



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

March 12, 1992

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Attn: Document Control Desk

- Subject: Quad Cities Nuclear Station Units 1 and 2
Resolution of Questions Pertaining
to Relief Request for
Inservice Inspection Program
NRC Docket Nos. 50-254 and 50-265
- Reference: (a) Conference call on March 10, 1992 between
Commonwealth Edison Company (J. Schrage, et al),
and NRR (D. Lynch, et al)
- (b) J.L. Schrage to T.E. Murley letter dated
September 30, 1991

Dear Dr. Murley:

During the referenced teleconference (Reference (a)), Commonwealth Edison Company (CECo) representatives provided information to NRR in response to questions concerning the accessibility of welds discussed in a Quad Cities Station Relief Request (Reference (b)). The purpose of this letter is to document the information presented during the teleconference.

The original Relief Request stated that approximately 60% of the Reactor Vessel Stabilizer Bracket weld surfaces were accessible. As such, a Section XI Class 1 Surface Examination could be performed on this portion of the weld surfaces. This examination would then be supplemented with a visual examination (VT-3/4).

Subsequent inspections of the equipment configuration during the current refuel outage (Q2R11) have indicated that approximately 78% of the vertical weld surface and 100% of the top horizontal weld surface are accessible to a visual examination. However, 100% of the bottom weld surface is inaccessible to a visual examination. This is due to the close proximity of the bottom weld surface to the top of the biological shield. In addition, the proximity of the Reactor Vessel Stabilizer Bracket to the Reactor Vessel insulation does not allow for Surface Examination of any portion

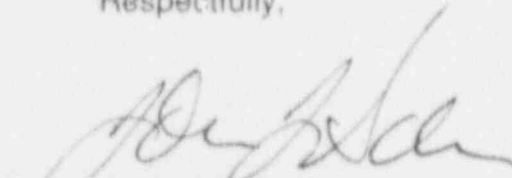
1047/10

of the weld surface. This insulation must be removed in order to perform a Surface Examination. Removal of the insulation would require the removal of screws holding two panels of insulation together. The screws holding the insulation panels together are located directly behind a bar running horizontally through the Reactor Vessel Stabilizer Brackets. The clearance between this bar and the insulation (approximately two inches) does not allow for expeditious removal of the screws. In addition, reinstallation of the screws is severely hindered by the clearances. This reinstallation is necessary to ensure the long term structural integrity tight the adjacent Reactor Vessel insulation panels.

In light of the revised accessibility of Reactor Vessel Stabilizer Bracket weld surfaces, CECO will perform a VT-1 Visual Examination of the accessible portions of the weld surfaces (approximately 78% of the vertical surfaces and 100% of the top horizontal surfaces).

Please direct any questions or comments to John L. Schrage at 708-515-7283.

Respectfully,



John L. Schrage
Nuclear Licensing Administrator

cc: A. Bert Davis, Regional Administrator-RIII
L.N. Olshan, NRR Project Manager-Quad Cities
T.E. Taylor, Senior Resident Inspector-Quad Cities
D. Lynch, NRR Project Manager-Quad Cities