

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

March 11, 1992

Docket No. 50-317

Mr. G. C. Creel Vice President - Nuclear Energy Baltimore Gas and Electric Company Calvert Cliffs Nuclear Power Plant MD Rts. 2 and 4 P. O. Box 1535 Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: WITHDRAWAL SCHEDULE CHANGE FOR REACTOR VESSEL MATERIAL SPECIMENS, CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M82686)

By letter dated January 24, 1992, Baltimore Gas and Electric Company (BG&E) requested Commission approval for a change to the withdrawal schedule for its reactor vessel materia; surveillance capsules which are located in the Unit 1 reactor. The requested charge was submitted pursuant to Appendix H of 10 CFR Part 50.

The specific request is to replace the reactor vessel material specimen capsule at the 104 degree location scheduled for withdrawal at the end of the second withdrawal interval, which is during the upcoming refueling outage in March of 1992, with the capsule at the 97 degree location currently scheduled to be withdrawn at the end of the fourth interval. BG&E also indicated that the capsule at the 263 degree location, not specifically required by the Appendix H program, would not be removed during the upcoming refueling outage.

The NRC staff has reviewed the BG&E request and concluded that BG&E has provided the necessary technical justification required by Section II.B.3 of Appendix H to 10 CFR Part 50 to support the proposed changes to the withdrawal schedules for reactor vessel material surveillance capsules in the Unit 1 reactor. The second withdrawal interval capsule to be withdrawn is at the 97 degree location and the 104 degree location capsule will be scheduled for withdrawal at the end of the fourth interval. In addition, the capsule at the 263 location will not be removed at the end of the second withdrawal interval

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in that the dosimetry in the 97 degree capsule location will provide the necessary in-vessel neutron dosimetry for benchmarking purposes. Details supporting this conclusion are included in the enclosed Safety Evaluation. These approved changes will be reflected in the next update of the Updated Final Safety Analysis Report.

Sincerely

Daniel G. McDonald, Senior Project Manager Project Dorectorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation

cc w/enclosure: See next page

Mr. G. C. Creel Baltimore Gas & Electric Company

CC:

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