U. S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) **UPDATE REPORT:** LICENSEE EVENT REPORT 5 . PREVIOUS REPORT DATE: 4-28-81 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 0 0 0 0 0 0 0 LICENSEE COOF CON'T REPORT 0 8 1(8) 0 4 0 4 1 6 SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During unit power operation, the snubber shaft of hydraulic snubber 1-E11-47SS326 0 2 broke as a result of water hammer of the A Residual Heat Removal (RHR) System steam 0 3 condensing piping, which occurred when the system was started to recirculate the 0 4 suppression pool for sampling. The snubber is located downstream of the subject V 5 | piping inlet pressure control valve, I-EII-F051A. This event did not affect the 0 6 health and safety of the public. Technical Specifications 3.7.5, 6.9.1.8i 80 SYSTEM CAUSE COMP CAUSE VALVE SUBCODE COMPONENT CODE SUBCODE C (13) D (15 (16) E R OCCURRENCE REVISION SEQUENTIAL REPORT REPORT NO. CODE TYPE NO. LER/RO 0 T REPORT 0 4 1 6 1 11 NUMBER NPRD-4 PRIME COMP. COMPONENT ATTACHMENT SUBMITTED ACTION FUTURE EFFECT ON PLANT METHOD HOURS (22) FORM SUB SUPPLIER MANUFACTURER Y (23 (25 2 1 Z (2) 01 0 Y (24 0 D F (19 Z 0 0 B A 18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) A steam pocket in the subject piping, resulting from leakage past the FO51A respective jupstream isolation valve, Ell-F052A, caused the water hammer. 47SS326 was repaired, tested and returned to service. To help preclude future similar events, appropriate modifications involving these valves have been installed on both units. Additional corrective actions regarding this event are presently being developed. 80 METHOD OF FACILITY OTHER STATUS DISCOVERY DESCRIPTION (32) % POWER B (31) E (28) 01 Plant Surveillance 7 5 NA ACTIVITY CONTENT 80 LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE Z (33) NA Z (34) NA PERSONNEL EXPOSURES 80 44 DESCRIPTION (39) NUMBER 0 0 0 37 Z NA (38 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 (40)NA LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) NA 8408160133 840803 PDR ADOCK 05000325 PUBLICITY PDR NRC USE ONLY DESCRIPTION (45 NA 69 80.5 919-457-9521 M. J. Pastva, Jr. NAME OF PREPARER PHONE ...

LER ATTACHMENT - RO #1-81-46

Facility: Unit 1

Event Date: April 14, 1981

During Unit 1 power operation, water hammer of the A Residual Heat Removal (RHR) System steam condensing piping to the A RHR heat exchanger occurred when the system was started to recirculate the suppression pool for chemistry sampling. Subsequent plant surveillance revealed that hydraulic snubber 1-E11-47SS326, which restrains the subject steam condensing piping at a point downstream of the piping inlet pressure control valve, 1-E11-F051A, was inoperable due to a broken snubber shaft.

A failure analysis of the broken snubber shaft indicates the shaft breakage occurred due to shear failure. The subject piping is normally full of RHR System water up to the line isolation valve, 1-E11-F052A, which is upstream of the F051A. Leakage past both closed valves allowed a steam pocket to form in the subject line. When the RHR System A loop was started for recirculation and sampling of the suppression pool, the steam pocket was displaced by the loop water volume and water hammer of the line occurred, thereby causing the snubber shaft to break.

Following the event, the snubber, Bergen and Patterson Part No. 3 KIP HSSA-3, was repaired, satisfactorily tested, and returned to service. Once per four hours, venting of the RHR System steam condensing loop piping on both units was initiated to ensure the piping full of water.

In accordance with plant modifications, the F052A and B line isolation values on both units were replaced to help eliminate the leakage problem. In addition, manual isolation values will be installed in accordance with plant modifications on the respective 1" bypass lines around the F051A, B, and F052A, B values of the A and B RHR System steam condensing loops on both units to help eliminate a source of inleakage to the loops. On both units, A and B RHR System steam condensing loops are currently being vented once per eight hours in plant operational modes requiring their operability.



No.

Carolina Power & Light Company 7

Brunswick Steam Electric Plant P. O. Box 10429 Southport, NC 28461-0429 August 3, 1984

FILE: B09-13510C SERIAL: BSEP/84-1149

Mr. James P. O'Reilly, Administrator U. S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street N.W. Atlanta, GA 30323

> BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1 DOCKET NO. 50-325 LICENSE NO. DPR-71 SUPPLEMENT TO LICENSEE EVENT REPORT 1-81-46

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.8i of the Technica! Specifications for Brunswick Steam Electric Plant, Unit 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within fourteen (14) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

Clint

C. R. Dietz, General Manager Brunswick Steam Electric Plant

RMP/mcg/LETCG2

Enclosure

cc: Mr. R. C. DeYoung NRC Document Control Desk

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