



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 21, 1995

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
IES Utilities Inc.
Post Office Box 351
Cedar Rapids, IA 52406

SUBJECT: SAFETY EVALUATION OF RELIEF REQUESTS FOR DUANE ARNOLD ENERGY
CENTER INSERVICE TESTING PROGRAM (TAC NO. M91683)

Dear Mr. Liu:

By letter dated January 30, 1995, IES Utilities Inc., the licensee, submitted Revision 13 to the "Pump and Valve Inservice Testing Program for Duane Arnold Energy Center." On April 12, 1995, the licensee supplemented Revision 13 by providing a calculation referenced in the January 30, 1995 submittal. The staff has evaluated the relief requests in Revision 13 and has provided a Safety Evaluation (SE) of the relief requests as Enclosure 1.

A summary of the NRC actions regarding your relief requests is provided in the enclosed Table 1 (Enclosure 2). The staff has granted relief from or approved alternatives to the proposed testing requirements. Some of your relief requests do not require NRC approval pursuant to 10 CFR 50.55a. In some cases you have included noncode components in the inservice test (IST) program to meet requirements for testing outside of the requirements for 10 CFR 50.55a. Certain relief requests are approved pursuant to the guidance given in GL 89-04, as noted in Table 1. Relief Request PR-07 was evaluated and granted in the NRC's SE of March 21, 1995, as interim relief for a period of one year; therefore, PR-07 is not evaluated in the attached SE and the licensee should follow actions specified in the March 21, 1995, SE regarding the long-term status of PR-07.

The IST program relief requests are acceptable for implementation provided the action items identified in Section 5 of the enclosed Technical Evaluation Report (TER) are addressed within one year of the date of the SE or by the end of the next refueling outage, whichever is later. Additionally, the granting of relief is based upon the fulfillment of any commitments made by the licensee in the basis for each relief request and the alternatives.

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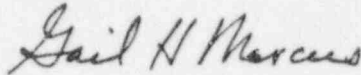
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Section 50.55a of the Code of Federal Regulations authorizes the Commission to grant relief from ASME Code requirements or to approve proposed alternatives upon making the necessary findings. The NRC staff's findings with respect to granting the relief requested or authorizing the proposed alternative as part of the licensee's IST program are contained in the enclosed SE.

Sincerely,



Gail H. Marcus, Director
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-331

Enclosures: 1. Safety Evaluation
2. Table 1 - Summary of Pump
and Valve Relief Requests
3. Technical Evaluation Report

cc w/encls: See next page

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Sincerely,

Original Signed By:

Gail H. Marcus, Director
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-331

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Mr. Lee Liu
IES Utilities Inc.

Duane Arnold Energy Center

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