NRC Form	n 366					LIC	ENSE	E EVE	NT RI	EPORT	(LER)	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85							
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Inc	pera	ole A	uxil	iar	y Contr	ol Air	Comp	resso	rs										
EVENT DATE (5) LER NUMBER (6)						RE	PORT DAT	TE (7)		OTHER	THER FACILITIES INVOLVED (8)								
HTMOM	MONTH DAY YEAR		YEAR		SEQUENTIAL NUMBER	REVISION	MONTH	DAY	YEAR		FACILITY NAM	MES	DOG	ER(S)					
										Seque	oyah, Uni	t 2	0	15/0/0/0/3/2/8					
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MC	ODE (9)	1	20.402(b)				20.406(c)			50.73(a)(2)(iv)			73.71(b)					
	POWER LEVEL (10) 1 0 10		-	20.406(a)(1)(i) 20.406(a)(1)(ii)			50.36(e)(1) 50.36(e)(2)			XX	50,73(a)(2)(v) 50,73(a)(2)(vii)					pecify in Abstract in Text, NRC Form			
			20	20.408(a)(1)(iii) 20.408(a)(1)(iv) 22.408(a)(1)(v)				e)(2)(i) e)(2)(ii)			50.73(a)(2)(viii)(/ 50.73(a)(2)(viii)(/ 50.73(a)(2)(x)			RC Form					
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Michael R. Cooper, Compliance Sec							tion	Engi	neer	the be	1. 1. 1.	6,1,5	8	17101	-1611	1416			
				-	COMPLETE	ONE LINE FOR	EACH CO	DMPONEN	TFAILUR	E DESCRIBE	ED IN THIS REPOR	T (13)							
CAUSE SYSTEM COMP		ONENT	N	MANUFAC- TURER	PEPORTABLE TO NPROS			CAUSI	SYSTEM	COMPONENT	MANUFAC- TURER		EPORTABLE TO NPRDS						

Unit 1 - Mode 1, 100% reactor power, 578 degrees F, 2235 psig. Unit 2 - Mode 1, 100% reactor power, 578 degrees F, 2235 psig.

Yes

SUPPLEMENTAL REPORT EXPECTED (14)

On June 25, 1984, the A-A auxiliary control air compressor was taken out of service for maintenance. Due to insufficient spare parts, it was not returned to service. On July 9, 1984, at 0750 CST, the B-B auxiliary control air compressor was removed from service. These compressors are not technical specification equipment, but are attendant equipment for various safety systems (auxiliary feedwater being the most limiting with respect to action times). With both trains inoperable, it was determined that entry into 3.0.3 should be made, and 3.0.3 was entered at 0750 CST on July 9, 1984. Power reduction to mode 3 was initiated but was stopped at 88% when the BB compressor was returned to service.

There was no effect on public health or safety.

1 0 7 15

Previous occurrences - none.

CMPI

XX YES (If yes, complete EXPECTED SUBMISSION DATE)

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MONTH

EXPECTED SUBMISSION DATE (15) YEAR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REQUEATORY COMMISSION

APPROVED OMB NO. 3150-0104

FACILITY NAME (1)	DOCKET NUMBER (2)						LER NUMBER (6)								PAGE (3)					
										FAR		SEOL	ENT: A	6	REVISIO NUMBE	N				
Sequoyah, Unit 1	0	15	0	0	0	13	12	7	8	14		0 1	41	5 -	- 010		0 2	OF	0 2	

TEXT (If more souce is required, use additional NRC Form 366A's) [17]

On June 25, 1984, the A-A auxiliary control air compressor became inoperable. A broken crankshaft was found during the teardown. A spare shaft was not available in Power Stores, so a new shaft was ordered on emergency basis. On July 9, 1984, at 0750 CST with the A-A compressor still inoperable, the B-B compressor began making a knocking noise and was removed from service. The NRC Resident Inspector, during a plant walkdown, noticed both compressors out of service and notified Operations of a potential technical specification problem with both air compressors out of service. A meeting was held by plant management, and it was determined (at 1350 CST) that both units should enter 3.0.3. Both units entered 3.0.3, and a late log entry of 0750 CST was made the 3.0.3 entry time. Power reduction was initiated and continued down to 88%. At that point (1448 CST), the B-B auxiliary control air compressor was returned to service and 3.0.3 was exited. Parts for the A-A compressor arrived on July 10, and the compressor was returned to service on July 11, 1984.

The auxiliary control air compressors are not technical specification equipment but are attendant equipment for various safety systems. This system is designed to remain operable during a maximum probable flood, and the design basis earthquake following a service air isolation.

This system supplies air to the following safety-related equipment:

- Control bay heating and ventilation system.
- Auxiliary building gas treatment system.
- Containment vacuum relief isolation valves.
- 4. Emergency gas treatment system.
- Auxiliary feedwater system. 5.
- 6. Steam generator pressure relief valves.
- Pressurizer spray valves.

Component Information

The compressors involved are Model No. 4-ESV-NL manufactured by Inger-Soll Rand. The cause of failure of the A-A compressor is unknown at this time. A failure analysis is being performed by TVA Metallurgy Staff and results will be supplied in a followup report. The B-B compressor's problem has been attributed to a broken lock-tab washer which allowed a locknut to back off. The piston rod then became disconnected from the cross-head.

Corrective Action

A review of the spare parts inventory will be made and the quantity of each stock item will be adjusted to ensure adequate parts are available for future repairs. A detailed maintenance procedure is being prepared specifically for these compressors and will be used for future repairs. The preventive maintenance program will be upgraded to ensure better reliability of the compressors.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant Post Office Box 2000 Soddy Daisy, Tennessee 37379

August 7, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT SQR0-50-327/84045

The enclosed licensee event report provides details concerning plant operation with both auxiliary control air compressors inoperable. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.v.

Very truly yours,

DR Walla

TENNESSEE VALLEY AUTHORITY

P. R. Wallace Plant Manager

Enclosure cc (Enclosure):

James P. O'Reilly, Director U.S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street, NW Atlanta, Georgia 30323

Records Center Institute of Nuclear Power Operations Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah