



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-42

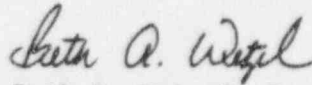
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1995, as supplemented August 9, and September 20, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 9, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

INSERT

TS iv

TS iv

TS xii

TS xii

TS 3.15-1

TS 3.15-1

TS 3.15-2

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Table 3.15-1

Table 3.15-1 (p. 1 of 2)

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Table 3.15-1 (p. 2 of 2)

Table 3.15-2

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Table 3.15-3

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Table 4.1-1C (p. 2 of 4)

Table 4.1-1C (p. 2 of 4)

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Bases 3.15-1

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	B. Power Distribution Limits	TS.3.10-1
	C. Quadrant Power Tilt Ratio	TS.3.10-4
	D. Rod Insertion Limits	TS.3.10-5
	E. Rod Misalignment Limitations	TS.3.10-6
	F. Inoperable Rod Position Indicator Channels	TS.3.10-6
	G. Control Rod Operability Limitations	TS.3.10-7
	H. Rod Drop Time	TS.3.10-7
	I. Monitor Inoperability Requirements	TS.3.10-8
	J. DNB Parameters	TS.3.10-8
3.11	Core Surveillance Instrumentation	TS.3.11-1
3.12	Snubbers	TS.3.12-1
3.13	Control Room Air Treatment System	TS.3.13-1
	A. Control Room Special Ventilation System	TS.3.13-1
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TECHNICAL SPECIFICATIONSLIST OF TABLES

<u>TS TABLE</u>	<u>TITLE</u>
1-1	Operational Modes
3.5-1	Engineered Safety Features Initiation Instrument Limiting Set Points
3.5-2A	Reactor Trip System Instrumentation
3.5-2B	Engineered Safety Feature Actuation System Instrumentation
3.9-1	Radioactive Liquid Effluent Monitoring Instrumentation
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4.12-1	Steam Generator Tube Inspection
4.13-1	Snubber Visual Inspection Interval
4.17-1	Radioactive Liquid Effluent Monitoring Instrumentation Surveillance Requirements
4.17-2	Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements
4.17-3	Radioactive Liquid Waste Sampling and Analysis Program
4.17-4	Radioactive Gaseous Waste Sampling and Analysis Program
5.5-1	Anticipated Annual Release of Radioactive Material in Liquid Effluents From Prairie Island Nuclear Generating Plant (Per Unit)
5.5-2	Anticipated Annual Release of Radioactive Nuclides in Gaseous Effluent From Prairie Island Nuclear Generating Plant (Per Unit)
6.1-1	Minimum Shift Crew Composition

Prairie Island Unit 1
 Prairie Island Unit 2

Amendment No. 111, 120, 121
 Amendment No. 10A, 11B, 114

3.15 EVENT MONITORING INSTRUMENTATION

Applicability

Applies during MODES 1, and 2.

Objective

To ensure that sufficient information is available to operators to determine the effects of and determine the course of an accident to the extent required to carry out required manual actions.

Specification

- A. Event monitoring instrumentation shall be OPERABLE as specified in Table TS.3.15-1.
- B. Action statements applicable by reference from Table TS.3.15-1.
- C. MODES 1, and 2 may be entered when a LIMITING CONDITION FOR OPERATION is not met.
- D. The provisions of specification 3.0.C are not applicable.

TABLE TS.3.15-1 (Page 1 of 2)
EVENT MONITORING INSTRUMENTATION

	<u>Function</u>	<u>Required Channels</u>	<u>Action^(*)</u>
1.	Power Range Neutron Flux (Logarithmic Scale)	2	1, 2
2.	Source Range Neutron Flux (Logarithmic Scale)	2	1, 2
3.	Reactor Coolant System (RCS) Hot Leg Temperature	2	1, 2
4.	RCS Cold Leg Temperature	2	1, 2
5.	RCS Pressure (Wide Range)	2	1, 2
6.	Reactor Vessel Water Level	2	1, 3
7.	Containment Sump Water Level (Wide Range)	2	1, 2
8.	Containment Pressure (Wide Range)	2	1, 2
9.	Automatic Containment Isolation Valve Position	2 per penetration flow path ^{(b)(c)}	1, 2
10.	Containment Area Radiation (High Range)	2	1, 3
11.	Hydrogen Monitors	2	1, 4
12.	Pressurizer Level	2	1, 2
13.	Steam Generator Water Level (Wide Range)	2 per steam generator	1, 2
14.	Condensate Storage Tank Level	2	1, 2
15.	Core Exit Thermocouples	4 per core quadrant	5, 6
16.	Refueling Water Storage Tank Level	2	1, 2

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 68, 78, 121
Amendment No. 62, 71, 114

Table TS.3.15-1
(Page 1 of 2)

TABLE TS.3.15-1 (Page 2 of 2)
EVENT MONITORING INSTRUMENTATION

(a) Action Statements

Separate Action Statement entry is allowed for each Function.

1. With one required channel inoperable, either restore the required channel to OPERABLE status within 30 days, or submit a report to the Commission within the following 14 days.
 2. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days or be in at least MODE 3 within the next 6 hours.
 3. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days, or submit a report to the Commission within the following 14 days.
 4. With two required channels inoperable, either restore one channel to OPERABLE status within 72 hours or be in at least MODE 3 within the next 6 hours.
 5. With the number of OPERABLE channels for the core exit thermocouples less than the Required Channels shown on Table TS.3.15-1, but with greater than or equal to 4 core exit thermocouples OPERABLE in the center core region and greater than or equal to one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 30 days, or submit a report to the Commission within the next 14 days. As a minimum, the Required Channels will be restored prior to startup following the next refueling outage.
 6. With the less than two core exit thermocouple channels OPERABLE in one or more quadrants, and with either less than 4 core exit thermocouples OPERABLE in the center region or less than one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 7 days, or be in at least MODE 3 within the next 6 hours.
- (b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, or blind flange.
- (c) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

TABLE TS.4.1-1C (Page 2 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHECK	CALIBRATE	FUNCTIONAL TEST	RESPONSE TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
13. Containment Sump A, B and C Level	N.A.	R	R	N.A.	1, 2, 3, 4
14. Deleted					
15. Turbine First Stage Pressure	S	R	Q	N.A.	1
16. Emergency Plan Radiation Instruments ⁽³⁵⁾	M	R	M	N.A.	1, 2, 3, 4, 5, 6
17. Seismic Monitors	R	R	N.A.	N.A.	1, 2, 3, 4, 5, 6
18. Coolant Flow - RTD Bypass Flowmeter	S	R	M	N.A.	1, 2, 3 ⁽³⁴⁾
19. CRDM Cooling Shroud Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3 ⁽³¹⁾ , 4 ⁽³¹⁾ , 5 ⁽³¹⁾
20. Reactor Gap Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3, 4
21. Post-Accident Monitoring Instruments (Table TS.3.15-1) ⁽³⁶⁾	M	R	N.A.	N.A.	1, 2
22. Deleted					

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 111, 116, 117
Amendment No. 104, 109, 110

TABLE TS.4.1-1C (Page 3 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
23. Deleted					
24. Steam Exclusion Actuation	W	Y	M	N.A.	1, 2, 3
25. Overpressure Mitigation	N.A.	R	R	N.A.	4 ⁽³⁸⁾ , 5
26. Auxiliary Feedwater Pump Suction Pressure	N.A.	R	R	N.A.	1, 2, 3,
27. Auxiliary Feedwater Pump Discharge Pressure	N.A.	R	R	N.A.	1, 2, 3
28. NaOH Caustic Stand Pipe Level	W	R	M	N.A.	1, 2, 3, 4
29. Hydrogen Monitors	S	Q	M	N.A.	1, 2
30. Containment Temperature Monitors	M	R	N.A.	N.A.	1, 2, 3, 4
31. Turbine Overspeed Protection Trip Channel	N.A.	R	M	N.A.	1

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 78, 111, 121
Amendment No. 88, 104, 114

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TABLE NOTATIONS

FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	Shift
D	Daily
W	Weekly
M	Monthly
Q	Quarterly
S/U	Prior to each startup
Y	Yearly
R	Each refueling shutdown
N.A.	Not Applicable

TABLE NOTATION

- | | |
|--|--|
| (30) Prior to each startup following shutdown in excess of two days if not done in previous 30 days. | (36) Except for containment hydrogen monitors and refueling water storage tank level which are separately specified in this table. |
| (31) When the reactor trip system breakers are closed and the control rod drive system is capable of rod withdrawal. | (37) When RHR is in operation. |
| (32) Following rod motion in excess of six inches when the computer is out of service. | (38) When the reactor coolant system average temperature is less than 310°F. |
| (33) Transfer logic to Refueling Water Storage Tank. | (39) Whenever CONTAINMENT INTEGRITY is required. |
| (34) When either main steam isolation valve is open. | |
| (35) Includes those instruments named in the emergency procedure. | |

3.15 EVENT MONITORING INSTRUMENTATIONBases

The OPERABILITY of the event monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, NUREG-0737 and the Regulatory Guide 1.97 evaluation of Prairie Island instrumentation.

The Action Statements for one or more inoperable channels in some circumstances require submittal of a report to the Commission. The report shall outline preplanned alternate method of monitoring as applicable, the cause of the inoperability, and the plans and schedule for restoring the instrument channels of the Function to OPERABLE status.

Some containment penetration flow paths may have one valve with position indication based on the justification that another mechanism is provided to assure containment integrity is maintained. In the event the position indication on the one valve fails, the justification for the penetration to have a single valve with position indication is assumed to remain valid. Since another mechanism continues to provide containment integrity, the applicable action is Action Statement 1.

The following core exit thermocouples are included in the center core region referenced in Table TS.3.15-1, Action Statement 5. If a thermocouple is not listed below, it is located in the outside core region.

<u>Thermocouple Number</u>	<u>Core Location</u>
9	D-5
10	D-7
12	E-4
13	E-6
14	E-10
16	F-7
18	G-4
19	G-6
22	H-5
23	H-9
28	I-4
29	I-8
30	I-10
32	J-6
33	J-8
34	J-9

Core exit thermocouple readings necessary to meet the requirements of Specification 3.15.A are available from the Plant Process Computer, the Control Room Core Exit Thermocouple Display or if no other readout is available, from test equipment readings from the Core Exit Thermocouple Junction Boxes.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. DPR-60

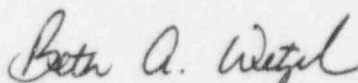
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1995, as supplemented August 9, and September 20, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 9, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 114

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
TS iv	TS iv
TS xii	TS xii
TS 3.15-1	TS 3.15-1
TS 3.15-2	--
Table 3.15-1	Table 3.15-1 (p. 1 of 2)
--	Table 3.15-1 (p. 2 of 2)
Table 3.15-2	--
Table 3.15-3	--
Table 4.1-1C (p. 2 of 4)	Table 4.1-1C (p. 2 of 4)
Table 4.1-1C (p. 3 of 4)	Table 4.1-1C (p. 3 of 4)
Table 4.1-1C (p. 4 of 4)	Table 4.1-1C (p. 4 of 4)
Bases 3.15-1	Bases 3.15-1

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	B. Power Distribution Limits	TS.3.10-1
	C. Quadrant Power Tilt Ratio	TS.3.10-4
	D. Rod Insertion Limits	TS.3.10-5
	E. Rod Misalignment Limitations	TS.3.10-6
	F. Inoperable Rod Position Indicator Channels	TS.3.10-6
	G. Control Rod Operability Limitations	TS.3.10-7
	H. Rod Drop Time	TS.3.10-7
	I. Monitor Inoperability Requirements	TS.3.10-8
	J. DNB Parameters	TS.3.10-8
3.11	Core Surveillance Instrumentation	TS.3.11-1
3.12	Snubbers	TS.3.12-1
3.13	Control Room Air Treatment System	TS.3.13-1
	A. Control Room Special Ventilation System	TS.3.13-1
3.14	Deleted	
3.15	Event Monitoring Instrumentation	TS.3.15-1

TECHNICAL SPECIFICATIONSLIST OF TABLES

<u>TS TABLE</u>	<u>TITLE</u>
1-1	Operational Modes
3.5-1	Engineered Safety Features Initiation Instrument Limiting Set Points
3.5-2A	Reactor Trip System Instrumentation
3.5-2B	Engineered Safety Feature Actuation System Instrumentation
3.9-1	Radioactive Liquid Effluent Monitoring Instrumentation
3.9-2	Radioactive Gaseous Effluent Monitoring Instrumentation
3.15-1	Event Monitoring Instrumentation
4.1-1A	Reactor Trip System Instrumentation Surveillance Requirements
4.1-1B	Engineered Safety Feature Actuation System Instrumentation Surveillance Requirements
4.1-1C	Miscellaneous Instrumentation Surveillance Requirements
4.1-2A	Minimum Frequencies for Equipment Tests
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4.10-3	RFMP - Reporting Levels for Radioactivity Concentrations in Environmental Samples
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4.17-2	Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements
4.17-3	Radioactive Liquid Waste Sampling and Analysis Program
4.17-4	Radioactive Gaseous Waste Sampling and Analysis Program
5.5-1	Anticipated Annual Release of Radioactive Material in Liquid Effluents From Prairie Island Nuclear Generating Plant (Per Unit)
5.5-2	Anticipated Annual Release of Radioactive Nuclides in Gaseous Effluent From Prairie Island Nuclear Generating Plant (Per Unit)
6.1-1	Minimum Shift Crew Composition

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 111, 120, 121
Amendment No. 10A, 113, 114

3.15 EVENT MONITORING INSTRUMENTATION

Applicability

Applies during MODES 1, and 2.

Objective

To ensure that sufficient information is available to operators to determine the effects of and determine the course of an accident to the extent required to carry out required manual actions.

Specification

- A. Event monitoring instrumentation shall be OPERABLE as specified in Table TS.3.15-1.
- B. Action statements applicable by reference from Table TS.3.15-1.
- C. MODES 1, and 2 may be entered when a LIMITING CONDITION FOR OPERATION is not met.
- D. The provisions of specification 3.0.C are not applicable.

TABLE TS.3.15-1 (Page 1 of 2)
EVENT MONITORING INSTRUMENTATION

	<u>Function</u>	<u>Required Channels</u>	<u>Action^(a)</u>
1.	Power Range Neutron Flux (Logarithmic Scale)	2	1, 2
2.	Source Range Neutron Flux (Logarithmic Scale)	2	1, 2
3.	Reactor Coolant System (RCS) Hot Leg Temperature	2	1, 2
4.	RCS Cold Leg Temperature	2	1, 2
5.	RCS Pressure (Wide Range)	2	1, 2
6.	Reactor Vessel Water Level	2	1, 3
7.	Containment Sump Water Level (Wide Range)	2	1, 2
8.	Containment Pressure (Wide Range)	2	1, 2
9.	Automatic Containment Isolation Valve Position	2 per penetration flow path ^{(b)(c)}	1, 2
10.	Containment Area Radiation (High Range)	2	1, 3
11.	Hydrogen Monitors	2	1, 4
12.	Pressurizer Level	2	1, 2
13.	Steam Generator Water Level (Wide Range)	2 per steam generator	1, 2
14.	Condensate Storage Tank Level	2	1, 2
15.	Core Exit Thermocouples	4 per core quadrant	5, 6
16.	Refueling Water Storage Tank Level	2	1, 2

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 68, 78, 121
Amendment No. 62, 71, 114

Table TS.3.15-1
(Page 1 of 2)

TABLE TS.3.15-1 (Page 2 of 2)
EVENT MONITORING INSTRUMENTATION

(a) Action Statements

Separate Action Statement entry is allowed for each Function.

1. With one required channel inoperable, either restore the required channel to OPERABLE status within 30 days, or submit a report to the Commission within the following 14 days.
 2. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days or be in at least MODE 3 within the next 6 hours.
 3. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days, or submit a report to the Commission within the following 14 days.
 4. With two required channels inoperable, either restore one channel to OPERABLE status within 72 hours or be in at least MODE 3 within the next 6 hours.
 5. With the number of OPERABLE channels for the core exit thermocouples less than the Required Channels shown on Table TS.3.15-1, but with greater than or equal to 4 core exit thermocouples OPERABLE in the center core region and greater than or equal to one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 30 days, or submit a report to the Commission within the next 14 days. As a minimum, the Required Channels will be restored prior to startup following the next refueling outage.
 6. With the less than two core exit thermocouple channels OPERABLE in one or more quadrants, and with either less than 4 core exit thermocouples OPERABLE in the center region or less than one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 7 days, or be in at least MODE 3 within the next 6 hours.
- (b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, or blind flange.
- (c) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Containment Sump A, B and C Level	N.A.	R	R	N.A.	1, 2, 3, 4
14. Deleted					
15. Turbine First Stage Pressure	S	R	Q	N.A.	1
16. Emergency Plan Radiation Instruments ⁽³⁵⁾	M	R	M	N.A.	1, 2, 3, 4, 5, 6
17. Seismic Monitors	R	R	N.A.	N.A.	1, 2, 3, 4, 5, 6
18. Coolant Flow - RTD Bypass Flowmeter	S	R	M	N.A.	1, 2, 3 ⁽³⁴⁾
19. CRDM Cooling Shroud Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3 ⁽³¹⁾ , 4 ⁽³¹⁾ , 5 ⁽³¹⁾
20. Reactor Gap Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3, 4
21. Post-Accident Monitoring Instruments (Table TS.3.15-1) ⁽³⁶⁾	M	R	N.A.	N.A.	1, 2
22. Deleted					

TABLE TS.4.1-1C (Page 3 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
23. Deleted					
24. Steam Exclusion Actuation	W	Y	M	N.A.	1, 2, 3
25. Overpressure Mitigation	N.A.	R	R	N.A.	4 ⁽³⁸⁾ , 5
26. Auxiliary Feedwater Pump Suction Pressure	N.A.	R	R	N.A.	1, 2, 3,
27. Auxiliary Feedwater Pump Discharge Pressure	N.A.	R	R	N.A.	1, 2, 3
28. NaOH Caustic Stand Pipe Level	W	R	M	N.A.	1, 2, 3, 4
29. Hydrogen Monitors	S	Q	M	N.A.	1, 2
30. Containment Temperature Monitors	M	R	N.A.	N.A.	1, 2, 3, 4
31. Turbine Overspeed Protection Trip Channel	N.A.	R	M	N.A.	1

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TABLE NOTATIONS

FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	Shift
D	Daily
W	Weekly
M	Monthly
Q	Quarterly
S/U	Prior to each startup
Y	Yearly
R	Each refueling shutdown
N.A.	Not Applicable

TABLE NOTATION

- | | |
|--|---|
| <p>(30) Prior to each startup following shutdown in excess of two days if not done in previous 30 days.</p> <p>(31) When the reactor trip system breakers are closed and the control rod drive system is capable of rod withdrawal.</p> <p>(32) Following rod motion in excess of six inches when the computer is out of service.</p> <p>(33) Transfer logic to Refueling Water Storage Tank.</p> <p>(34) When either main steam isolation valve is open.</p> <p>(35) Includes those instruments named in the emergency procedure.</p> | <p>(36) Except for containment hydrogen monitors and refueling water storage tank level which are separately specified in this table.</p> <p>(37) When RHR is in operation.</p> <p>(38) When the reactor coolant system average temperature is less than 310°F.</p> <p>(39) Whenever CONTAINMENT INTEGRITY is required.</p> |
|--|---|

3.15 EVENT MONITORING INSTRUMENTATIONBases

The OPERABILITY of the event monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, NUREG-0737 and the Regulatory Guide 1.97 evaluation of Prairie Island instrumentation.

The Action Statements for one or more inoperable channels in some circumstances require submittal of a report to the Commission. The report shall outline preplanned alternate method of monitoring as applicable, the cause of the inoperability, and the plans and schedule for restoring the instrument channels of the Function to OPERABLE status.

Some containment penetration flow paths may have one valve with position indication based on the justification that another mechanism is provided to assure containment integrity is maintained. In the event the position indication on the one valve fails, the justification for the penetration to have a single valve with position indication is assumed to remain valid. Since another mechanism continues to provide containment integrity, the applicable action is Action Statement 1.

The following core exit thermocouples are included in the center core region referenced in Table TS.3.15-1, Action Statement 5. If a thermocouple is not listed below, it is located in the outside core region.

<u>Thermocouple Number</u>	<u>Core Location</u>
9	D-5
10	D-7
12	E-4
13	E-6
14	E-10
16	F-7
18	G-4
19	G-6
22	H-5
23	H-9
28	I-4
29	I-8
30	I-10
32	J-6
33	J-8
34	J-9

Core exit thermocouple readings necessary to meet the requirements of Specification 3.15 A are available from the Plant Process Computer, the Control Room Core Exit Thermocouple Display or if no other readout is available, from test equipment readings from the Core Exit Thermocouple Junction Boxes.