

UNITED STATES NUCLEAR REGULATORY COMMISSION

REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

APR 1 3 1984

MEMORANDUM FOR:

Region III Management and Technical Staff

FROM:

Charles E. Norelius, Director, Division of Project

and Resident Programs

SUBJECT:

REVISION TO MC 0516 - SYSTEMATIC ASSESSMENT OF

LICENSEE PERFORMANCE (SALP)

Attached is an advance copy of the proposed revision to NRC Manual Chapter 0516, Systematic Assessment of Licensee Performance, for your review.

As requested in Mr. DeYoung's memorandum of March 12, 1984, we plan to implement this procedure immediately and will follow the new guidelines beginning with the Clinton SALP Report which is now in preparation at the Resident Office.

While the significant changes to the new procedure are highlighted by darkened lines in the right margin, I would like to summarize a few of the administrative changes which have been incorporated into the procedure which differ from those previously followed.

- The format for SALP inputs has not been modified, but expanded to include submittal of an Evaluation Matrix which will serve to provide background in establishing an overall performance rating.
- Issuance of a Preliminary SALP Report will no longer be required. The SALP Board Report, upon approval of the SALP Board, will be issued as FINAL and forwarded to the licensee.
- 3. You will note in the revised procedure that meetings with the licensee are no longer a requirement but are left to the discretion of the Regional Administrator or held specifically at the request of licensees. We plan to continue, however, to normally hold meetings with licensee management, as we have in the past.

- Included in the transmittal letter enclosing the SALP Board Report is a paragraph formally requesting that the licensee provide written comments on the SALP Board Report and the NRC findings included therein.
- 5. Once the report is issued to the licensee, the procedure to be followed for making significant corrections to the report becomes more involved than in past practice.

While the Errata Sheet continues as the primary means for making corrections to the report, a narrative basis is now to be included on the Errata Sheet to further explain the necessity for each correction made. A corrected page must then be prepared incorporating the changes, the original page included but lined-through referencing the Errata Sheet and all pages assembled for transmittal to the licensee as an attachment to the Appendix Package from the Regional Administrator. (The Appendix Package is defined in Item 6.)

The final stage of the SALP process involves the transmittal of 6. corrections made to the report, if appropriate, and an Appendix to the SALP Board Report which includes (a) a summarization of the meeting which was held with the licensee to discuss the report, (b) the verbatim written comments received from the licensee, and (c) presentation of all the conclusions reached by the Regional Administrator after thorough review of any such comments.

These changes, as well as others of a technical nature not discussed herein, will be covered formally in a Regional Procedure currently being prepared by the Technical Support Staff.

Charles & Moreline

Charles E. Norelius, Director Division of Project and Resident Programs

Attachment: Revised SALP Procedure



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON D. C. 20555

March 12, 1984

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MEMORANDUM FOR: Those on Attached List

FROM:

Richard C. DeYoung, Director

Office of Inspection and Enforcement

· SUBJECT:

PROPOSED REVISION TO NRC 0516 (SALP PROGRAM)

Comments on the proposed revision to NRC 0516, Systematic Assessment of Licensee Performance (SALP), have been received. Enclosure 1 provides a summary of the changes made based on the comments received. Enclosure 2 provides a copy of NRC 0516 with the specific changes made. Significant changes are indicated by lines in the right hand margin.

The enclosed procedure is being submitted to the Director, Office of Resource Management, for final review and—approval and is being forwarded to you as an advance copy. We believe the revision to NRC 0516 represents a significant improvement to the program, including some changes already in effect, and we are anxious to expedite formal issuance of the revised program. the revises program late effect as soon as possible but ant in interrupt Sal

currently in pregress

Richard C. DeYoung, Director Office of Inspection and Enforcement

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Enclosures:

Resolution of Comments to the Proposed Revision to NRC 0516 (SALP Program)

2. NRC 0516 (changes marked)

Contact: James P. Kearney (492-9669)

Steven D. Richardson (492-9612)

MAR 1 5 1984

Victor Stello, Deputy Executive Director for Regional Operations and Generic Requirements Office of the Executive Director for Operations

Harold R. Denton, Director Office of Nuclear Reactor Regulation

John G. Davis, Director
Office of Nuclear Material Safety and Safeguards

Clemens J. Heltemes, Director Office for Analysis and Evaluation of Operational Data

Thomas E. Murley, Regional Administrator, Region I

James P. O'Reilly, Regional Administrator, Region II

-Vames G. Keppler, Regional Administrator, Region III

John T. Collins, Regional Administrator, Region IV

John B. Martin, Regional Administrator, Region V

Resolution of Comments to the Proposed Revision to NRC 0516 (SALP Program)

- 1. Comment. Regions II and IV were opposed to conducting the SALP management meetings as open meetings.
 - Resolution. No change was made to the proposed program, which reflects decision of EDO as discussed in the previous Management Meeting.
- 2. Comment. Regions I, II, and III were opposed to having the Regional Administrator transmit the SALP Board Report to the licensee. The recommended alternative by the regions was to structure the SALP process to allow the licensee to respond to the report before the assessment results are made final. (ie. keep it the way it has been).

Resolution. No change was made to the proposed program, which reflects decision of EDO as discussed in previous Management Meeting.

- Comment. Regions II, III, and IV, and AEOD were opposed to the Table format proposed in Part VII. Exhibit 2, for documenting LER information.
 - Resolution. Changed the format of the Table used for documenting LER information.
- 4. Comment. Each Region felt the guidance provided on the timeliness of SALP Board Meetings and meetings with the licensee were too restrictive.
 - Resolution. Modified the guidance given for meetings with the licensee to 90 days to allow sufficient time for public notification of the meeting.
- 5. Comment. Regions I, III, and the EDO's office recommended that Table 1 (Evaluation Criteria) be modified.
 - Resolution. This was outside the scope of the proposed change. It is recommended that a task force, composed of a representative of each Region, IE, AEOD, and NRR, review the evaluation criteria for future changes in the SALP process.
- 6. Comment. Each Region had specific comments on the functional area definitions.
 - Resolution. The functional area definitions were modified to incorporate the majority of the specific comments.
- 7. Comment. NMSS wanted to include fuel facilities in the SALP
 - Resolution. No change was made to the proposed program because the budget will not support the manpower required.
- 8. Comment. Region II opposed the composition of the SALP Board. Specifically, the inclusion of non-management persons on the Board.
 - Resolution. No change was made to the proposed program.

Enclosure 2 + indicates change based on comments received

U.S. NUCLEAR REGULATORY COMMISSION

NRC MANUAL

Volume: 000

0000 General Administration

Part:

0500 Health and Safety

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CHAPTER 0516 SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE .

0516-01 COVERAGE

This Chapter and its appendix describe the basic structure and overall procedures for implementation of the NRC program to assess licensee performance. This program applies to all power reactors with operating licenses or construction permits (hereinafter referred to as licensees).

0516-02 OBJECTIVES

- 021 To improve the NRC regulatory program.
- 022 To permit sound decisions regarding NRC resource allocations.
- 023 To improve licensee performance.
- O24 To collect available observations on a periodic basis and evaluate licensee performance based on those observations, through the Systematic Assessment of Licensee Performance (SALP), an integrated NRC staff effort. Positive and negative attributes of licensee performance are considered. Emphasis is placed upon understanding the reasons for licensee's performance in important functional areas, and sharing this understanding with the licensee. The SALP process is oriented toward furthering NRC's understanding of the manner in which: (a) the licensee management directs, guides, and provides resources for assuring plant safety; and (b) such resources are used and applied. The integrated SALP assessment is intended to be sufficiently diagnostic to provide a rational basis for allocating NRC resources to provide meaningful guidance to licensee management.

C516-C3 RESPONSIBILITIES AND AUTHORITIES

O31 The Executive Director for Operations (EDO). Provides oversight for the activities described herein.

032 The Director, Office of Inspection and Enforcement (IE):

- a. Implements the requirements of this chapter within the Office of Inspection and Enforcement.
- b. Monitors the SALP process; evaluates and develops SALP policy, criteria, and methodology; and assesses the uniformity and adequacy of the regions' implementation of the program.
- D33 The Directors, Offices of Nuclear Reactor Regulation (NRR),
 Analysis and Evaluation of Operational Data (AEOD), and Nuclear
 Materials Safety and Safeguards (NM9S). Implement the requirements of this chapter within their Offices.

034 Regional Administrators

- a. Implements the requirements of this chapter within the Regions.
- b. Ensures that assessments of licensee nuclear safety performance are conducted.
- c. Determines when—a meeting with the licensee is necessary to assure mutual understanding of the issues discussed in the SALP Board report.
- d. Evaluates the SALP Board report and the licensee's comments; provides a characterization of overall safety performance; formally issues the NRC SALP report; follows up on licensee commitments; and reallocates region inspection resources as appropriate.
- e. Provides to the Director, Office of Inspection and Enforcement, recommendations for improvements to the SALP program and comments on proposed changes to SALP policy.

0516-04 EVALUATION CRITERIA AND FUNCTIONAL AREAS

O41 Evaluation. Licensees will be evaluated in the functional areas listed in section 042 using the criteria provided herein and further amplified in the Appendix to this Chapter. Each functional area evaluated will be assigned a Category as defined in Section 043 and a performance trend as defined in Section 044. Not all functional areas need be covered in a given review. If a functional area appropriate to a licensee is not covered, the reasons should be given in the report. The Appendix to this Chapter lists a number of attributes for each evaluation criterion. The functional area being evaluated may have some attributes that would place the evaluation in Category 1 and others that would place it in either Category 2 or 3. The final rating for each functional area will be a composite of the attributes tempered with judgment as to significance of individual items. Departures

from this guidance may sometimes be warranted. In such cases, the rationale for such departures should be explained in the report.

042 Functional Areas. A grouping of similar activities.

a. Operating Phase Reactors

1. Plant Operations

Consists chiefly of the activities of the licensee's operational staff (e.g., Picensed operators, shift technical advisors, and auxiliary operators). It is intended to be limited to operating activities such as plant startup, power operation, plant shutdown, and system linears. Thus, it includes activities such as reading and logging plant conditions; responding to off-normal conditions; manipulating the reactor and auxiliary controls; and training/retraining of licensed operators, shift technical advisors, and auxiliary operators.

2. Radiological Controls

Includes controls for occupational radiation protection; radioactive materials and contamination controls; radiological surveys and monitoring; processing of gaseous, liquid, and solid wastes; transportation of radioactive materials; radiological effluent and environmental monitoring; and the results of the NRC's independent measurement program.

3. Maintenance

Includes all activities associated with preventive or corrective maintenance of instrumentation and control equipment and mechanical and electrical systems.

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4. Surveillance

Includes all surveillance testing activities as well as all inservice inspection and testing activities. Examples of activities included are: instrument calibrations, equipment operability tests, containment leak rate tests, special tests, inservice inspection and performance tests of pumps and valves, and—all other inservice inspection activities.

5. Fire Protection

Includes routine housekeeping and fire protection/ prevention program activities. Thus, it includes the storage of combustible material; fire brigade staffing and training; fire suppression system maintenance and operation; and those fire protection features provided for structures, systems, and components important to safe shutdown.

6. Emergency Preparedness

Includes activities relating to the implementation of the emergency plan and implementing procedures. Thus, it includes such activities as licensee's performance during exercises which test the licensee, state, and local emergency plans; plan administration and implementation; notification; communications; facilities and equipment; staffing; training; assessment; emergency classification; medical treatment; radiological exposure control; recovery; protective actions; and interfaces with ensite and offsite emergency response organizations.

7. Security

Includes all activities whose purpose is to ensure the security and continued operability of the plant. Specifically it includes all aspects of the licensee's security program (e.g. access control, security checks, badging).

8. Refueling

Includes all activities associated with refueling. Thus, it includes outage management, and the manipulation of new and spent fuel.

Quality Programs and Administrative Controls Affecting Quality

Includes all verification and oversight activities which affect or assure the quality of plant activities, structures, systems and components. This area may be viewed as a comprehensive management system for controlling the quality of work performed as well as the quality of verification activities that confirm that the work was performed correctly. The evaluation of the effectiveness of the quality assurance system should be based on the results of management actions to ensure that necessary people, procedures, facilities and materials are provided and used during the operation of the nuclear power plant. Principal emphasis should be given to evaluating the effectiveness and involvement of management in establishing and assuring the effective implementation of the quality assurance program along with evaluating the history of licensee performance in the key areas of: committee activities, design and procurement control, control of design

change processes, inspections, audits, corrective action systems, and records.

10. Licensing Activities

Includes the adequacy and timeliness of all licensing submittals, responsiveness to NRC licensing initiatives, and the licensee's approach to resolution of technical issues from a safety standpoint.

11. Others (As Needed) :

b. Construction Phase Reactors

1. Soils and Foundation

Includes all soil and foundation activities related to the construction of the ultimate heat sink and safety-related structures. Specifically, this covers, as applicable, subgrade investigation and preparation, fill materials and compaction, embankments, foundations and associated laboratory testing, instrumentation and monitoring-systems.

Containment, Safety-Related Structures, and Major Steel Supports

Includes all activities related to the structural concrete and steel used in the containment (including the basemat) and safety-related structures, and major steel equipment supports. It includes all aspects of structural concrete (e.g., reinforcing steel; concrete batching, delivery, placement, in-process testing, and curing; liner plate erection and fabrication; and containment post-tensioning), structural steel used in safety-related structures (welded and bolted), and major steel equipment supports (for reactor vessel, reactor coolant pumps, steam generators, pressurizer, polar crane, tanks, heat exchangers, etc.).

Piping Systems and Supports

Includes those safety-related piping systems described in 10 CFR 50.2(v) and R.G. 1.26, quality groups A. B and C. It is intended to be limited to the primary pressure boundary and other safety-related water, steam and radioactive waste containment piping systems. It includes those quality checks necessary to ensure compliance with the applicable codes and other requirements specified in the SAR for these systems. The primary inspection emphasis in this area is on piping systems and their supports/restraints.

4. Safety-Related Components - Mechanical

Covers mechanical components such as pressure vessels, pumps, and valves located in, and attached to, the piping systems described in 3 above. The primary emphasis here is on components rather than piping.

5. Auxiliary Systems

Includes those safety-related auxiliary systems included in the nuclear facility which are essential for the safe shutdown of the plant or the protection of the health and safety of the public. Included here are systems such as HVAC, radwaste, fire protection and fuel storage and handling.

6. Electrical Equipment and Cables

Includes safety-related electrical components, cables and associated items used in the electrical systems of the plant, such as: motors, transformers, batteries, emergency diesel generators, motor control centers, switchgear, electric raceways, cable (power, control, and instrument), circuit breakers, relays and other interrupting and protective devices.

7. Instrumentation

Covers safety-related instrument components and systems that are designed to measure, transmit, display, record and/or control various plant variables and conditions. The Reactor Protection System and the Engineered Safety Features Actuation System are two plant systems utilizing such devices as: sensors, transmitters, signal conditioners, controllers and other actuating devices, recorders, alarms, logic devices, instrument air supplies, racks and panels.

8. Quality Programs and Administrative Controls Affecting

Includes all verification and oversight activities which affect or assure the quality of plant structures, systems and components. This area may be viewed as a comprehensive management system for controlling the quality of work performed as well as the quality of verification activities that confirm that the work was performed correctly. The evaluation of the effectiveness of the quality assurance system should be based on the results of management actions to ensure that necessary people, procedures, facilities and materials are provided and used during the design and construction of the nuclear power plant. Principal emphasis should be

given to evaluating the effectiveness and involvement of management in establishing and assuring the effective implementation of the quality assurance program along with evaluating the history of licensee/contractor performance in the key areas of: quality assurance program, design and procurement control, control of construction processes, inspections, audits, corrective action systems, and records.

9. Licensing Activities

Includes all activities sapporting the NRC review of the application for and the issuance of the Construction Permit and Operating License, and amendments thereto. In addition, it includes the adequacy and timeliness of all licensing submittals, responsiveness to NRC licensing initiatives, and the applicant's or licensee's approach to resolution of technical issues from a safety standpoint.

10. Others (As needed)

c. Preoperational Phase Reactors

Preoperational Testing

Covers the preparation, conduct, and evaluation of test results for preoperational tests performed by or under the direction of the dicense's staff to demonstrate the proper functioning and conformance to design requirements of components, systems, and structures.

2. Others (As Needed)

For reactors in the preoperational phase, functional areas from the listing for either operating phase reactors or construction phase reactors should be selected as appropriate...

d. Startup Phase Reactors

1. Startup Testing

Covers the preparation, conduct, and evaluation of test results for testing conducted following the issuance of the operating license. It starts with initial fuel loading and precritical tests, and continues until the plant reaches commercial operating status at or near its licensed power rating.

2. Others (As Needed)

For reactors in the startup phase, functional areas

from the listing for operating phase reactors should be used.

O43 Performance Categories. A rating of licensee performance in a given functional area.

a. Category 1

Reduced NRC attention may be appropriate. Licensee management attention and involvement are aggressive and oriented toward nuclear safety; licensee resources are ample and effectively used so that a high level of performance with respect to operational safety or construction is being achieved.

b. Category 2

NRC attention should be maintained at normal levels. Licensee management attention and involvement are evident and are concerned with nuclear safety; licensee resources are adequate and reasonably effective so that satisfactory performance with respect to operational safety or construction is being achieved.

c. Category 3

Both NRC and licensee attention should be increased. Licensee management attention or involvement is acceptable and considers nuclear safety, but weaknesses are evident; licensee resources appear to be strained or not effectively used so that minimally satisfactory performance with respect to operational safety or construction is being achieved.

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Trend. The general or prevailing tendency (the performance gradient) over the course of the SALP assessment period. The determination of the trend should not be forced. In those instances where a prevailing trend can not be determined, the SALP Board Report should state that the trend was not determined, and the reason for its omission should be provided (e.g., insufficient data).

a. Improved

Licensee performance has generally improved over the course of the SALP assessment period.

b. Same

Licensee performance has remained essentially constant over the course of the SALP assessment period.

c. Declined

Licensee performance has generally declined over the course

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of the SALP assessment period.

- O45 Evaluation Criteria. Elements which must be considered when assessing a licensee's performance in a functional area.
 - a. The evaluation criteria are as follows:
 - 1. Management involvement in assuring quality
 - Approach to resolution of technical issues from safety standpoint.
 - 3. Responsiveness to NRC initiatives
 - 4. Enforcement history
 - 5. Reporting and analysis of reportable events
 - Staffing (including management)
 - 7. Training effectiveness and qualification
 - b. Guidance for using these criteria to arrive at a category assignment is found in the Appendix to this Chapter.
- 0516-05 BASIC REQUIREMENTS
 - OS1 Applicability. This Chapter applies to and shall be followed by NRC Heacquarters Offices and Regional Offices:
 - D52 Appendix O516. Procedures for implementation of these directives are presented in the Appendix to this Chapter.

END

APPENDIX

SYSTEMATIC ASSESSMENT OF LECENSEE PERFORMANCE

UNITED STATES NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT

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PART I

GENERAL

- A. The SALP program was established to improve the NRC Regulatory Program, to permit sound decisions regarding NRC resource allocations, and to better understand the reasons for the performance of each reactor licensee.
- B. The NRC will conduct a review and evaluation of each power reactor licensee possessing an operating license or construction permit every 18 months except:
 - When the Regional Administrator determines that a particular utility or facility should be evaluated more frequently; or
 - When a SALP Report will be used as part of an evaluation of readiness for license issuance (IE 94300), a SALP evaluation should be scheduled approximately six to nine months before the scheduled licensing date.

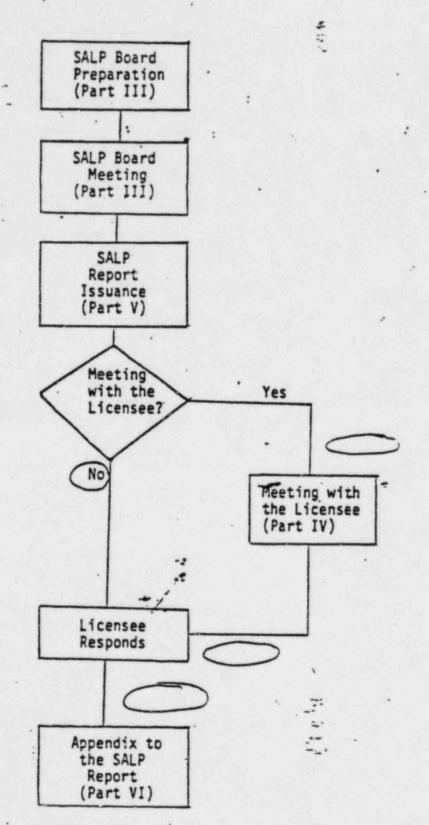
The individual facility assessments are intended to take place at an approximately uniform rate throughout the year within each Regional office.

- C. The evaluation process is composed of (Figure 1, Part I):
 - 1. A SALP Board assessment;
 - 2. Issuance of the SALP report;
 - If requested by the licensee or if otherwise determined to be necessary by the Regional Administrator, a meeting with licensee management to discuss the assessment; and
 - 4. Consideration of any written comments received from the licensee and issuance of an appendix to the SALP report which is to include the verbatim written comments received from the licensee and the conclusions of the Regional Administrator on the basis of his consideration of the comments.

Procedures for implementing the SALP program are provided in this Appendix.

*

FIGURE 1
SALP Evaluation Process .



PART II

EVALUATION CRITERIA

The assessment of licensee performance is implemented through the use of seven evaluation criteria. The criteria provide standard guidance that shall be applied to each functional area for the categorization of licensee performance.

To provide consistent evaluation of licensee performance, several attributes associated with each criterion are listed to describe the characteristics applicable to the three categories.

The seven criteria discussed in Chapter NRC-0516-045 are listed in Table 1 with their associated attributes. These form the guidance which aids in understanding and evaluating licensee performance by identifying the causes and factors appropriate for categorization. It is not intended that consideration of these attributes influence established programs of the agency. For example, it is not intended that specific inspections be performed to evaluate attributes. It is expected that during the implementation of established programs, many of the attributes which describe performance will be observed. Cognizance of these attributes should assist the staff in their observation of licensee performance during routine activities.

All of the attributes of the evaluation criteria are not necessarily applicable. In some instances, the observed performance within a functional area may be insufficient to allow consideration in the evaluation. Conversely, additional attributes may be appropriate for the evaluation. Matters such as management involvement and training are criteria of each functional area and should be considered in the evaluation of the functional areas. On the other hand, if there is a problem with one of these criterion that is observed in several functional areas, it may be desirable to highlight that criterion in a separate discussion; e.g., training may be a problem in Plant Operations, Radiological Centrol and Surveillance. It would be appropriate to discuss training as if it were a functional area, in addition to covering the specific training problem in each functional area.

It is emphasized that all available information should be analyzed by the SALP Board, and its significance, whether it be positive or negative, should be weighed. If information is scarce or nonexistent, a decision regarding the performance category or trend as it relates to an attribute should not be forced.

Category

Category 2

Category 3

1. Management Involvement and Control in Assuring Quality

consistent evidence of prior planning and assignment of priorities; well stated, controlled and explicit procedures for control of activities

evidence of prior planning and assignment of priorities; stated, defined procedures for control of activities

little evidence of prior planning and assignment of priorities; poorly stated or ill understood procedures for control of activities

well stated, disseminated, and understandable policies

adequately stated and understood policies

poorly stated, poorly understood or nonexistent policies.

decisionmaking consistently at a level that ensures adequate management review

decisionmaking usually at a level that ensures adequate management review decisionmaking seldom at a level that ensures adequate management review

corporate management frequently involved in site activities

corporate management usually involved in site activities

corporate management seldom involved in site activities

audits complete, timely, and thorough

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audits generally complete, timely, and thorough audits frequently not timely, incomplete, or not thorough

NRC APPENDIX 0516 PART II

Management Involvement and Control in Assuring Quality (Continued)

committees properly staffed and functioning in almost all cases

committees usually properly staffed and functioning

committees not properly staffed or functioning

reviews timely, thorough, and technically sound

reviews generally timely, reviews not timely, thorough, and technically sound or technically sound

reviews not timely, thorough,

records complete, well. maintained, and available

records generally complete, irecords not complete, not well well maintained, and available maintained, or unavailable

procedures and policies strictly adhered to

procedures and policies rarely violated

procedures and policies occasionally violated

corrective action systems promptly and consistently recognize and address nonreportable concerns

corrective action systems generally recognize and address nonreportable concerns

corrective action systems rarely recognize and address nonreportable concerns.

procurement well controlled and documented

procurement generally well controlled and documented

repetitive breakdown in procurement control

d'i polar design well controlled and verified

rare breakdowns of minor significance in design control

repetitive breakdown in design control or verification

SYSTEMATIC ASSESSMENT I

Approach to Resolution of Technical Issues from a Safety Standpoint

clear understanding of issues demonstrated .

understanding of issues generally apparent

understanding of issues : . frequently lacking

conservatism routinely . exhibited when potential for safety significance exists

conservatism generally: exhibited

meets minimum requirements

technically sound and thorough approaches in almost all cases

viable and generally sound and often viable approaches; but 'thorough approaches

lacking in thoroughness or depth

timely resolutions in almost all cases

generally timely resolutions

resolutions often delayed

Responsiveness to NRC Initiatives

meets deadlines

generally timely responses

frequently requires extensions of Lime

timely resolution of issues

few longstanding regulatory issues attributable to licensee

longstanding regulatory vissues attributable to licensee

3. Responsiveness to NRC Initiatives (Continued)

. .

technically sound and thorough responses in almost all, cases

viable and generally sound and thorough responses

often viable responses, but lacking in thoroughness or depth

acceptable resolutions proposed initially in most cases

acceptable resolutions generally proposed

considerable MRC e ort or repeated submittals needed to obtain acceptable resolutions

4. Enforcement History

major violations are rare and are not indicative of programmatic breakdown

minor violations are not repetitive and not indicative of programmatic breakdown

corrective action is prompt and effective

major violations are rare and may indicate minor programmatic breakdown

multiple minor violations or minor programmatic breakdown indicated

corrective action is timely and effective in most cases

multiple major violations or programmatic breakdown indicated

minor violations are repetitive and indicative of programmatic breakdown

corrective action is delayed or not effective

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SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE events are promptly and 4. completely reported

events are reported in a timely manner, some information may be . late or incomplete lacking

event reporting is frequently

events are properly identified and analyzed

events are accurately identified, some analyses are marginal

events are poorly identified or analyses are marginal, events are associated with programmatic weaknesses

corrective action is effective, as indicated by lack of repetition

corrective action is usually. taken but may not be effective. as indicated by occasional repetition

corrective action is not timely . nor effective, events are repetitive

Staffing (Including Management)

positions are identified. authorities and responsibilities are well defined

key positions are identified. . and authorities and responsibilities are defined

positions are poorly identified, or authorities and responsibilities are illdefined

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SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE

6. Staffing (Including Management)

vacant key positions are filled on priority basis

key positions usually filled in a reasonable time

key positions are left vacant for extended periods of time

staffing is ample as indicated by control over backlog and overtime staffing is adequate, occasional difficulties with backlog or overtime

staffing is weak or minimal as indicated by excessive backlog and overtime.

7. Training and Qualification Effectiveness

training and qualification program makes a positive contribution, commensurate with procedures and staffing, to understanding of work and adherence to procedures with few personnel errors

training and qualification program contributes to an adequate understanding of work and fair adherence to procedure with a modest number of personnel errors

training and qualification program is found to be the major contributing factor to poor understanding of work, as indicated by numerous procedure violations or personnel errors

training program is well defined and implemented with dedicated resources and a means for feedback experience; program is applied to nearly all staff

a defined program is implemented for a large portion of the staff

program may be either lacking, poorly defined, or ineffectively applied for a significant segment of the staff

PART III

SALP BOARD ASSESSMENT

A. SALP Board Preparation

- 1. Each Region shall issue a memorandum establishing the assessment period, SALP Board input due date, SALP Board meeting date, and date of licensee meeting, if necessary, for all facilities within the Region for all SALP meetings to occur in the calendar year.

 This memorandum shall be sent to NRR, IE, NMSS, AEOD, and the EDO by the end of the preceding calendar year. SALP board members will be notified promptly of any unavoidable changes.
- Prepare the SALP Board Report in a preliminary form.
 - a. Obtain SALP Board inputs. NRR, IE, AEOD, and NMSS shall provide a written input. If the Office does not have sufficient basis for an input, the Office shall state that fact to the Region by memorandum.
 - b. Prepare the Supporting Data Summary section of the report. (See Part VII, Exhibit 2 for format.)
 - c. Prepare a performance analysis for each of the functional areas identified in NRC 0516-042. (See Part VII, Exhibit 2 for format.)

B. SALP Board Meeting

- The SALP Board meeting should be conducted within 45 days after the end of the assessment period.
- The SALP Board shall be composed of:
 - a. SALP Board Chairman (Branch Chief or above)
 - b. NRR Project Manager
 - c. NRR management representative (participation is further described in NRR Office Letter 44)
 - d. Senior Resident Inspector
 - e. Representatives from IE, AEOD, and NMSS as appropriate
 - f. Other individuals as determined by the Regional Administrator

- 3. During the SALP Board meeting, the SALP Board shall:
 - Ensure that sufficient information has been provided in each functional area analysis to form a conclusion regarding licensee performance or alternatively confirm that sufficient information is not available to support a conclusion regarding licensee performance.
 - b. Rate licensee performance in each functional area after considering the evaluation criteria with their associated attributes listed in Table 1 of Part II of this appendix. Tables 2 and 3 may be used by the SALP Board members to assist them in their rating of a licensee.
 - c. Recommend revisions to the inspection program, as necessary.

TABLE 2

EVALUATION MATRIX FOR OPERATING

111 14

PHASE FUNCTIONAL AREAS

Licensing Activities	Quality Programs and Administrative Controls	Refueling	Security	Emergency Preparedness	Fire Protection	Surveillance	Kaintenance	Radiological Controls	Plant Operations	
				Ŀ						Management Involvement in Assuring, Quality
						•				Approach to Resolution of Technical Issues from a Safety Standpoint
						44				Responsiveness to NRC Initiatives
										Enforcement History
										Reporting and Analysis of Reportable Events
		1'								Staffing (including Management)
										Training and Qualification Effectiveness

TABLE 3

EVALUATION MATRIX FOR CONSTRUCTION

1 15 to

PHASE FUNCTIONAL AREAS

Licensing Activities	Qualify Assurance and Administrative Controls	Instruments and Control Systems	Electrical Power Supply and Distribution	Support Systems	Safety-Related Components	Piping Systems and Supports	Containment and Other Safety-Kelated Structures	Spils and Foundation	
									Management Involvement in . Assuring Quality
	1								Approach to Resolution of Technical Issues from a
					44			·	Responsiveness to NRC
									Enforcement History
							-1		Reporting and Analysis of Reportable Events
									Staffing (including Management)
							1		Training and Qualification Effectiveness

PART IV

MEETING WITH LICENSEE

A. General. If requested by the licensee or if otherwise determined to be necessary by the Regional Administrator, a meeting with licensee management to discuss the assessment will be held.

B. Meeting Preparation

- Notification of the meeting (if held) should be made by the Region at least one month in advance. Notification should be made to the licensee, the resident inspectors at the involved facilities, the NRR Project Managers for the involved facilities, and cognizant NRC managers.
- The licensee should be encouraged to have the following management representatives participate in the meeting.
 - a. Senior corporate management representative
 - b. Management officials responsible for the major functions wherein problem areas have been identified (e.g., health physics, security, engineering)
 - c. Site Manager

C. Meeting with Licensee

- The meeting (if held) should be conducted within 90 days after the end of the assessment period.
- 2. . NRC representatives for this meeting should include the following:
 - a. Either the Regional Administrator, Deputy Administrator, or Division Director
 - Responsible Regional Division Director(s), Branch Chiefs, or Section Chiefs, as appropriate
 - c. NRR Project Manager and designated NRR manager
 - d. Resident Inspector and/or assigned inspectors
 - e. Public Affairs Officer, when media interest is anticipated
- 3. The Regional Administrator, Deputy Administrator, or Division Director will chair the meeting and discussions of the adequacy of the licensee's management controls. These meetings are intended to provide a forum for candid discussion of issues relating to the licensee's performance. Those aspects of the licensee's operation that need improvement will be identified.

The licensee will also be given the opportunity to make comments on the report in writing within 30 days after the meeting or receipt of the SALP Board report if no meeting is held. Only written comments from the licensee must be addressed by the Regional Administrators.

4. SALP management meetings with the licensee should be conducted as open meetings, with the exception of those portions of the meetings that involve discussion of matters not required to be mandatorily placed in the public domain pursuant to 10 CFR 2.790 which must be closed. Members of the public should be treated as observers. Adequate notification of the SALP meeting can be accomplished by PDR distribution of the letter to the licensee which schedules the meeting, with copies to the service list for the appropriate docket.

PART V

ISSUANCE OF REPORT

A. Issuance of Report

The SALP Board Report (Exhibit 2, Part VII) shall be transmitted by the Regional Administrator to the licensee with copies to NRR and IE. The transmittal letter should include:

- A request for licensee's written comments and amplification, as appropriate, on these comments within 30 days after the meeting (if held) or receipt of the SALP Board report;
- Amplification of the findings of the SALP Board as appropriate.
 This includes, as a minimum, functional areas rated Category One, Category Three and those functional areas which have declined since the last SALP evaluation period (examples are shown in Exhibit 1, Part VII); and
- 3. A characterization of overall safety performance.

This letter, enclosing the SALP Report, will receive standard docket distribution including the NRC Public Document Room and the local Public Document Room, and INPO (Record Center, INPO: 1100 Circle 75)

Parkway: Suite 1500; Atlanta, GA 30339). Each report will be assigned an Inspection Report number.

B. Changing the SALP Report

Any changes made to the report as originally transmitted to the licensee shall be done using the following procedure (an example is shown in Exhibits 3, 4, and 5, Part VII).

- a. Include an errata sheet (Exhibit 3,:Part VII) as a separate enclosure to a Regional Administrator's cover letter denoting the change and the basis for the change.
- b. Add the corrected page (Exhibit 5, Part VII) to the report, leaving the original page (Exhibit 4, Part VII) in the report.
- c. Make a diagonal line through the original page, referencing the Errata sheet.

PART VI

APPENDIX TO THE SALP REPORT

A. General

After receiving the licensee's written comments, the Regional Administrator shall issue an appendix to the SALP report within 30 days. This appendix will receive standard docket distribution including the NRC Public Document Room, the local Public Document Room and INPO.

B. Appendix to the SALP Report

The appendix to the SALP report shall consist of:

- The verbatim written comments received from the licensee;
- A summary of any meetings held with the licensee concerning the SALP report; and
- The conclusions of the <u>Regional Administrator</u> on the basis of his consideration of the licensee's comments.

PART VII

SALP REPORT

FORMAT AND CONTENT

A. General

The SALP Board report is considered to be a final report upon approval by the Board and dispatch to the licensee.

B. Multiple Facility Licensees

In cases such as Duke and Commonwealth Edison, the SALP package may address more than one site. However, each site must have a separate SALP Board Report (Exhibit 2, Part VII).

C. Report Format and Content

The SALP Board report shall be prepared in general conformance to the guidelines provided in Exhibit 2. The standard entries described in this Exhibit should be used to the extent possible.

EXHIBIT 1

Samples of Overall Safety Performance Chacterizations

Example 1

Overall, we find that your performance of licensed activities generally is acceptable and directed toward safe facility operation. In addition, your overall performance has showed only moderate improvement since the last SALP evaluation period. Your performance in the area of Plant Modifications with contractors having limited experience was found to be in need of increased management attention.

Example 2

In addition to the assessments and recommendations made by the SALP Board in the enclosed SALP Report, it is my view that your overall regulatory performance continued at a high level during the assessment period. It is evident that safe operation and compliance with regulatory requirements are priority considerations at your facility. I concur, however, with the SALP Board findings that management attention is required to correct problems in the area of Radiological Controls and the long standing problems associated with the existing perimeter alarm system.

Example 3

The overall performance of your facility was acceptable but exhibited a declining trend since the last SALP evaluation period. Resources were strained or not effectively used such that minimally satisfactory performance with respect to operational safety was achieved. The SALP Board identified weaknesses in the areas of plant operations, radiological controls, maintenance, security and safeguards, and the quality assurance program. Your performance in these areas will be closely monitored and discussed in the next SALP Board Assessment for your facility. A major strength was noted in the area of refueling.

Example 4

Overall, we found your performance acceptable and directed toward safe facility operation. In addition, we found your overall performance improved since the last SALP avaluation period. We found agressive management attention and a high level of performance in the following areas: Radiological Controls, Surveillance, Fire Protection and Housekeeping, Emergency Preparedness, and Refueling. Your performance in assuring that equipment and procedural changes and adequately controlled was found to need increased attention on your part and we will pay particular attention to this area during our subsequent inspections.

EXHIBIT 2 SALP BOARD REPORT

U.S. NUCLEAR REGULATORY COMMISSION REGION [region]*

SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE

[Inspection Report Number]

[Name of Licensee]

[Name of Facility]

[Assessment Period]

I. INTRODUCTION

The Systematic Assessment of Licensee Performance (SALP) program is an integrated NRC staff effort to collect available observations and data on a periodic basis and to evaluate licensee performance based upon this information. SALP is supplemental to normal regulatory processes used to ensure compliance to NRC rules and regulations. SALP is intended to be sufficiently diagnostic to provide a rational basis for allocating NRC resources and to provide meaningful guidance to the licensee's management to promote quality and safety of plant construction and operation.

A NRC SALP Board, composed of the staff members listed below, met on [date], to review the collection of performance observations and data to assess the licensee performance in accordance with the guidance in NRC Manual Chapter 0516, "Systematic Assessment of Licensee Performance." A summary of the guidance and evaluation criteria is provided in Section II of this report.

This report is the SALP Beard's assessment of the licensee's safety performance at [name of facility] for the period [date] through [date].

SALP Board for [name of facility]:

[List SALP Board Members]

II. CRITERIA

Licensee performance is assessed in selected functional areas, depending whether the facility is in a construction, preoperational, or operating phase. Each functional area normally represents areas significant to nuclear safety and the environment, and are normal programmatic areas. Some functional areas may not be assessed because or little or no licensee activities or lack of meaningful observations. Special areas may be added to highlight significant observations.

One or more of the following evaluation criteria were used to assess each functional area.

- 1. Management involvement and control in assuring quality
- 2. Approach to resolution of technical issues from a safety standpoint
- 3. Responsiveness to NRC initiatives
- 4. Enforcement history
- 5. Reporting and analysis of reportable events
- 6. Staffing (including management)
- 7. Training effectiveness and qualification

However, the SALP Board is not limited to these criteria and others may have been used where appropriate.

Based upon the SALP Board assessment each functional area evaluated is classified into one of three performance categories. The definition of these performance categories is:

Category 1. Reduced NRC attention may be appropriate. Licensee management attention and involvement are aggressive and oriented toward nuclear safety; licensee resources are ample and effectively used so that a high level of performance with respect to operational safety or construction is being achieved.

Category 2. NRC attention should be maintained at normal levels. Licensee management attention and involvement are evident and are concerned with nuclear safety; licensee resources are adequate and are reasonably effective so that satisfactory performance with respect to operational safety or construction is being achieved.

Category 3. Both NRC and licensee attention should be fincreased. Licensee management attention or involvement is acceptable and considers nuclear safety, but weaknesses are evident; licensee resources appear to be strained or not effectively used so that minimally satisfactory performance with respect to operational safety or construction is being achieved.

III. SUMMARY OF RESULTS

[Provide a narrative summary of the licensee's overall significant strengths and weaknesses. It should be similar to the overall performance narrative used in the letter to the licensee.]

Functional Area	<pre>[last period]</pre>	[this period]	Trend
[functional area]	[rating lastperiod]	[rating this period]	[trend]

IV. PERFORMANCE ANALYSIS

A. [Functional Area being discussed]

1. Analysis

The analysis of the licensee's performance in an area should include pertinent facts and observations to highlight the specific strong and weak aspects of the licensee's performance. These facts and observations shall be presented in a manner to place matters in perspective and to allow the reader to understand the rationale for stated conclusions. This analysis should concentrate on the adequacy of the licensee's management control systems, adequacy of resources, training of personnel, etc., and the effectiveness of these efforts. Upon presentation of the analyses, the attributes associated with the specified criteria are to be referred to for purposes of both completeness and to compare the conclusions reached with the attributes of each category. The attributes listed in Part II are specifically oriented toward this

Category 3. Both NRC and licensee attention should be increased. Licensee management attention or involvement is acceptable and considers nuclear safety, but weaknesses are evident; licensee resources appear to be strained or not effectively used so that minimally satisfactory performance with respect to operational safety or construction is being achieved.

III. SUMMARY OF RESULTS

[Provide a narrative summary of the licensee's overall significant strengths and weaknesses. It should be similar to the overall performance narrative used in the letter to the licensee.]

Functional Area	[last period]	[this period]	Trend
[functional area]	[rating lastperiod]	[rating this period]	[trend]

IV. PERFORMANCE ANALYSIS

A. [Functional Area being discussed]

1. Analysis

[The analysis of the licensee's performance in an area should include pertinent facts and observations to highlight the specific strong and weak aspects of the licensee's performance. These facts and observations shall be presented in a manner to place matters in perspective and to allow the reader to understand the rationale for stated conclusions. This analysis should concentrate on the adequacy of the licensee's management control systems, adequacy of resources, training of personnel, etc., and the effectiveness of these efforts. Upon presentation of the analyses, the attributes associated with the specified criteria are to be referred to for purposes of both completeness and to compare the conclusions reached with the attributes of each category. The attributes listed in Part II are specifically oriented toward this

and should be utilized. In no event, however, are the examples of licensee performance for specific attributes to be used as stand-alone assessments; they represent a sampling of possible conclusions which must be supported by appropriate facts, observations or analysis. Each analysis should be written to avoid either 10 CFR 2.790 or safeguards information.

The analysis section is composed of three major subsections:

- . A brief account of the inspection activity which occurred in this area.
- A brief summary of the previous year's evaluation if there has been a significant change or if there should have been significant improvement but there was not.
- A summary of the strengths, weaknesses, and other significant observations made by the NRC staff during the evaluation period.]

2. Conclusion

[Provide the performance assessment (Category 1, 2, or 3) and trend (Improved, Same, Declined) for each functional area considered and if appropriate, a summary assessment.]

Board Recommendations

[Recommend NRC actions to be taken, if any are required. A basis for changes in the NRC program must be provided. Note that even in the absence of a recommendation to vary inspection levels, the Regional Office may do so based on the assessment as discussed in appropriate chapters of the IE manual.]

V. SUPPORTING DATA AND SUMMARIES

A. Licensee Activities

[Provide an outline of major licensee activities, such as major outages, power limitations, important licensee amendments and significant modifications.]

B. Inspection Activities

[Provide a summary of major inspection activities, such as major team inspections. Include Table I]

C. Investigations and Allegations Review

[Provide a summary of major investigative activities and their results.]

- D. Escalated Enforcement Actions
 - a. Civil Penalties [Provide a summary]
 - Orders (only those relating to enforcement) [Provide a summary]
- E. Management Conferences Held During Appraisal Period
 - Conferences [Discuss conferences that dealt with regulatory performance or enforcement.]
 - b. (Confirmation of Action Letters [Provide a summary]

[Other]

[Discuss any other issues at the discretion of the SALP Board.]

F. Review of Licensee Event Reports, Construction Deficiency Reports, and 10 CFR 21 Reports

[Provide a brief summary of significant findings and trends resulting from a review of these reports.]

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TABLE 1

INSPECTION ACTIVITY AND ENFORCEMENT

FUNCTIONAL AREA

INSP.

NO. OF VIOLATIONS IN EACH SEVERITY LEVEL

TOTAL

TABLE 3

LICENSEE EVENT REPORTS

NUMBER

DESCRIPTION

TABLE 4

10 CFR PART 21 REPORTS

NUMBER

DESCRIPTION

EXHIBIT 3

AN ERRATA SHEET

SALP BOARD REPORT ERRATA SHEET

PAGE	LINE	NOW READS	SHOULD READ
5	24	operator's cognitive decision	operator's decision

Basis: The word cognitive was deleted to avoid further problems in interpreting its meaning. As used the work was intended to mean that the operator as the cognizant individual on shift knew the operating requirements of the Technical Specification but made a conscious decision to operate the plant in a manner which he felt was equivalent to the requirements. It was not intended to mean that the operator took actions in total disregard of the Technical Specification objectives.

EXHIBIT 4

ORIGINAL PAGE

(10) Severity Level IV - Failure to take timely and proper corrective action following the failure of a cold leg RTD (50-000/81-24).

(N) Severity Level VI - Failure to make a 30 day report on a degraded bus voltage relay (50-000/81-26).

Six of the noncompliances were for failure to make required reports or to make simely reports, four for failure to follow procedures, and one for incomplete documentation. One noncompliance for failure to properly report a breach in containment, Item (9) above, is part of an escalated enforcement action with Civil Penalty. The actual event, is described in Section 4. Surveillance.

Nine LER's relating to this area were caused by personnel errors, six at Unit 1 and three at Unit 2. Sixty percent of these occurred in the last half of the period and thirty percent in the last quarter indicating an increasing occurrence rate in the period. Six of the nine were for incorrect valve or breaker alignments and three were for failure to follow operating procedures.

Two events (LER's 50-000/81-67 and 50-000/81-52) were of particular concern since they reflected a Prensed operator's cognitive decision to operate a system (charging and let wh and containment isolation, respectively) in a manner not allowed by the Technical Specifications.

Unit 1 experienced nine automatic trace evering the evaluation period, four caused by operator error and five the equipment failure. Of the four caused by errors, two were due to increedly conducted instrument surveillance tests, one to an incorrect year lineup on the steam side, and the last to unfamiliarity with turbine entrols.

Unit 2 experienced nine reactor trips one being manually initiated turbine trip. Four of the trips were related to resonnel errors; two by loss of vacuum in the main condenser, one resulted from a low steam generator level, and one resulted from a turbine value misalignment.

No significant safety concern is associated with these trips and each was reviewed to verify proper safety system operation and operator actions.

Various operating problems and events identified during the period resulted in an enforcement meeting on August 4, 1981, with followoo meeting on August 4, 1981, with followup meetings on November 2, 1981

EXHIBIT 5

CORRECTED PAGE

- (10) Severity Level IV Failure to take timely and proper corrective action following the failure of a cold leg RTD (50-000/81-24).
- (11) Severity Level VI Failure to make a 30 day report on a degraded bus voltage relay (50-000/81-26).

Six of the noncompliances were for failure to make required reports or to make timely reports, four for failure to follow procedures, and one for incomplete documentation. One noncompliance for failure to properly report a breach in containment, Item (9) above, is part of an escalated enforcement action with Civil Penalty. The actual event, is described in Section 4, Surveillance.

Nine LER's relating to this area were caused by personnel errors, six at Unit 1 and three at Unit 2. Sixty percent of these occurred in the last half of the period and thirty percent in the last quarter indicating an increasing occurrence rate in the period. Six of the nine were for incorrect valve or breaker alignments and three were for failure to follow operating procedures.

Two events (LER's 50-000/81-67 and 50-000/81-52) were of particular concern since they reflected a licensed operator's decision to operate a system (charging and letdown and containment isolation, respectively) in a manner not allowed by the Technical Specifications.

Unit 1 experienced nine automatic trips during the evaluation period, four caused by operator error and five by equipment failure. Of the four caused by errors, two were due to incorrectly conducted instrument surveillance tests, one to an incorrect valve lineup on the steam side, and the last to unfamiliarity with turbine controls.

Unit 2 experienced mine reactor trips, one being a manually initiated turbine trip. Four of the trips were related to personnel errors; two by loss of vacuum in the main condenser, one resulted from a low steam generator level, and one resulted from a turbine valve misalignment.

No significant safety concern is associated with these trips and each was reviewed to verify proper safety system operation and operator actions.

Various operating problems and events identified during the period resulted in an enforcement meeting on August 4, 1981, with followup meeting on August 4, 1981, with followup meetings on November 2, 1981