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OPERATING MODE (8)         THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)           POWER (10)         1         20.406(a)         60.73(a)(2)(iv)         73.71(a)           POWER (10)         20.406(a)(1)(i)         90.36(a)(1)         90.36(a)(2)(iv)         73.71(a)           POWER (10)         20.406(a)(1)(ii)         90.36(a)(1)         90.36(a)(2)(iv)         90.73(a)(2)(iv)         73.71(a)           20.406(a)(1)(iii)         50.37(a)(2)(i)         50.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)           20.406(a)(1)(iv)         20.406(a)(1)(iv)         50.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)           20.406(a)(1)(iv)         20.406(a)(1)(iv)         50.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)           20.406(a)(1)(iv)         50.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)         90.73(a)(2)(iv)           IAME         CAUSE SYSTEM         COMPONENT         TELEPHONE NUMBER         AFEA CODE         41 1 9         2 5 9 - 5 0 0 0           CAUSE SYSTEM         COMPONENT         MANUFAC         REPORTABLE         TURER         TO NPROS               X         A 1B         F1C 1V<	0 5	1 5	8 4	8	4-	000	0 1	0 8	0 9	8 4				0	51010	101		
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20.406(a)(11(iii)         XX         50.73(a)(2)(i)         50.73(a)(2)(will)(A)         305A/           20.406(a)(11(iv)         50.73(a)(2)(ii)         50.73(a)(2)(will)(B)         50.73(a)(2)(will)(B)         50.73(a)(2)(will)(B)         50.73(a)(2)(will)(B)           LICENSEE CONTACT FOR THIS LER (12)           TELEPHONE NUMBER           AREA CODE           G. E. Narus, Shift Technical Advisor           COMPLETE ONE LINE FOR EACH COMPONENT PAILURE DESCRIBED IN THIS REPORT (13)           CAUSE SYSTEM COMPONENT           ANUFAC           REPORTABLE           COMPONENT           MANUFAC           TURER           COMPONENT           MANUFAC           TURER           SUPPLEMENTAL, HEPORT EXPECTED (14)           EXPECTED SUBMISSION DATE)           X           VES (// yes, complete EXPECTED SUBMISSION DATE)           X           SUPPLEMENTAL, HEPORT EXPECTED (14)           SUPPLEMENTAL, HEPORT EXPECTED (14)           SUBMISSION DATE)           X           SUPPLEMENTAL, HEPORT EXPECTED (14) <td>(10)</td> <td>0</td> <td>914</td> <td></td> <td>20.408</td> <td>(a)(1)(ii)</td> <td></td> <td>50.36(c)</td> <td>(2)</td> <td></td> <td></td> <td>50.73(a)(2)(vii</td> <td></td> <td></td> <td>OTHER (Son</td> <td>Text NR</td> <td>C Form</td>	(10)	0	914		20.408	(a)(1)(ii)		50.36(c)	(2)			50.73(a)(2)(vii			OTHER (Son	Text NR	C Form	
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20.408(a)(1)(v)     50.73(a)(2)(iii)     50.73(a)(2)(iii)       LICENSEE CONTACT FOR THIS LER (12)       TELEPHONE NUMBER       AREA CODE       G. E. Narus, Shift Technical Advisor       COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)       CAUSE SYSTEM COMPONENT       MANUFAC       TURER       REPORTABLE       CAUSE SYSTEM COMPONENT       MANUFAC       TURER       TURER       COMPONENT       MANUFAC       TURER       COMPONENT       MANUFAC       TURER       TURER       COMPONENT       MANUFAC       TURER       TURER       COMPONENT       MANUFAC       TURER       TURER       COMPONENT       MANUFAC       TURER       TURER       SUPPLEMENTAL, HEPORT EXPECTED [14]       EXPECTED SUBMISSION DATE)       NO       DATE				-	20.406	(a)(1)(iv)	-	50.73(a)	(2)(0)		H	60.73(a)(2)(viii	)(8)					
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At 2020 hours on May 13, 1964, the Control Room operator noted makeup tank level dropping at a rate of 3 GPM. This placed the unit in the Action Statement of Technical Specification 3.4.6.2(b). Attempts to isolate the leak were unsuccessful. A containment entry was made, and the leak was found to be in the vicinity of the pressurizer spray control valve. At 0020 hours on May 16, 1984, a reactor shutdown was initiated, and an Unusual Event was declared at 0130 hours. The unit entered Mode 3, Hot Standby, at 0600 hours. During the course of the Reactor Coolant System depressurization on May 16, the unit entered Reactor Protection System Shutdown Bypass without performing a required surveillance test which was then done later the same day after the error was confirmed. At 1515 hours on May 16, 1984, the unit was removed from the Unusual Event status after the determination that the leakage was not pressure boundary leakage. The leaking valve, RC49, the spray valve bypass, had a packing leak and was repacked and declared operable at 1115 hours on May 17, 1984. During the subsequent repressurization a Reactor Protection System actuation was received from Reactor Protection System Shutdown Bypass. This was due to difficulties in pressure control at this point in the repressurization.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO. 3150-010 EXPIRES: 8/31/85									
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Description of Occurrence: At 2020 hours on May 15, 1984, a Control Room operator noted makeup tank level decreasing at a rate of 3 GPM with the plant in Mode 1 at 94% power. This placed the unit in the action statement of Technical Specification 3.4.6.2(b) which allows up to 1 GPM of unidentified reactor coolant leakage, and the unit entered action item (b) of Technical Specification 3.4.6.2 which requires reduction of the leak rate to within limits within 4 hours, or Hot Standby within the next 6 hours, and in Cold Shutdown within the following 30 hours.

The Control Room began isolating portions of the Makeup System to try to isolate the leak, but the leak rate did not decrease. At 2323 hours on May 15, 1984, a containment entry was made at power to locate the leak. At 2330 hours, the containment entry crew reported that the leak was in the vicinity of the pressurizer spray valve (RC2).

A reactor shutdown was initiated at 0020 hours on May 16, 1984. At 0130 hours on May 16, 1984, the Shift Supervisor declared an Unusual Event since the containment entry crew could not determine that the leak was not from a pressure boundary and, therefore, could not be necessarily classified as identified leakage. At 0600 hours on May 16, 1984, the was manually isolated which removed the unit from the Unusual Event and removed the unit from the action items of Technical Specification 3.4.6.2. It was discovered that the leakage was from the spray valve bypass valve (RC49) packing.

At 0740 hours on May 16, 1984, in the course of the reactor shutdown and depressurization, the unit went into Reactor Protection System (RPS) Shutdown Bypass without the surveillance requirements of Technical Specification 4.3.1.1 being met. This was confirmed later on May 16, 1984, and the Surveillance Test ST 5030.15 was begun and finished with Shift Supervisor approval at 0100 hours on May 17, 1984.

At 1245 hours on May 17, 1984, the unit received a RPS actuation due to reaching the RPS shutdown bypass high pressure trip setpoint prior to the point in the startup procedure where shutdown bypass is removed. This occurred when they were increasing pressure to clear the Safety Features Actuation System (SFAS) Reactor Coolant System (RCS) low pressure trips which were set high in the SFAS instrumentation allowable band at this time. That situation, plus the error inherent in the wide-range RCS pressure strip chart recorders that the operators were looking at for manually controlling pressure caused them to approach and reach the RPS shutdown bypass trip setpoint before they could stop the pressure increase and maintain pressure in order to remove the RPS shutdown bypass.

Designation of Apparent Cause of Occurrence: The apparent cause of this occurrence was a component failure of the packing of the mini-spray valve RC49. This valve is set in a throttled position so the packing is always exposed to system pressure.

The cause of the failure to perform the surveillance test on the RPS shutdown bypass channels was due to personnel error related to the fact that each RPS channel is

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LICENSEE EVENT REPOR	T (LER) TEXT CONTI	NUATIO	N	U.S.	APPROVED O	MB NO. 31	COMMISSIO
PACILITY NAME (1)	DOCKET NUMBER (2)		LER	NUMBER (6)		PA	GE (3)
		VEAR	5	NUMBER	REVISION NUMBER		
Davis-Besse Unit I	0 15 10 10 10 13 14 1	6 814	-	01016	- 011	013	OF 0 13
TEXT (If more space is required, use additional NRC Form 356A's) (17)		01014		01017	1011		10 13
tested monthly. However, previous to check on the bypass circuit that must reactor shutdown.	this event the be done prior t	month] o enab	ly te bling	est di g shut	dn't in down by	clude pass	the on a
The cause of the RPS actuation during to operational difficulties concerning	g the subsequent ng pressure contr	unit r ol at	cepre this	ssuri: poin	zation the	was d e sta	ue rtup.
Analysis of Occurrence: There was no All the leakage was internal to the o occurred as a result of this occurren controlled manner.	o danger to the h containment vesse nce. The unit wa	ealth 1. No s shut	and rad down	safet lioact in an	y of the ive rele n order:	e pub ease ly,	lic.
Corrective Action: At 1515 hours on isolated, and the line was permitted ance personnel found about half of th packing was removed from RC43, and th Order 1-84-1602-00. The valve was ma position and declared operable at 111	May 16, 1984, RC to depressurize he packing blown he valve was repa anually cycled an 15 hours on May 1	49 (mi prior out of cked u d relo 7, 198	to m RC4 Inder ocked 34.	pray mainten 9. Th Main 1 in in	valve) v nance. he rema: tenance ts throu	was Main ining Work ttled	ten-
Toledo Edison is investigating the popacking means for selected RCS valves	ossible use of di 3.	fferen	nt ty	pes o	f packin	ng an	d
The Surveillance Test, ST 5030.15, for immediately after it was determined to reactor shutdown process. We are now Check Monthly Surveillance Test, ST 5 ST 5030.15 on a monthly basis, regard always be current when needed during	or the RPS shutdo that it wasn't do v in the process 5030.02, to perfo iless of plant op a plant shutdown	wn byp ne as of mod orm the eratin	requiring mo	circu ired ing the uired ode, so	its was earlier e RPS Cl section uch that	perf in t hanne ns of t it	ormed he 1 will
Several corrective efforts are in proties prior to going off of RPS shutdo avoid tripping on shutdown bypass hig procedure are being considered, and H indication instrumentation. The Inst ing the possibility of altering how to for SFAS low pressure block reset and	ogress to correct own bypass during th pressure. Mod Engineering is re trument & Control the setpoints are the RPS shutdow	the p a rep ificat viewin Depar set i m bypa	oress ions ig Co tmer in th iss h	to the state of th	ontrol o tion in he star Room pr also inv llowable ressure	diffi orde tup ressu vesti e ban trip	cul- r to re gat- d
Failure Data: No similar occurrences	s have been repor	ted.					

Report No: NP-33-84-06

DVR No(s): 84-063, 84-066, 84-067

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August 9, 1984

Log No. K84-1074 File: RR 2 (NP-33-84-06)

Docket No. 50-346 License No. NPF-3

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Gentlemen:

## LER No. 84-006 Davis-Besse Nuclear Power Station Unit 1 Date of Occurrence: May 15, 1984

Enclosed is Revision 1 to Licensee Event Report 84-006. The revisions to the report are indicated by a "1" in the left margin of each page.

Please replace your previous copy of this report with the attached revision.

Yours truly,

Stephen menny

Stephen M. Quennoz Acting Station Superintendent Davis-Besse Nuclear Power Station

SMQ/1jk

Enclosure

cc: Mr. James G. Keppler, Regional Administrator, USNRC Region III

> Mr. Walt Rogers DB-1 NRC Resident Inspector



JCS/001