

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JULY 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

8408160140 840731
PDR ADDCK 05000373
R PDR

Document 0043r/0005r

IE24
11

TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT ONE
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - 2. Facility or Procedure Changes Requiring NRC Approval
 - 3. Tests and Experiments Requiring NRC Approval
 - 4. Corrective Maintenance of Safety Related Equipment
 - C. LICENSEE EVENT REPORTS
 - D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
 - E. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

JULY 1-31

The Unit started the reporting period at 94% power. At 0900 hours on July 17, Reactor power was reduced to 76% due to a single rod scram. At 2200 hours on July 19, reactor power was raised to 92% power. At 2030 hours on July 20, reactor power was raised to 100%. At 2200 hours on July 21, reactor power was reduced to 92% due to high condensate temperature and high condenser back pressure. The reactor was critical for the entire month of July totaling 744 hours.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Amendment No. 17. - This amendment revised the main steam tunnel differential temperature setpoint as specified in Technical Specification Table 3.3.2-2. The change was necessitated to reflect actual plant as built conditions rather than original design criteria.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L31475		Cable Tray	Hanger support missing	None. Other support still functional.	Added hanger to provide additional support.
L31700		Post LOCA Monitor	Recorder Pen ramps upscale after warm up.	None. Redundant channel still operable.	Installed new output transistor and regulator chip.
L35485		Computer Room Supply Damper	Damper sticks closed.	Inadequate ventilation in computer room.	Freed up binding linkage and lubricated.
L36772		Suppression Pool Spray Valve	Trips on Thermals after valve closes	Valve still functional after reset.	Replaced defective torque switch.
L38158		RHR Service Water Pump	Lube oil leak from pump bearing	Potential loss of pump lubrication.	Repaired oil leak.
L38249		LPRM	Reads downscale at 75% power	None. Redundant channel operable.	Cleaned connectors.
L38327		LPCS water Leg pump	Pump coupling damaged. Cover O-rings out of position.	Could cause pump failure leaving LPCS system unfilled	Repaired coupling cover O-rings.
L38481		HPCS Water- leg discharge check valve.	Valve leaks through	Allows backflow through pump from HPCS system.	Repaired check valve seat.
L38909		Suppression pool temper- ature re- corder	Recorder stops driving while points still printing.	None. Redundant channel still operable.	Replaced low speed servo motor for chart drive.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, July 1 through July 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-031-00	6/15/84	Reactor Water Cleanup Isolation on Pump Mode Differential Temperature.
84-032-00	6/12/84	Reactor Water Cleanup High Differential Flow Isolation.
84-033-00	6/24/84	Reactor Water Cleanup Differential Flow .
84-034-00	6/17/84	Unsecured High Radiation Area Door.
84-035-00	6/19/84	B Diesel Fire Pump Flywheel.
84-036-00	6/14/84	Unsecured High Radiation Area Door.
84-037-00	6/21/84	OA Diesel Fire Pump Flywheel.
84-038-00	6/22/84	Unsealed Fire Penetration/Sleeve.
84-039-00	6/24/84	Scram on Low Reactor Water Level.
84-040-00	6/25/84	Reactor Water Cleanup Differential Flow Isolation.
84-041-00	6/28/84	Mechanical Fire Penetrations.
84-042-00	7/3/84	High Radiation Area Unsecured and Unposted.
84-043-00	6/27/84	Reactor Water Cleanup Differential Flow Isolation.
84-044-00	7/2/84	Failure of High Pressure Core Spray Water Leg Pump Check Valves.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE August 6, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: JULY 1984 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>744.0</u>	<u>4001.8</u>	<u>4001.8</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1076.3</u>	<u>1076.3</u>
7. HOURS GENERATOR ON LINE	<u>744.0</u>	<u>3841.7</u>	<u>3841.7</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>2328797</u>	<u>10902922</u>	<u>10902922</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>756661</u>	<u>3569784</u>	<u>3569784</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>725819</u>	<u>3402863</u>	<u>3402863</u>
12. REACTOR SERVICE FACTOR	<u>100%</u>	<u>78.3%</u>	<u>78.3%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>99.3%</u>	<u>99.3%</u>
14. UNIT SERVICE FACTOR	<u>100%</u>	<u>75.2%</u>	<u>75.2%</u>
15. UNIT AVAILABILITY FACTOR	<u>100%</u>	<u>75.2%</u>	<u>75.2%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>94.2%</u>	<u>64.3%</u>	<u>64.3%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>90.5%</u>	<u>61.8%</u>	<u>61.8%</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0%</u>	<u>20.7%</u>	<u>20.7%</u>
19. SHJTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
On October 1, 1984 there will be a four week outage to inspect the drywell and perform scheduled surveillances.			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>N/A</u>			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY	<u> </u>	<u>6/21/82</u>	
INITIAL ELECTRICITY	<u> </u>	<u>9/04/82</u>	
COMMERCIAL OPERATION	<u> </u>	<u>1/1/84</u>	

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
 UNIT: LASALLE ONE
 DATE: AUGUST 6, 1984
 COMPLETED BY: Randy S. Dus
 TELEPHONE: (815) 357-6761

MONTH: July 1984

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1. _____	995	17. _____	831
2. _____	1014	18. _____	912
3. _____	954	19. _____	962
4. _____	995	20. _____	995
5. _____	1010	21. _____	1019
6. _____	971	22. _____	927
7. _____	920	23. _____	933
8. _____	967	24. _____	988
9. _____	1051	25. _____	972
10. _____	1037	26. _____	997
11. _____	1055	27. _____	1008
12. _____	1058	28. _____	990
13. _____	1052	29. _____	950
14. _____	888	30. _____	1028
15. _____	893	31. _____	937
16. _____	933		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JULY 1984

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE August 6, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				

NONE.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

There were no relief valve operations for Unit One for this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-560-84	Suppression Pool Spray Isolation Valve.	Replace Torque Switch
1-562-84	HPCS Diesel/Gen.	Replace air starter motor.
1-563-84	HPCS Water Leg Pump	Repair discharge check valve.
1-564-84	HPCS Water Leg Pump discharge check valve.	Repair discharge check valve.
1-565-84	HPCS Pump	Troubleshoot CY losses through HPCS suction.
1-571-84	LPCS Water Leg Pump.	Repair Water Leg Pump.
1-573-84	HPCS Suction from CY.	Reduce CY Utilization
1-574-84	1A Diesel/Gen.	Lubrication.
1-575-84	1B Diesel/Gen.	Lubrication.
1-585-84	1RHR Pump	Oil Samples.
1-615-84	RHR Heat Exchange Outlet Stop Valve.	Repair valve motor.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JULY 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT TWO
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - 2. Facility or Procedure Changes Requiring NRC Approval
 - 3. Tests and Experiments Requiring NRC Approval
 - 4. Corrective Maintenance of Safety Related Equipment
 - C. LICENSEE EVENT REPORTS
 - D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
 - E. UNIQUE REPORTING REQUIREMENTS
 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JULY 1-7

The unit started the reporting period at 64% power. At 2100 hours on July 5, the turbine generator was manually tripped as part of the startup test program. At 0040 hours on July 6, the mode switch was placed in "Shutdown". The reactor was critical for 120 hours and 40 minutes.

JULY 8-9

At 0617 hours on July 8, the reactor was critical. At 1750 hours on July 8, the generator was synchronized to the grid. At 2330 hours on July 8, reactor power was raised to 19%. At 1906 hours on July 9, an intercept valve fast closure occurred. At 2250 hours on July 9, the reactor mode switch was placed in "Shutdown". The reactor was critical for 40 hours and 33 minutes.

JULY 10-31

At 0646 hours on July 10, the reactor was critical. At 0500 hours on July 11, the generator was synchronized to the grid. At 0630 hours on July 11, reactor power was raised to 26%. At 1500 hours on July 11, reactor power was raised to 52%. At 1500 hours on July 13, reactor power was raised to 70%. At 0700 hours on July 20, reactor power was raised to 85%. At 0700 hours on July 21, reactor power was reduced to 63% for flux shaping. At 1700 hours on July 23, reactor power was raised to 86%. At 0000 hours on July 27, reactor power was raised to 96%. At 2025 hours on July 27, a power reduction was commenced for TDRFP testing. At 0700 hours on July 28, reactor power was reduced to 47%. At 1400 hours on July 28, reactor power was reduced to 14% for investigation of steam leaks on the Moisture Separator Reheater. At 2115 hours on July 28, the main turbine was tripped. At 2130 hours on July 28, the reactor was manually scrammed so that repairs could be made to the Moisture Separator Reheater. The reactor was critical for 446 hours and 44 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Amendment No. 2 - This amendment revised the main steam tunnel differential temperature setpoint as specified in Technical Specification Table 3.3.2-2. The change was necessitated to reflect actual plant as built conditions rather than original design criteria.

Amendment No. 3- This amendment added a reator scram on low control rod drive pump discharge pressure as required by license condition 2. c(7).

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LEP	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L33233		Control Room Rad Monitor	Rad Monitor would not operate due to faulty detector.	None. Redundant channels still operable.	Replaced defective detector and detector connectors.
L37122		"O" Diesel/ Generator	Small Fuel oil leak into the lube oil system.	Could cause a breakdown of Lubricating oil causing bearing damage.	Stopped leak.
L38262		RHR Service Water Pump.	Suction pressure reading Incorrectly.	None. Redundant pressure indication available.	Repaired defective suction pressure gauge.
L38430		Rod Block Monitor	Defective power supply causing false reading	Reads 50% when APRM read 65%.	Replaced power supply.
L38452		LPRM	Occasionally bouncing causing false indication.	Causes downscale alarm.	Replaced defective cards.
L38597		Main Steam Isolation Valve.	Solenoid valve leaking air.	Potential loss of air to main steam isolation valve.	Replaced solenoid valve.
L39254		Div. III Battery.	Corrosion on battery terminals.	Potential loss of DC power supply to HPCS pump.	Replaced connections bolts & nuts and cleaned connector

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, July 1 through July 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-026-00	6-9-84	RWCU Hi Differential Temperature Pump Room and Heat Exchanger Room Isolation, Div. I.
84-027-00	6/4/84	Loss of Reactor Water Cleanup Isolation Leak Detection.
84-028-00	6/17/84	Reactor Water Cleanup High Ambient Temperature Isolation.
84-029-00	6/11/84	Reactor Water Cleanup High Differential Flow Isolations.
84-030-00	6/18/84	HPCS Discharge Relief Valve Failure.
84-031-00	6/22/84	Reactor Water Cleanup Differential Temperature Isolation.
84-032-00	6/26/84	Reactor Water Cleanup Isolation.
84-033-00	6/8/84	Failure of Isolation Valve to Close.
84-034-00	7/3/84	High Radiation Area Unsecured and Unposted.
84-035-00	7/9/84	Scram On Reactor Vessel High Pressure.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE August 6, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: July 1984 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWe): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>608.0</u> | <u>2518.5</u> | <u>2518.5</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>136.1</u> | <u>928.4</u> | <u>928.4</u> |
| 7. HOURS GENERATOR ON LINE | <u>566.5</u> | <u>1704.9</u> | <u>1704.9</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1324810</u> | <u>2813768</u> | <u>2813768</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>400934</u> | <u>777450</u> | <u>777450</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>382416</u> | <u>730509</u> | <u>730509</u> |
| 12. REACTOR SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 14. UNIT SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: August 1, 1984
 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
- | | FORECAST | ACHIEVED |
|----------------------|-------------------|-------------------|
| INITIAL CRITICALITY | <u> </u> | <u>3/10/84</u> |
| INITIAL ELECTRICITY | <u> </u> | <u>4/20/84</u> |
| COMMERCIAL OPERATION | <u>Aug. 84</u> | <u> </u> |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
 UNIT: LASALLE TWO
 DATE: August 6, 1984
 COMPLETED BY: Randy S. Dus
 TELEPHONE: (815) 357-6761
 MONTH: July 1984

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1. _____	606	17. _____	650
2. _____	658	18. _____	469
3. _____	680	19. _____	670
4. _____	610	20. _____	811
5. _____	571	21. _____	613
6. _____	0	22. _____	699
7. _____	0	23. _____	839
8. _____	29	24. _____	946
9. _____	284	25. _____	983
10. _____	0	26. _____	990
11. _____	321	27. _____	908
12. _____	626	28. _____	264
13. _____	687	29. _____	0
14. _____	733	30. _____	0
15. _____	660	31. _____	0
16. _____	628		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
 UNIT NAME LaSalle Two
 DATE August 6, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

REPORT MONTH JULY 1984

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
21	840706		S	68.8	B	2	Turbine/Generator manually tripped and reactor shutdown per STP-27-2.
22	840709		S	33.9	B	2	Unit shutdown to troubleshoot problem with turbine/Generator intercept valve.
23	840721		S	0.0	H	5	Power reduction for flux shaping.
24	840728		S	74.8	B	2	Unit shutdown to repair steam leaks on the moisture separator reheater.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
6/9/84	2B21-F013D	1 AUTO	981 psig	Rx Scram
6/9/84	2B21-F013S	1 AUTO	981 psig	Rx Scram
6/9/84	2B21-F013U	1 AUTO	981 psig	Rx Scram

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-847-84	2A RHR Pump	Oil Samples
2-867-84	2A Diesel/Generator	Lubrication
2-870-84	HPCS Diesel/Generator motor driven air start compressor.	Repair air compressor
2-871-84	HPCS Diesel/Generator motor driven air start compressor.	Repair air compressor
2-900-84	2B Diesel/Generator motor driven air start compressor.	Repair air compressor.
2-912-84	MSR 1st stage reheat scavenging steam stop valve.	Repair valve.
2-928-84	2B Diesel Generator motor driven air start compressor.	Repair air compressor.
2-929-84	2B Diesel Generator motor driven air start compressor.	Repair air compressor
2-932-84	RHR heat exchanger shell side vent upstream stop valve.	Adjust limitorque

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 6, 1984

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period covering July 1 through
July 31, 1984.

Very truly yours,

G. J. Diederich
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident