MEMORANDUM TO: Dennis M. Crutchfield, Director

Division of Reactor Program Management

FROM:

Alfred E. Chaffee, Chief [Original signed by]

Events Assessment and

Generic Communications Branch

Division of Reactor Program Management

SUBJECT:

OPERATING REACTORS EVENTS BRIEFING NOVEMBER 8, 1995 - BRIEFING 95-13

On November 8, 1995, we conducted an Operating Reactors Events Briefing (95-13) to inform senior managers from offices of the Commission, AEOD, NRR and regional offices of selected events that occurred since our last briefing on September 27, 1995. Attachment 1 lists the attendees. Attachment 2 presents the significant elements of the discussed events.

Attachment 3 contains reactor scram statistics for weeks ending October 1, October 8, October 15, October 22, October 29, and November 5, 1995. Three significant events were identified for input into the NRC Performance Indicator Program (Attachment 4).

Attachments: As stated (4)

cc w/atts: See next page

CONTACT:

Kathy Gray, NRR

(301) 415-1166

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NAME	KGray: kag Kyuy	NHunemuller /	RA	RDennig	Achaffee
DATE	11/08/95	11/08/95		11/8 /95	11/5/95

OFFICIAL RECORD COPY

W. Russell, NRR (0-12G18)

F. Miraglia, NRR (0-12G18)

F. Gillespie, NRR (0-12G18)

R. Zimmerman, NRR (0-12G18)

A. Thadani, NRR (0-12G18)

S. Varga, NRR (0-14E4)

J. Zwolinski, NRR (0-14H3)

J. Roe, NRR (0-13E4)

E. Adensam, NRR (0-13E4)

B. Sheron, NRR (0-7D26)

G. Lainas, NRR (0-7D26)

G. Holahan, NRR (0-8E2)

M. Virgilio, NRR (0-8E2)

S. Rosenberg, NRR (0-10E4)

R. L. Spessard, NRR (0-9A2)

B. Boger, NRR (0-10H5)

M. Markley, ACRS (T-2E26)

E. Jordan, AEOD (T-4D18)

C. Rossi, AEOD (T-4A9)

F. Congel, AEOD (T-4D28)

K. Brockman, AEOD (T-4A23)

S. Rubin, AEOD (T-4D28)

M. Harper, AEOD (T-4A9)

V. McCree, EDO (0-17G21)

J. Gilliland, PA (0-2G4)

D. Morrison, RES (T-10F12)

A. Bates, SECY (0-16G15)

T. Martin, Region I

R. Cooper, Region I

S. Ebneter, Region II

E. Merschoff, Region II

S. Vias, Region II

H. Miller, Region III

W. Axelson, Region III

L. Callan, Region IV

J. Dyer, Region IV

K. Perkins, Region IV/WCFO S. Newton, INPO

J. Zimmer, DOE

K. Jabbour (0-14H25) H. Berkow (0-14H25)

LIST OF ATTENDEES

OPERATING REACTORS EVENTS FULL BRIEFING (95-13)

NOVEMBER 8, 1995

NA	ME	OFFICE	MAME	OFFICE
- Commonweal	CHAFFEE	NRR	D. O'NEAL	NRR
J.	CARTER	NRR	J. BONGARRA	NRR
R.	DENNIG	NRR	L. VICK	NRR
	SKEEN	NRR	C. THOMAS	NRR
N.	HUNEMULLER	NRR	R. JONES	NRR
E.	BENNER	NRR	K. JABBOUR	NRR
Κ.	GRAY	NRR	M. TSCHILTZ	OCM/SJ
В.	GRIMES	NRR	J. ROSENTHAL	AEOD
Α.	CUGGAGE	NRR		

TELEPHONE ATTENDANCE (AT ROLL CALL)

Regions Region I Region II Region IV Resident Inspectors B. Holbrook, Hatch

Misc. T. Ross, Team Leader

Attachment 1

HATCH, UNIT 2 INADVERTENT DRAINING OF REACTOR VESSEL AND ISOLATION OF SHUTDOWN COOLING SYSTEM NOVEMBER 2, 1995

PROBLEM
WHILE IN THE COLD SHUTDOWN MODE WITH THE RESIDUAL HEAT
REMOVAL (RHR) SYSTEM IN THE SHUTDOWN COOLING MODE, BOTH
'B' RHR PUMP SUCTION VALVES (TORUS AND SHUTDOWN COOLING)
OPENED SIMULTANEOUSLY, RESULTING IN AN INADVERTENT
DRAINDOWN (OF APPROXIMATELY 60 INCHES OR 12,000 GALLONS)
OF THE REACTOR VESSEL TO THE TORUS.

CAUSE
THE ROOT CAUSE WAS MISWIRING OF A LIMIT SWITCH FOR THE SHUTDOWN COOLING SUCTION VALVE (FOO6B). THE MISWIRING REVERSED THE REMOTE SHUTDOWN PANEL (RSP) INTERLOCK LOGIC FOR THE TORUS SUCTION VALVE (FOO4B).

THE SAFETY SIGNIFICANCE OF THIS PARTICULAR EVENT APPEARS
TO BE LIMITED. THE SHUTDOWN COOLING CONTAINMENT ISOLATION
FUNCTIONED AS DESIGNED, AND MULTIPLE EMERGENCY CORE
COOLING SYSTEM (ECCS) MAKEUP SOURCES WERE OPERABLE AS
REQUIRED BY TECHNICAL SPECIFICATIONS. REACTOR VESSEL
WATER LEVEL REMAINED ABOVE THE TOP OF ACTIVE FUEL (TAF).

CONTACT: N. HUNEMULLER, NRR/DRPM/PECB

REFERENCE: 10 CFR 50.72 #29548

AIT: NO SIGEVENT: TBD

DISCUSSION

- ON OCTOBER 25, 1995, FOO6B FAILED TO OPERATE FOR UNKNOWN REASONS DURING A TEST FROM THE RSP.
- ON OCTOBER 30, 1995, FOO4B EXHIBITED IRREGULAR VALVE POSITION INDICATION FOR UNKNOWN REASONS DURING A TEST FROM THE CONTROL ROOM.
- ON NOVEMBER 2, 1995, THE LICENSEE WAS PERFORMING TROUBLESHOOTING TO DETERMINE THE CAUSE OF THE FAILURES.
- TWO NON-LICENSED PLANT EQUIPMENT OPERATORS (PEO) WERE STATIONED AT THE RSP.
- PRIOR TO THE EVENT, REACTOR VESSEL WATER LEVEL WAS +60 INCHES (TAF IS -160 INCHES), AND BOTH FOO6B AND FOO4B WERE CLOSED.
- THE LICENSEE BELIEVES THE CONTROL SWITCH ON THE RSP FOR FOO4B WAS IN THE 'OPEN' (NORMAL) POSITION.
- WHEN THE EMERGENCY TRANSFER SWITCH FOR THE 'B' RHR LOOP SUCTION VALVES WAS OPERATED, FOO4B DID NOT OPEN BECAUSE THE WIRING ERROR PREVENTED IT FROM OPENING UNLESS FOO6B WAS OPEN.
- THE OPERATORS DID NOT VERIFY CONTROL SWITCH POSITIONS ON THE RSP BEFORE OR AFTER THE TRANSFER. THE FAILURE OF FOO4B TO OPEN UPON OPERATION OF THE EMERGENCY TRANSFER SWITCH APPEARED TO CONFIRM THAT THE CONTROL SWITCH WAS IN THE 'CLOSE' POSITION.

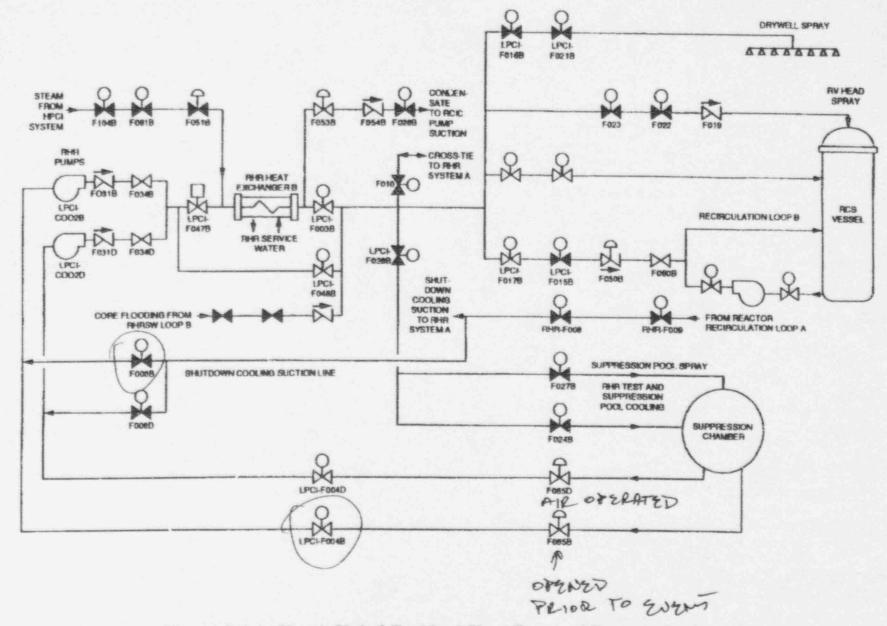
- PER DIRECTION FROM THE CONTROL ROOM, THE PEO AT THE RSP OPENED THE FOOGB VALVE, COMPLETING THE ERRONEOUS 'OPEN' LOGIC FOR FOO4B AND, INITIATING THE DRAINDOWN.
- THE DRAINDOWN WAS TERMINATED IN APPROXIMATELY 50 SECONDS BY AUTOMATIC CLOSURE OF THE SHUTDOWN COOLING SUCTION ISOLATION VALVES, AS DESIGNED. LEVEL WAS RESTORED IN APPROXIMATELY 25 MINUTES. SHUTDOWN COOLING WAS LOST FOR APPROXIMATELY 30 MINUTES WITH NO INCREASE IN REACTOR TEMPERATURE (42 DAYS AFTER SHUTDOWN WITH ONE-THIRD OF THE CORE BEING NEW FUEL).
- THERE WERE ALSO TWO OTHER MINOR DRAINDOWN EVENTS ON NOVEMBER 2 & 4, 1995, OF 1 TO 2 INCHES EACH, VIA THE RHR PUMP MINIMUM FLOW LINE TO THE SUPPRESSION POOL.

FOLLOWUP

- THE LICENSEE ASSEMBLED AN EVENT REVIEW TEAM AND AN NRC SPECIAL INSPECTION TEAM WAS SENT TO THE SITE.
- PRELIMINARY RESULTS FROM THE SPECIAL INSPECTION TEAM:
 - THE ROOT CAUSE WAS MISWIRING OF FOOGB
 - CONTRIBUTING FACTORS
 - POOR PLANNING AND GUIDANCE
 - INADEQUATE COMMUNICATION AND CONTROL
 - LACK OF SUPERVISION
- FINAL SPECIAL INSPECTION TEAM RESULTS WILL BE DOCUMENTED 'I INSPECTION REPORT 50-366/95-23 WHICH IS EXPECTED TO BE ISSUED IN APPROXIMATELY 30 DAYS.

THE NEED FOR A GENERIC COMMUNICATION WILL BE EVALUATED.

- THERE ARE NUMEROUS PREVIOUS GENERIC COMMUNICATIONS RELATED TO LOSS OF REACTOR COOLANT INVENTORY WHILE SHUTDOWN.



3.6-7

Figure 3.6-4. Hatch Unit 1 Residual Heat Removal System, Train B

REACTOR SCRAM

Reporting Period: 09/25/95 to 10/01/95

						YTD ABOVE	SELOW	YTD
DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	15%	15%	TOTAL
09/27/95	MCGUIRE 1	100	SM	Equipment Failure	NO	2	0	2
09/28/95	DRESDEN 3	77	SA	Equipment Failure	NO	3	0	3
09/30/95	BRUNSWICK 1	58	SA	Equipment Failure	NO	3	0	3
10/01/95	MCGUIRE 1	49	SA	Equipment Failure	NO	3	0	3

ATTACHMENT 3

Note: Year To Date (YTD) Totals Include Events Within The Calendar Year Indicated By The End Date Of The Specified Reporting Period

PERIOD ENDING 10/01/95

4 0 0	1.99 0.15	1.52			
0		1.52			
0	0.15		1.83	2.62	2
		0.08	0.04		0.00
	0.15	0.21	0.27	0.31	0.04
0	0.41	0.54	0.52	0.50	
0	0.23	0.17	0.13		
0	0.05		0.02		0.62
4	2.98	2.52	2.81	3.43	3.51
0	0.13	0.27	0.38	0.42	0.27
0	0.00	0.02			10.7
0	0.15	0.08	0.13	0.15	
0	0.10		0.02	0.08	
0	0.00		0.04	77.7	
0	0.00				0.19
0	0.38	0.37	0.57	0.65	0.46
4	3.36	2.89	3.38	4.08	3.97
	1995	1994	1993	1992	1991
NO. OF	WEEKLY	WEEKLY	MEEKLY		WEEKLY
SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
3	2.20	2.19	2.44	3.06	3.25
1	1.18	0.69	0.94	1.02	0.69
	0 0 0 0 0 0 0 0 0 0 0 0 4 NO. OF scrams	0 0.15 0 0.41 0 0.23 0 0.05 4 2.98 0 0.13 0 0.00 0 0.15 0 0.10 0 0.00 0 0.00 0 0.38 4 3.36 1995 NO. OF WEEKLY SCRAMS AVERAGE (YTD) 3 2.20	0 0.15 0.21 0 0.41 0.54 0 0.23 0.17 0 0.05 - 4 2.98 2.52 0 0.13 0.27 0 0.00 0.02 0 0.15 0.08 0 0.10 - 0 0.00 - 0 0.00 - 0 0.38 0.37 4 3.36 2.89 NO. OF WEEKLY WEEKLY SCRAMS AVERAGE (YTD) 3 2.20 2.19	0 0.15 0.21 0.27 0 0.41 0.54 0.52 0 0.23 0.17 0.13 0 0.05 - 0.02 4 2.98 2.52 2.81 0 0.15 0.08 0.13 0 0.10 - 0.02 0 0.15 0.08 0.13 0 0.00 - 0.04 0 0.00 - 0.04 0 0.00 - 0.04 0 0.00 - 0.04 0 0.38 0.37 0.57 4 3.36 2.89 3.38 NO. OF WEEKLY WEEKLY WEEKLY SCRAMS AVERAGE AVERAGE AVERAGE (YTD) 3 2.20 2.19 2.44	0 0.15 0.21 0.27 0.31 0 0.41 0.54 0.52 0.50 0 0.23 0.17 0.13 - 0 0.05 - 0.02 - 4 2.98 2.52 2.81 3.43 0 0.15 0.08 0.13 0.15 0 0.10 - 0.02 0.08 0 0.10 - 0.02 0.08 0 0.00 - 0.04 - 0 0.00 - 0.04 - 0 0.38 0.37 0.57 0.65 4 3.36 2.89 3.38 4.08 NO. OF WEEKLY WEEKLY WEEKLY WEEKLY SCRAMS AVERAGE AVERAGE AVERAGE AVERAGE (YTD) 3 2.20 2.19 2.44 3.06

^{*} Detailed breakdown not in database for 1991 and earlier

⁻ EXTERNAL cause included in EQUIPMENT FAILURE

⁻ MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR

⁻ OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

PERIOD ENDING 10/08/95

	NUMBER OF	1995 WEEKLY	1994 WEEKLY	1993 WEEKLY	1992 WEEKLY	1991* WEEKLY
SCRAM CAUSE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
POWER GREATER THAN OR EQUAL TO	15%					
EQUIPMENT FAILURE*	0	1.94	1.52	1.83	2.62	2.83
DESIGN/INSTALLATION ERROR*	0	0.15	0.08	0.04		0.02
OPERATING ERROR*	0	0.15	0.21	0.27	0.31	0.04
MAINTENANCE ERROR*	0	0.40	0.54	0.52	0.50	
EXTERNAL*	0	0.22	0.17	0.13		*
OTHER*	0	0.05		0.02		0.62
Subtotal	0	2.91	2.52	2.81	3.43	3.51
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.12	0.27	0.38	0.42	0.27
DESIGN/INSTALLATION ERROR*	0	0.00	0.02			
OPERATING ERROR*	0	0.15	0.08	0.13	0.15	
MAINTENANCE ERROR*	0	0.10		0.02	0.08	
EXTERNAL*	0	0.00	or become	0.04		
OTHER*	0	0.00				0.19
Subtotal	0	0.37	0.37	0.57	0.65	0.46
TOTAL	0	3.28	2.89	3.38	4.08	3.97
		1995	1994	1993	1992	1991
	NO. OF	WEEKLY	WEEKLY	MEEKLY	MEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
TOTAL AUTOMATIC SCRAMS	0	2.14	2.19	2.44	3.06	3.25
TOTAL MANUAL SCRAMS	0	1.15	0.69	0.94	1.02	0.69

- * Detailed breakdown not in database for 1991 and earlier
 - EXTERNAL cause included in EQUIPMENT FAILURE
 - MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR
 - OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

REACTOR SCRAM

Reporting Period: 10/09/95 to 10/15/95

						YTD	YTD	
						ABOVE	BELOW	YTD
DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	15%	15%	TOTAL
10/12/95	HARRIS 1	5	SA	Operating Error	NO	0	1	- 1

REACTOR SCRAM

Reporting Period: 10/16/95 to 10/22/95

						YTD	YTD	
						ABOVE	BELOW	YTO
DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	15%	15%	TOTAL
10/17/95	TURKEY POINT 3	100	SM	Maintenance Error	NO	1	1	2

Note: Year To Date (YTD) Totals Include Events Within The Calendar Year Indicated By The End Date Of The Specified Resorting Period

PERIOD ENDING 10/15/95

SCRAM CAUSE	NUMBER OF SCRAMS	1995 WEEKLY AVERAGE	1994 WEEKLY AVERAGE	1993 WEEKLY AVERAGE	1992 WEEKLY AVERAGE	1991* WEEKLY AVERAGE
		(YTD)				
POWER GREATER THAN OR EQUAL TO	15%					
EQUIPMENT FAILURE*	0	1.90	1.52	1.83	2.62	2.83
DESIGN/INSTALLATION ERROR*	0	0.15	0.08	0.04	*	0.02
OPERATING ERROR*	0	0.15	0.21	0.27	0.31	0.04
MAINTENANCE ERROR*	0	0.39	0.54	0.52	0.50	
	0	0.22	0.17	0.13		
EXTERNAL* OTHER*	0	0.05		0.02		0.62
Subtotal	0	2.86	2.52	2.81	3.43	3,51
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.12	0.27	0.38	0.42	0.27
DESIGN/INSTALLATION ERROR*	0	0.00	0.02			
	1	0.17	0.08	0.13	0.15	
OPERATING ERROR* MAINTENANCE ERROR*	0	0.10		0.02	0.08	
	0	0.00		0.04	*	
EXTERNAL* OTHER*	0	0.00				0.19
Subtotal	1	0.39	0.37	0.57	0.65	0.46
TOTAL	1	3.25	2.89	3.38	4.08	3.97
		1995	1994	1993 WEEKLY	1992 WEEKLY	1991 WEEKLY
	NO. OF	MEEKTA	WEEKLY	AVERAGE	AVERAGE	AVERAGE
SCRAM TYPE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	A Chinas	
TOTAL AUTOMATIC SCRAMS	1	2.11	2.19	2.44	3.06	3.25
TOTAL MANUAL SCRAMS	0	1.12	0.69	0.94	1.02	0.69

- * Detailed breakdown not in database for 1991 and earlier
 - EXTERNAL cause included in EQUIPMENT FAILURE
 - MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR
 - OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

PERIOD ENDING 10/22/95

	NUMBER OF SCRAMS	1995 WEEKLY AVERAGE	1994 WEEKLY AVERAGE	1993 WEEKLY AVERAGE	1992 WEEKLY AVERAGE	1991* WEEKLY AVERAGE
CRAM CAUSE	SCRAMS	(YTD)	ATERAGE	NVENNUS	ATE WASE	7116 17166
POWER GREATER THAN OR EQUAL TO	15%					
EQUIPMENT FAILURE*	0	1.85	1.52	1.83	2.62	2.83
DESIGN/INSTALLATION ERROR*	0	0.14	0.08	0.04		0.02
OPERATING ERROR*	0	0.14	0.21	0.27	0.31	0.04
MAINTENANCE ERROR*	1	0.40	0.54	0.52	0.50	
EXTERNAL*	0	0.21	0.17	0.13		
OTHER*	0	0.05	*	0.02		0.62
Subtotal	1	2.79	2.52	2.81	3.43	3.51
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.12	0.27	0.38	0.42	0.27
DESIGN/INSTALLATION ERROR*	0	0.00	0.02			
OPERATING ERROR*	0	0.17	0.08	0.13	0.15	
MAINTENANCE ERROR*	0	0.09		0.02	0.08	
EXTERNAL*	0	0.00	18.1	0.04		
OTHER*	0	0.00				0.19
Subtotal	0	6.38	0.37	0.57	0.65	0.46
TOTAL	1	3.17	2.89	3.38	4.08	3.97
		1995	1994	1993	1992	1991
	NO. OF	WEEKLY	WEEKLY	MEEKLY	MEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGE
TOTAL AUTOMATIC SCRAMS	0	2.06	2.19	2.44	3.06	3.25

^{*} Detailed breakdown not in database for 1991 and earlier

⁻ EXTERNAL cause included in EQUIPMENT FAILURE

⁻ MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR

⁻ OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

REACTOR SCRAM

Reporting Period: 10/23/95 to 10/29/95

						YTD	YTD	
						ABOVE	BELOW	YTD
DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	15%	15%	TOTAL
10/29/95	DRESDEN 3	90	SM	Other	NO	4	0	4

REACTOR SCRAM

Reporting Period: 10/30/95 to 11/05/95

						YTD	YTD	
						ABOVE	BELOW	YTD
DATE	PLANT & UNIT	POWER	TYPE	CAUSE	COMPLICATIONS	15%	15%	TOTAL
11/05/95	HARRIS 1	100	SA	Equipment Failure	NO	1	1	2

Note: Year To Date (YTD) Totals Include Events Within The Calendar Year Indicated By The End Date Of The Specified Reporting Period

PERIOD ENDING 10/29/95

	NUMBER OF	1995 WEEKLY	1994 WEEKLY	1993 WEEKLY	1992 WEEKLY AVERAGE	1991* WEEKLY AVERAGE
SCRAM CAUSE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	ATERAGE	n i Ennar
POWER GREATER THAN OR EQUAL TO	15%	(1107				
EQUIPMENT FAILURE*	0	1.81	1.52	1.83	2.62	2.83
DESIGN/INSTALLATION ERROR*	0	0.14	80.0	0.04		0.02
OPERATING ERROR*	0	0.14	0.21	0.27	0.31	0.04
MAINTENANCE ERROR*	0	0.39	0.54	C.52	0.50	
EXTERNAL*	0	0.21	0.17	0.13		
OTHER*	1	0.07		0.02		0.62
Subtotal	1	2.76	2.52	2.81	3.43	3.51
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.12	0.27	0.38	0.42	0.27
DESIGN/INSTALLATION ERROR*	0	0.00	0.02		5-14-15	11.4
OPERATING ERROR*	0	0.16	0.08	0.13	0.15	
MAINTENANCE ERKOR*	0	0.09		0.02	0.08	
EXTERNAL*	0	0.00		0.04		
OTHER*	0	0.00				0.19
Subtotal	0	0.37	0.37	0.57	0.65	0.46
TOTAL	1	3.13	2.89	3.38	4.08	3.97
		1995	1994	1993	1992	1991
	NO. OF	WEEKLY	WEEKLY	MEEKLY	WEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE	AVERAGE	AVERAGE	AVERAGE	AVERAGE
		(YTD)				
TOTAL AUTOMATIC SCRAMS	0	2.02	2.19	2.44	3.06	3.25
TOTAL MANUAL SCRAMS	- 1	1.11	0.69	0.94	1.02	0.69

^{*} Detailed breakdown not in database for 1991 and earlier

⁻ EXTERNAL cause included in EQUIPMENT FAILURE

⁻ MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR

⁻ OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

PERIOD ENDING 11/05/95

CHAN CAUCE	NUMBER OF SCRAMS	1995 WEEKLY AVERAGE	1994 WEEKLY AVERAGE	1993 WEEKLY AVERAGE	1992 WEEKLY AVERAGE	1991* WEEKLY AVERAGE
CRAM CAUSE	SURAMS	(YTD)	AVERAGE	PILANGE	ATEMPASE	
POWER GREATER THAN OR EQUAL TO	15%	(110)				
EQUIPMENT FAILURE*	1	1.79	1.52	1.83	2.62	2.83
DESIGN/INSTALLATION ERROR*	0	0.14	0.08	0.04		0.02
OPERATING ERROR*	0	0.14	0.21	0.27	0.31	0.04
MAINTENANCE ERROR*	0	0.39	0.54	0.52	0.50	
EXTERNAL*	0	0.20	0.17	0.13		
OTHER*	0	0.07		0.02		0.62
Subtotal	1	2.73	2.52	2.81	3.43	3.51
POWER LESS THAN 15%						
EQUIPMENT FAILURE*	0	0.11	0.27	0.38	0.42	0.27
DESIGN/INSTALLATION ERROR*	0	0.00	0.02			***
OPERATING ERROR*	0	0.16	0.08	0.13	0.15	
MAINTENANCE ERROR*	0	0.09		0.02	0.08	
EXTERNAL*	0	0.00		0.04		
OTHER*	0	0.00				0.19
Subtotal	0	0.36	0.37	0.57	0.65	0.46
TOTAL	1	3.09	2.89	3.38	4.08	3.97
		1995	1994	1993	1992	1991
	NO. OF	WEEKLY	WEEKLY	WEEKLY	WEEKLY	WEEKLY
SCRAM TYPE	SCRAMS	AVERAGE (YTD)	AVERAGE	AVERAGE	AVERAGE	AVERAGI
TOTAL AUTOMATIC SCRAMS	1	1.99	2.19	2.44	3.06	3.25
TOTAL MANUAL SCRAMS	0	1.09	0.69	0.94	1.02	0.69

- * Detailed breakdown not in database for 1991 and earlier
 - EXTERNAL cause included in EQUIPMENT FAILURE
- MAINTENANCE ERROR and DESIGN/INSTALLATION ERROR causes included in OPERATING ERROR
 - OTHER cause included in EQUIPMENT FAILURE 1991 and 1990

NOTES

- PLANT SPECIFIC DATA POSED ON INITIAL REVIEW OF 50.72 REPORTS FOR THE WEEK OF INTEREST. PERIOD IS MIDNIGHT SUNDAY THROUGH MIDNIGHT SUNDAY. SCRAMS ARE DEFINED AS REACTOR PROTECTIVE ACTUATIONS WHICH RESULT IN ROD MOTION, AND EXCLUDE PLANNED TESTS OR SCRAMS AS PART OF PLANNED SHUTDOWN IN ACCORDANCE WITH A PLANT PROCEDURE. THERE ARE 111 REACTORS HOLDING AN OPERATING LICENSE.
- 2. PERSONNEL RELATED PROBLEMS INCLUDE HUMAN ERROR, PROCEDURAL DEFICIENCIES, AND MANUAL STEAM GENERATOR LEVEL CONTROL PROBLEMS.
- 3. COMPLICATIONS: RECOVERY COMPLICATED BY EQUIPMENT FAILURES OR PERSONNEL ERRORS UNRELATED TO CAUSE OF SCRAM.
- 4. "OTHER" INCLUDES AUTOMATIC SCRAMS ATTRIBUTED TO ENVIRONMENTAL CAUSES (LIGHTNING), SYSTEM DESIGN, OR UNKNOWN CAUSE.

OEAB SCRAM DATA

Manual	and	Automatic	Scrams	for	1987		435
Manual	and	Automatic	Scrams	for	1988		291
Manual	and	Automatic	Scrams	for	1989		252
Manual	and	Automatic	Scrams	for	1990		226
Manua?	and	Automatic	Scrams	for	1991		206
Manual	and	Automatic	Scrams	for	1992		212
Manual	and	Automatic	Scrams	for	1993		175
Manual	and	Automatic	Scrams	for	1994		1 0
Manual	and	Automatic	Scrams	tor	1995	(YTD 11/05/95)	1 6
1101110101	THE R P. LEWIS	Water or an annual or to be					

OPERATING REACTOR PLANTS SIGNIFICANT EVENTS

SORT> Event Date QUERY> Event Type SIG & Close Out Date >= 09/19/95 & Close Out Date <= 10/31/95 & Event Type = "SIG"

PLANT & UNIT	DATE OF 50.72 EVENT NUMBER	DESCRIPTION OF EVENT	SIGNIFICANCE	OR BRIEFING	PRESENTER	CLOSEOUT RECORD
SALEM 1,2	06/07/9528904	500kv bus breaker problems caused two reactor coolant pumps to trip. Potential problems with pressurizer spray and three EDGs inoperable at other unit.	Safety-Related Cooling System		TAPPERT J.	HIGHLIGHT
HOPE CREEK 1	07/08/95 0	The plant entered a configuration in which shutdown cooling flow was unintentionally bypassing the core, with subsequent temperature rise and pressure change.	Safety-Related Cooling System	95-11	HODGE V.	HIGHLIGHT
LIMERICK 1	09/11/9529316	Licensee manually scrammed the plant due to a spurious open/stuck open SRV. Plant experienced some difficulty with operation of a one RHR pump in pool cooling mode, and possibly exceeded cooldown limits.	Unexpected Plant Performance	95-12	CARTER J.	HIGHLIGHT

OPERATING REACTORS EVENTS BRIEFING 95-13

LOCATION: 0-8 B11, WHITE FLINT WEDNESDAY, NOVEMBER 8, 1995, 11:00 A.M.

HATCH, UNIT 2

INADVERTENT DRAINING OF REACTOR VESSEL AND ISOLATION OF SHUTDOWN COOLING SYSTEM

PRESENTED BY:

EVENTS ASSESSMENT AND GENERIC COMMUNICATIONS BRANCH DIVISION OF REACTOR PROGRAM MANAGEMENT, NRR