-6	18	-4
3	-6	19	-4
4	-6	20	-4
5	-5	21	-4
6	-3	22	-5
7	-3	23	-5
8	-4	24	-4
9	-4	25	-5
10	-4	26	-5
11	-4	27	-4
12	-5	28	-6
13	-3	29	-6
14	-4	30	-6
15	-5	31	-6
16	-4		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 8-1-84
 COMPLETED BY Philip J. North
 TELEPHONE 912-367-7851 X-2882

REPORT MONTH July

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
84-49	84-07-01	F	744	A	5	N/A	HA	TURBIN	13th stage buckets were damaged and out for the duration of July.
84-50	84-07-08	S	2.2	B	5	N/A	HA	TURBIN	Load reduction for weekly turbine testing.
84-51	84-07-11	F	8.75	A	5	N/A	HF	PUMPXX	Load reduction to investigate problems with 1A circ water pump.
84-52	84-07-12	S	.33	H	5	N/A	IB	INSTRU	Power increase discontinued to adjust APRM Gains.
84-53	84-07-12	S	.20	H	5	N/A	IF	INSTRU	Power increase discontinued for P-1 run.
84-54	84-07-13	S	3.58	B	5	N/A	HA	TURBIN	Load reduction for weekly turbine testing and rod pattern adjustment.
84-55	84-07-14	S	1.46	H	5	N/A	RB	CONROD	Rod pattern adjustment.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 8-1-84
 COMPLETED BY Philip J. North
 TELEPHONE 912-367-7851 X-2882

REPORT MONTH July

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
84-56	84-07-20	F	.62	A	5	N/A	HF	HTEXCH	Power reduction to compensate for 6 fans tripped on "C" cooling tower, 3 fans tripped on "B" cooling tower.
84-57	84-07-28	S	2.0	B	5	N/A	HA	TURBIN	Load reduction for weekly turbine testing.
84-58	84-07-28	S	.32	B	5	N/A	RB	CONROD	Load reduction for control rod exercise.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(3/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July

DOCKET NO. 50-366
 UNIT NAME Hatch 2
 DATE 8-1-84
 COMPLETED BY Philip J. North
 TELEPHONE 912-367-7851 X-2882

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
84-5	84-07-01	S	744.0	H	2	N/A	CB	PIPEXX	Recirc Pipe Replacement outage.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

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 Exhibit C - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444

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Z *Att. W*



Georgia Power

Edwin I. Hatch Nuclear Plant

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August 2, 1984
GM-84-677

*File From
EW-359*

PLANT E. I. HATCH
NRC Monthly Operating Report

Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC
Monthly Operating Report for Hatch Unit 1, Docket #50-321, and
for Hatch Unit 2, Docket #50-366.

H. C. Nix

H. C. Nix
General Manager

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