

MAR 9 1992

Docket No. 50-458  
License No. NPF-47

Gulf States Utilities  
ATTN: James C. Deddens  
Senior Vice President (RBNG)  
P.O. Box 220  
St. Francisville, Louisiana 70775

Gentlemen:

This refers to the management meeting conducted at Region IV's request at the River Bend Station on March 4, 1992. This meeting related to activities authorized by NRC License NPF-47 for the River Bend Station and was attended by those on the attached attendance list.

The subjects discussed at the meeting are described in the enclosed Meeting Summary.

It is our opinion that this meeting was beneficial and provided a better understanding of your management controls to address shutdown risk for the refueling outage and provided a good summary of major outage activities. In accordance, with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter will be placed in the NRC's Public Document Room.

Should you have any questions concerning this matter, we will be pleased to discuss them with you.

Sincerely,

A. Bill Beach, Director  
Division of Reactor Projects

Enclosure:  
Meeting Summary w/attachments

cc w/enclosure:  
Gulf States Utilities  
ATTN: J. E. Booker, Manager-  
Nuclear Industry Relations  
P.O. Box 2951  
Beaumont, Texas 77704

RIV/PE:DRP  
EECollins;bh  
3/9/92

*Harrell*  
C:PSO  
PHHarrell  
3/9/92

*Collins*  
D:DRS  
SJCcollins  
3/9/92

D:DRP  
ABBeach  
3/9/92

*IE45*

9203160148 920309  
PDR ADOCK 05000458  
P PDR

Gulf States Utilities

-2-

Winston & Strawn  
ATTN: Mark J. Wetterhahn, Esq.  
1401 L Street, N.W.  
Washington, D.C. 20005-3502

Gulf States Utilities  
ATTN: Les England, Director  
Nuclear Licensing  
P.O. Box 220  
St. Francisville, Louisiana 70775

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3456 Villa Rose Drive  
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Baton Rouge, Louisiana 70895

Hall Bohlinger, Administrator  
Radiation Protection Division  
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Baton Rouge, Louisiana 70884-2135

bcc to DMB (IE45) - DRP

bcc distrib. by RIV:

R. D. Martin

DRP

Lisa Shea, RM/ALF

DRSS-RPEPS

Project Engineer (DRP/C)

Senior Resident Inspector, Fort Calhoun

Resident Inspector

Section Chief (DRP/C)

RIV File

Senior Resident Inspector, Cooper

DRS

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MEETING SUMMARY

Licensee: Gulf States Utilities  
Facility: River Bend Station  
License No.: NPF-47  
Docket No.: 50-458  
Subject: Meeting to Discuss Shutdown Risk Management During the  
Refueling Outage and Major Outage Activities

On March 4, 1992, representatives of Gulf States Utilities met with NRC personnel at the River Bend Station to discuss shutdown risk management during the refueling outage and major outage activities. The attendance list and licensee presentation are attached to this summary. The meeting was open to the public.

The licensee presented their outage schedule and the approach used to evaluate how multiple activities could contribute to shutdown risk. Also, the management controls of outage activities were reviewed. In addition, outage work involving the service water system, motor operated valve testing program, source term reduction, and feedwater nozzle repairs was summarized.

Attachments:

1. Attendance List
2. Licensee Presentation (NRC distribution only)



ATTENDANCE LIST

Attendance at the GSU/NRC management meeting on March 4, 1992, at the River Bend Station:

GSU

J. Donnelly, Chief Executive Officer/Chairman of the Board of Directors  
P. Graham, Plant Manager, River Bend  
K. Suhrke, General Manager Engineering and Administration  
J. Booker, Manager, Nuclear Industry Relations  
M. Sankovich, Manager, Engineering  
T. Crouse, Manager, Administration  
J. Pruitt, Manager, Business Systems  
W. Odell, Manager, Oversight  
E. Cargill, Director, Radiological Programs  
L. England, Director, Nuclear Licensing  
J. Shippert, Assistant Plant Manager, Operations, Radwaste, Chemistry  
P. Freehill, Assistant Plant Manager, Outage Management  
J. Leavines, Supervisor, NSAG  
L. Dietrich, Supervisor, Nuclear Licensing  
J. Burton, Supervisor, PRA  
D. Wells, Senior Licensing  
T. Brice, Engineer, Service Water Chemical Cleaning  
M. Stein, Engineer, Service Water Pipe Replacement  
T. Davey, Presenter, Service Water System  
R. Buell, Presenter, Motor Operated Valve Testing Program  
W. Beck, Presenter, Source Term Reduction  
T. Hoffman, Presenter, Nozzle Repairs  
G. Mohan, Presenter, Nozzle Repairs  
M. Dreher, Administrator, Municipal Affairs

NRC

S. Collins, Director, Division of Reactor Safety (DRS)  
B. Boyer, Director, Reactor Projects III, IV, V, Office of Nuclear Reactor Regulation (NRR)  
T. Westerman, Chief, Plant Systems Section, DRS  
E. Ford, Senior Resident Inspector, River Bend Station (RBS)  
D. Loveless, Resident Inspector, RBS  
E. Collins, Project Engineer, RBS

NUCLEAR SAFETY ADVISORY COMMITTEE

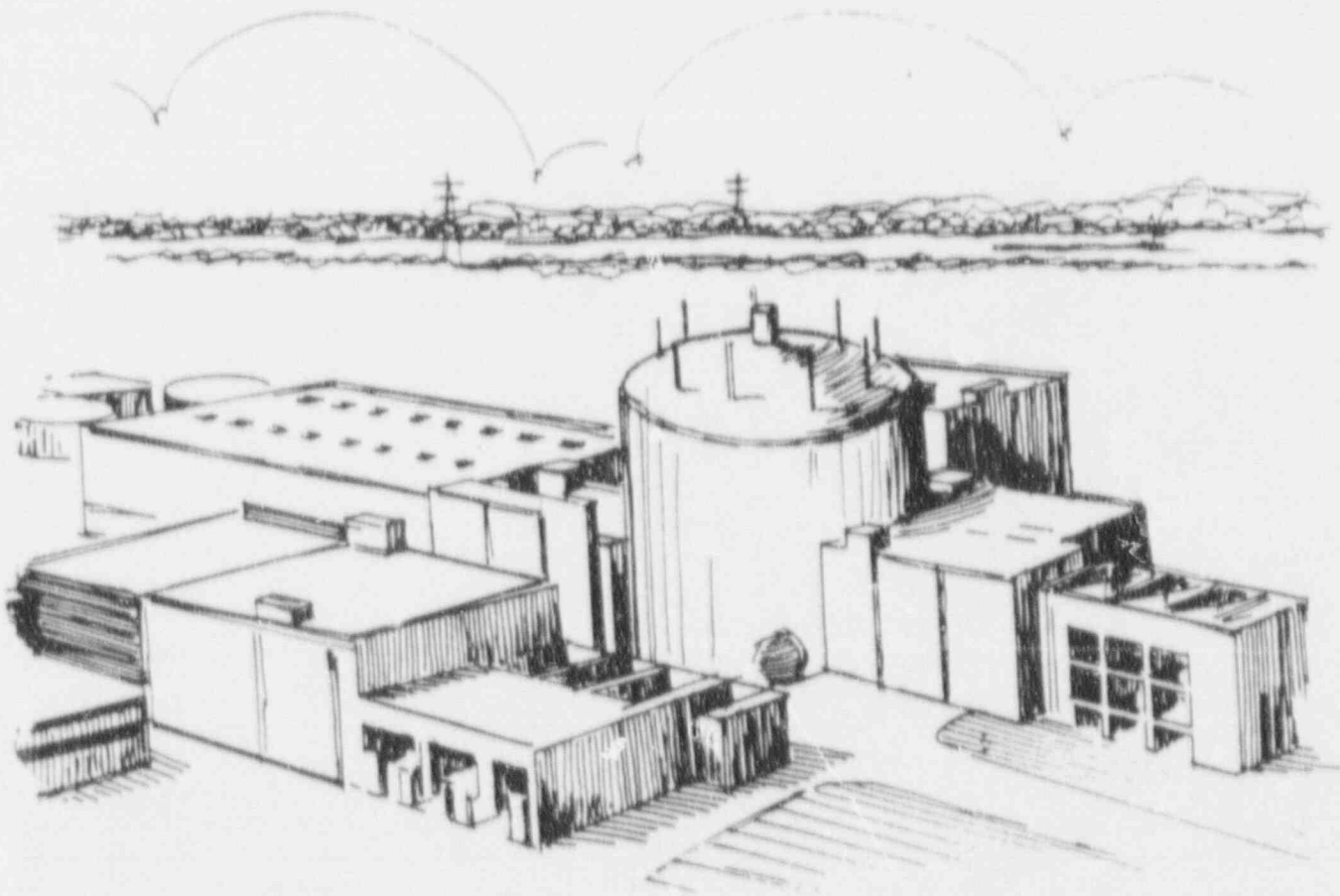
E. Lambremont, LSU Nuclear Science Center  
T. Pigford, University of California - Berkeley  
H. Woodson, Dean of Engineering, University of Texas at Austin  
J. Smith, Consultant

OTHER ATTENDEES

H. Anne Plettinger, Member of the Public  
J. Minton, Reporter, The Advocate  
P. Shinkle, Reporter, The Advocate  
W. Curran, Site Representative, Cajun Electric

# RIVER BEND STATION 4TH REFUELING OUTAGE PLANS

A PRESENTATION TO THE  
NUCLEAR REGULATORY COMMISSION  
MARCH 4, 1992



GULF STATES UTILITIES COMPANY

FINAL AGENDA  
 GSU/NRC MEETING: 4TH REFUELING OUTAGE PLANS  
 MARCH 4, 1992  
 RIVER BEND STATION

9:30-9:40	OPENING ANNOUNCEMENTS	(10 MIN)	LES ENGLAND
9:40-9:55	OUTAGE CONTROL	(15 MIN)	PHIL GRAHAM
	A. CONTRACTOR WORK CONTROL		
	B. SECURITY		
	C. RADIATION PROTECTION		
9:55-10:15	RF4 SCOPE AND SCHEDULE	(20 MIN)	PETE FREEHILL
	A. MAJOR EVOLUTIONS		
	B. OTHER WORK		
10:15-10:35	OUTAGE RISK ASSESSMENT	(20 MIN)	JOE LEAVINES
10:35-10:55	OUTAGE RISK MANAGEMENT	(20 MIN)	JOE BURTON
	A. GSU INITIATIVES		
10:55-11:15	OPERATIONS OVERVIEW- RF4	(20 MIN)	JOE SCHIPPERT
	A. RISK MANAGEMENT		
	B. CLEARANCES		
11:15-11:30	ENGINEERING PROJECT CONTROL	(15 MIN)	MEL SANKOVICH
11:30-11:50	BREAK FOR LUNCH	(PROVIDED)	
11:50-1:05	SERVICE WATER PROGRAM		
	A. INTRODUCTION	(10 MIN)	KEN SUHRKE
	B. CHEMICAL CLEANING	(25 MIN)	TAMMY BRICE
	C. PIPE REPAIR/REPLACEMENT	(15 MIN)	MIKE STEIN
	D. CLOSED LOOP MODIFICATION	(25 MIN)	TOM DAVEY
1:05-1:20	MOTOR OPERATED VALVE TESTING PROGRAM	(15 MIN)	RICH BUELL
1:20-1:40	SOURCE TERM REDUCTION	(20 MIN)	WALLY BECK
1:40-2:10	N4A NOZZLE REPAIRS		
	A. ENGINEERING	(10 MIN)	TOM HOFFMAN
	B. INSTALLATION	(20 MIN)	GARLAND MAHAN
2:10-2:25	OPEN DISCUSSION		ALL
2:25-2:30	CLGSING REMARKS		PHIL GRAHAM NRC

RIVER BEND STATION  
REFUELING FOUR  
OUTAGE CONTROL

- 0 Contractor Management & Control
- 0 Integrated Organization
- 0 Facilitators
- 0 Training
- 0 Maintenance Management Field Observations
- 0 Using More Long Term Contractors

# RISK MANAGEMENT

- 0 Standards and Expectations
- 0 Power Supplies and Switchyard Controls
- 0 Decay Heat Removal
- 0 Water Supply
- 0 Containment Control

# OUTAGE MANAGEMENT

- 0 APM Shift Outage Managers
- 0 Enhanced Risk Managed L-II Schedule
- 0 Computerized Tag System
- 0 Bar Code Work Package/Tool Tracking
- 0 Drywell Coordinators



# MINIMIZE ESF ACTUATIONS

- 0 Revised MR Planning Process
- 0 Goal < 20 LER's
- 0 goal < 3 ESF Actuations
- 0 STAR Program

# RADIATION PROTECTION

- 0 Hot Machine Shop Decon Personnel
- 0 R.P. Technician Assignment to Planning
- 0 Foreman Assignment to Training
- 0 Computer Generated RWP's
- 0 Noble Gas Release Procedure
- 0 Local Plant Postings
- 0 Surrogate Tour
- 0 Chemical Decon
- 0 Plant Decon
- 0 RWCU Ring Header Replacement
- 0 Camera Equipment
- 0 Alarming Dosimetry

# ALARA

< 500 Person Rem

< 200 Contamination

## SECURITY CONTROL

- 0 Revised Visitor Escort Training
- 0 Expanded Protected Area
- 0 Changes to PAP Processing

# RIVER BEND STATION REFUELING FOUR

- 0 Start March 15, 1992
- 0 Duration 156 Days
- 0 End August 13, 1992
- 0 Risk Management
- 0 Scope
- 0 Schedule

## SERVICE WATER TASKS

- 0 Pipe & Valve Replacement
- 0 Chemical Cleaning
- 0 Closed Loop System Preop



1992					
MAR	APR	MAY	JUN	JUL	AUG

12MAR92 D=72  15MAR92  
 CONTROLLED RX SHUTDOWN  
 15MAR92  15MAR92  
 RX SHUTDOWN & GENERATOR OUTPUT BREAKER OPEN  
 15MAR92 D=48  17MAR92  
 NOZZLE FLUSH/LPDS FLUSH  
 15MAR92 D=24  16MAR92  
 DRAIN RX CAVITY  
 15MAR92 D=24  17MAR92  
 REMOVE DRYWELL HEAD  
 17MAR92 D=48  19MAR92  
 REMOVE RX HEAD & DRYER (MODE 5 @ 20 FT.)  
 19MAR92 D=40  20MAR92  
 INSTALL STM LINE PLUGS, CATTLE CHUTE, REMOVE  
 SEPARATOR AND FILL CAVITY  
 (MODE 5 @ 20 FT.)  
 20MAR92 D=40  24MAR92  
 DIV 2 SSW AS FOUND LLRT'S  
 SHUTDOWN DIV II NSW & CWS  
 21MAR92 D=72  26MAR92  
 RUMP DOWN DIV 2 SSW, NSW & CWS  
 26MAR92  23APR92  
 DIV 2 SSW PPS MODE, CLOSED LOOP TIE-IN, LLRT  
 REPAIR SYSTEM, OUTAGE WORK (SSW & CWS)  
 21APR92 D=40  26APR92  
 REFILL WITH CLARIFIED WATER, STARTUP TEMP SYS  
 26APR92  10MAY92  
 CHEMICAL CLEAN & SYSTEM FLUSH DIV 2 & NSW  
 10MAY92 D=200  20MAY92  
 LLRT'S & SYSTEM RESTORATION DIV II SSW  
 20MAY92 D=100  20MAY92  
 REFILL & HYDRO SYSTEM  
 20MAY92 D=100  25MAY92  
 REFILL & HYDRO SYSTEM  
 25MAY92 D=100  30MAY92  
 VALVE LINEUP & SYSTEM STARTUP DIV 2 SSW & NSW  
 30MAY92 D=60  31JUN92  
 PREP CLOSED LOOP SYSTEM  
 31JUN92 D=48  5JUN92  
 DIVISIONAL SWAP, INCLUDES DIESEL TESTING  
 5 HR TEST RUN  
 1 HR OPERABILITY RUN  
 5JUN92 D=48  9JUN92  
 DIV 1 SSW AS FOUND LLRT'S  
 8JUN92 D=48  10JUN92  
 RUMP DOWN DIV 1 SSW  
 10JUN92 D=240  22JUN92  
 DIV 1 & 2 PPS MODE, LLRT REPAIR  
 SYSTEM OUTAGE WORK (SSW & CWS)  
 22JUN92 D=60  25JUN92  
 REFILL WITH CLARIFIED WTR, STARTUP TEMP SYS  
 22JUN92 D=280  7JUL92  
 CHEM CLEAN & SYSTEM FLUSH DIV 1 SSW  
 7JUL92 D=200  17JUL92  
 LLRT'S & SYSTEM RESTORATION DIV I SSW  
 17JUL92 D=100  22JUL92  
 REFILL & HYDRO DIV 1 SSW  
 22JUL92 D=100  27JUL92  
 VALVE LINEUP & SYSTEM STARTUP DIV I SSW  
 27JUL92 D=24  28JUL92  
 DIESEL AVAILABILITY TESTING  
 5 HR TEST RUN  
 1 HR OPERABILITY  
 28JUL92  28JUL92  
 MODE 5 @ 25 FT.  
 28JUL92 D=114  2AUG92  
 DRAIN CAVITY & RPV REASSEMBLY  
 DECON RX CAVITY AS NECESSARY  
 2AUG92  2AUG92  
 MODE 4  
 2AUG92 D=60  4AUG92  
 RPV 10 YR HYDRO & 10% STRAM TIMING  
 PERFORM CSP-3100-14 DUR=2 HRS  
 4AUG92 D=48  6AUG92  
 DRYWELL HEAD INSTALLATION & FILL CAVITY  
 6AUG92 D=48  6AUG92  
 FILL RX CAVITY FROM CST VIA SFC AT 600 GPM  
 (EST 215K GAL)  
 6AUG92 D=120  11AUG92  
 DRYWELL BYPASS TEST / ILRT  
 11AUG92 D=148  17AUG92  
 MODE 2 CHECKLISTS  
 DIVISION REPORTS  
 LICENSE COMMITMENTS  
 CLOSE LOGS  
 17AUG92  17AUG92  
 MODE 2  
 17AUG92 D=24  18AUG92  
 PULL RODS / DRYWELL WALKDOWN @ 900 PSI  
 18AUG92  18AUG92  
 TIC GENERATOR TO THE GRID

## MAJOR TASKS

- 0 Reactor Refueling
- 0 Turbine Generator Low Pressure Inspection
- 0 Source Term Reduction
  - RWCU & Recirc Chemical Cleaning
  - RWCU Ring Header Replacement
- 0 Recirculation Pump & Discharge Valve Inspections
- 0 Suppression Pool Cleanup
- 0 RPV Feedwater Nozzle Safe End Replacement

## REPETITIVE TASKS

- 0 Diesel Generator Inspection
- 0 Division I Bus Outages
- 0 Safety Relief Valve Replacement
- 0 CRD Rebuild
- 0 LLRT & ILRT
- 0 MOV Signature Tests
- 0 Surveillance Tests
- 0 Check Valve Operability Tests
- 0 Pipe Erosion Ultra Sonic Tests
- 0 Pipe & Reactor Inservice Inspections

## REACTOR SCHEDULE

- 0 Shutdown
- 0 RPV Disassembly
- 0 Mode 5
- 0 Off Load
- 0 Bottom Head Drain Cleaning
  - 0 RCS Pump & Motor PM
  - 0 N4A Safe End Replace
- 0 Reload
- 0 12 Days Float
- 0 RPV, DW BYP & ILRT, Mode 2  
Checklist & Startup

# SCHEDULE FEATURES

- 0 Availability v.s. Unavailability Based Schedule
  - 0 Containment
  - 0 Off Site Power
  - 0 Diesel Generator
  - 0 Shutdown Cooling - RPV
  - 0 Decay Heat Removal - Pools
  - 0 ECCS
  - 0 Level Control - RPV
- 0 Fuel Off Load
  - 0 Simplified Cooling Requirements
  - 0 Eliminate RPV Potential to Drain
  - 0 Simplified Reactivity & Core Alteration control

## INITIATIVES

- 0 Standards & Expectations
- 0 RF-3 Critique
- 0 Schedule Change Control-MPLD
- 0 Bus Outage Procedures
- 0 NRC Notice 91-22
- 0 Numarc 91-06
- 0 INPO - Draft 'INPO Shutdown Evaluation Guidance'
- 0 Contingency Plans



## STANDARDS and EXPECTATIONS

- 0 Off-site & Emergency Power
- 0 Decay Heat Removal & Makeup Capability
- 0 Containment Control
- 0 Fuel Handling
- 0 Operations Schedule Reviews
- 0 Critical Schedule Reviews
- 0 Schedule Change Control
- 0 Periodic Schedule Reviews
- 0 Availability Based Schedule
- 0 Outage Window Timing
- 0 Electrical Work Plan Reviews
- 0 Summary L-II Schedule

## OVERVIEW OF ASSESSMENT

- USED INPO AND NUMARC GUIDELINES
  
- CONDUCTED ASSESSMENT WITH TEAM OF ISEG ENGINEERS
  - ALL SRO LICENSED OR CERTIFIED
  - ALL EXPERIENCED IN RBS PREVIOUS OUTAGES
  - ALL SENIOR LEVEL PERSONNEL
  
- REVIEWED LEVEL II SCHEDULE VERSUS THE ELEMENTS OF THE GUIDELINES
  
- PRODUCED SUMMARY SCHEDULE FOR KEY FUNCTIONS AND HIGHER RISK EVOLUTIONS

## HIGHER RISK EVOLUTIONS

- WATER MOVEMENT (Upper Pools, RHR, Suppression Pool)
- LOWERED D/G OR ECCS AVAIL.
- FUEL MOVEMENT
- SWITCHYARD WORK
- CONTAINMENT NOT SET
- FREEZE SEALS
- COMBINATIONS OF THE ABOVE

## KEY SAFETY FUNCTIONS

- DECAY HEAT REMOVAL (RHR & SFC)
- MAKEUP TO VESSEL AND POOLS (ECCS)
- POWER AVAILABILITY (OFFSITE AND D/G)
- CONTAINMENT (ABILITY TO RESTORE)
- FUEL HANDLING/CRITICALITY

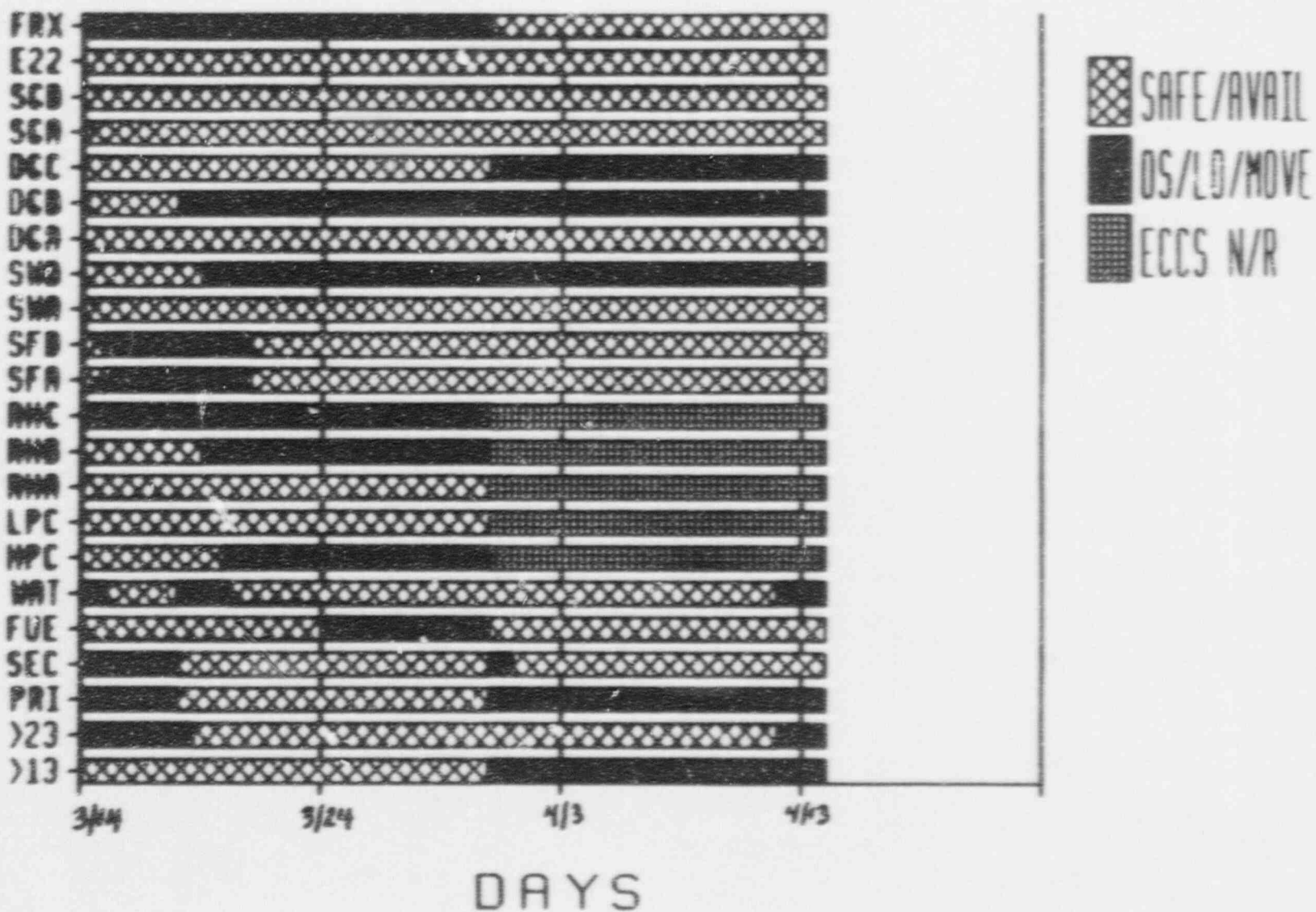
## CHANGES AS A RESULT OF REVIEW

- RECOMMENDED DEVELOPMENT OF POLICY STATEMENT REGARDING OUTAGE RISK MANAGEMENT. PLANT MANAGER ISSUED STATEMENT INCORPORATING APPROPRIATE ELEMENTS OF INPO GUIDELINES.
- RECOMMENDED POSITIVE CONTROL OF SWITCHYARD ACTIVITIES. POSITIVE COMMUNICATION AND CONTROL OF T&D ACTIVITIES ESTABLISHED BY OPERATIONS DEPT. SYSTEM WILL MONITOR.
- RECOMMENDED CONTINGENCY PLAN FOR LOSS OF SFC WHEN CORE OFFLOADED. BACKUP D/G & FPW WATER.
- RECOMMENDED CONTINGENCY PLAN FOR SETTING CONTAINMENT. OPS DEVELOPING PLANS WITH CHECKLISTS ON STATUS OF PENETRATIONS SO THEY CAN BE CLEARED AND CLOSED.
- RECOMMENDED DEVELOPMENT OF LOSS OF DHR CONTINGENCY PLAN. AOP UNDER DEVELOPMENT.
- CONTROL ACCESS TO AND ENHANCE SURVEILLANCE OF D/G WHEN ONLY ONE OPERABLE.
- ORIGINAL LII HAD "B" AND "C" DG OUTAGE WHILE FUEL IS IN VESSEL. "C" DG WAS MOVED TO CORRESPOND TO END OF FUEL MOVEMENT. "B" OPERABILITY EXTENDED MAXIMIZE AVAILABILITY.

# RF4 SUMMARY SCHEDULE

3/15 TO 4/14

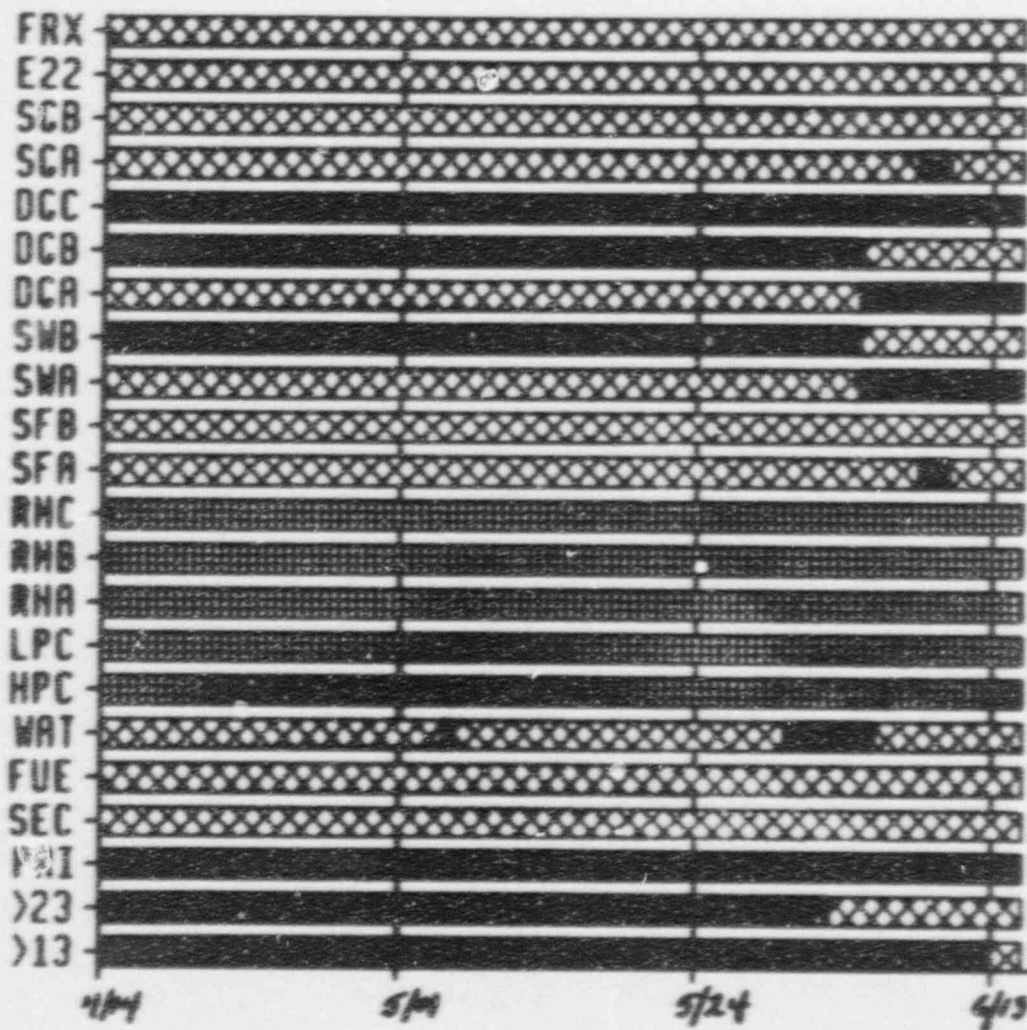
ACTIVITIES



# RF4 SUMMARY SCHEDULE

4/15 TO 6/15

ACTIVITIES



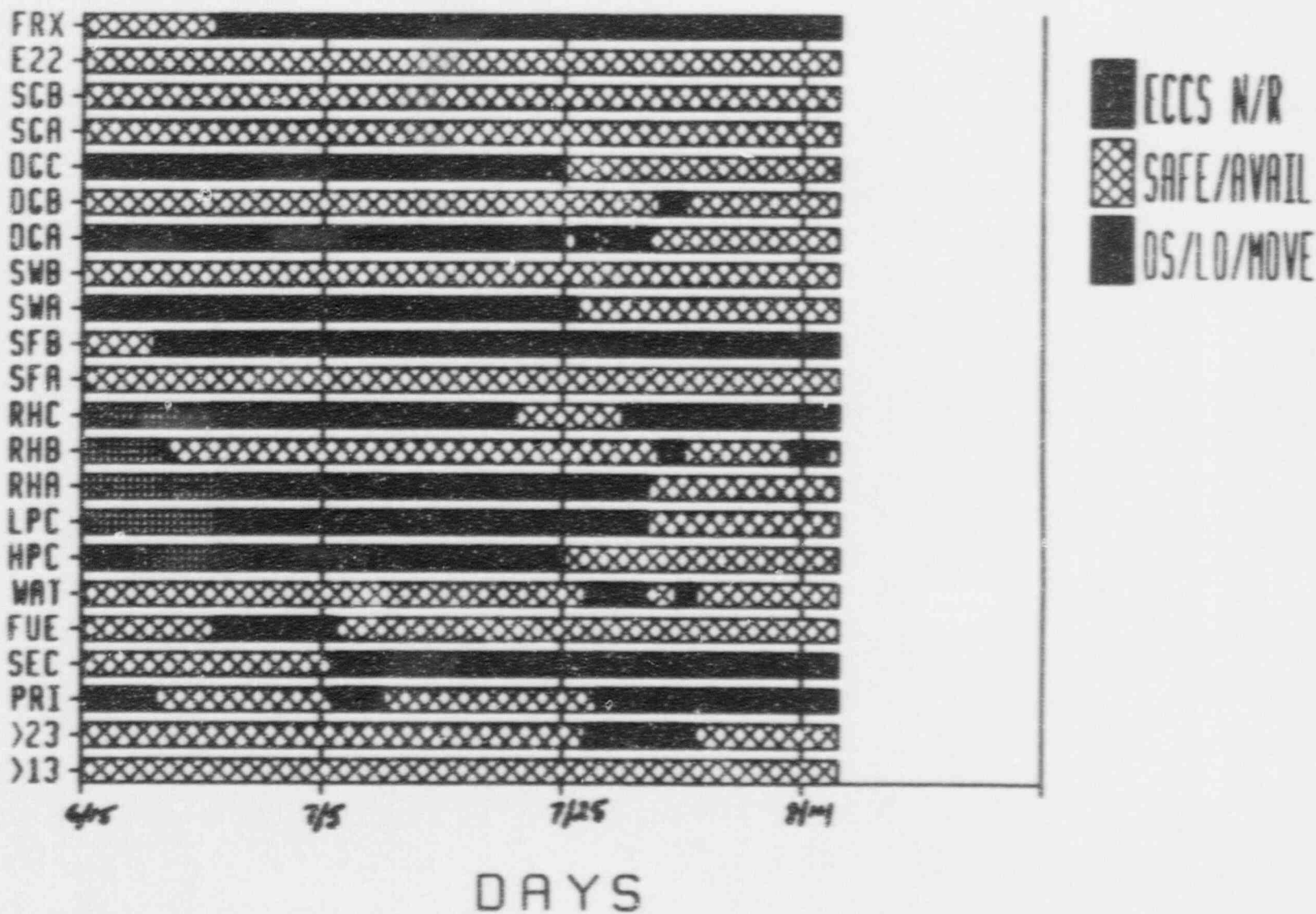
DAYS



# RF4 SUMMARY SCHEDULE

6/16 TO 8/17

ACTIVITIES



- ECCS N/R
- SAFE/AVAIL
- OS/LD/MOVE



## CONTINUING EFFORT - NSAG

- NSAG WILL HAVE ON SHIFT COVERAGE OF THE OUTAGE.
- WILL REVIEW CHANGES TO LEVEL II IN THE SAME WAY WE DID THE ORIGINAL ASSESSMENT.
- WILL INDEPENDENTLY MONITOR STATUS OF KEY SAFETY SYSTEMS.

# OUTAGE RISK MANAGEMENT

J. L. Burton  
Supervisor of PRA/Radiological Analysis

River Bend Station  
Gulf States Utilities Company  
St. Francisville, La.

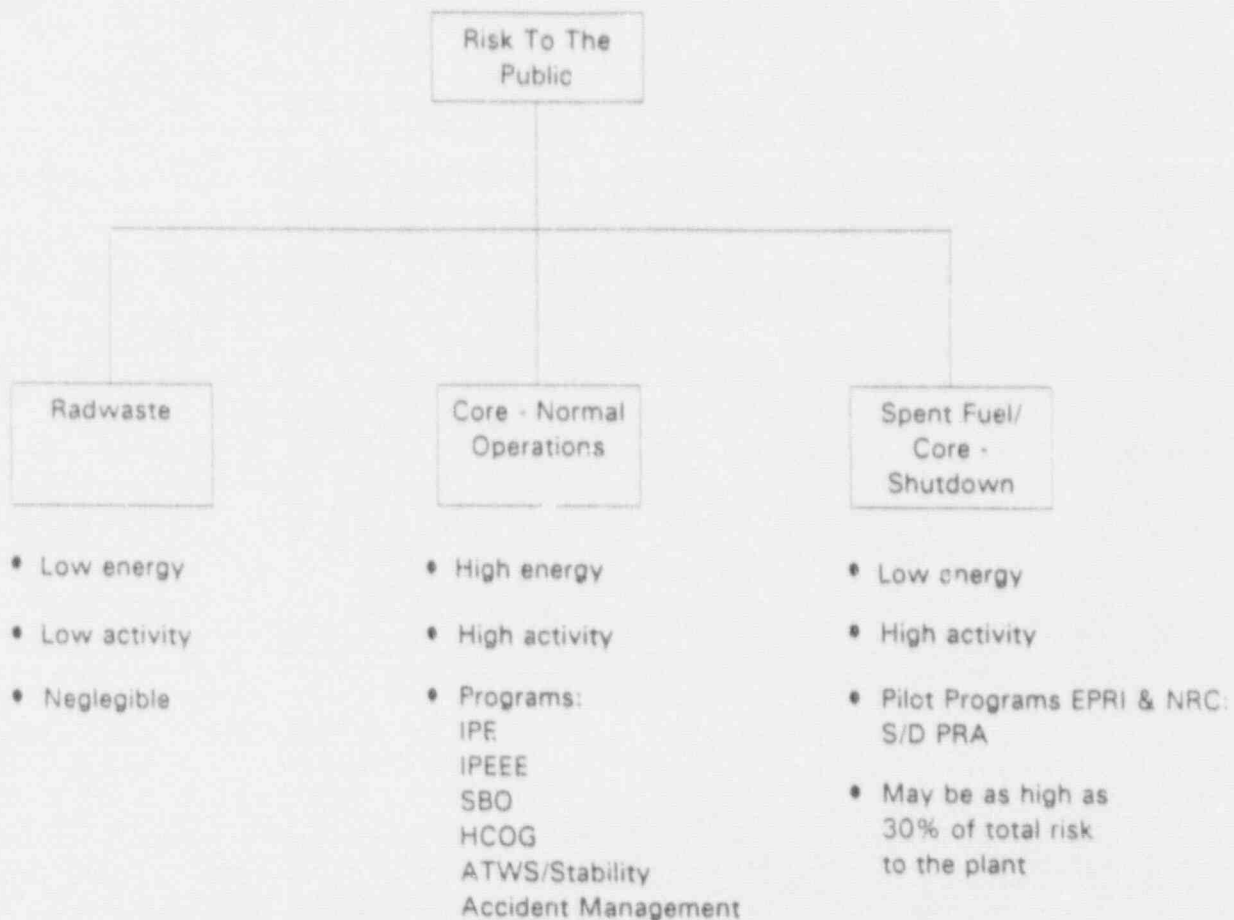
Update of RF4 Status  
Nuclear Regulatory Commission  
March 4, 1992

## SUMMARY OF PRESENTATION

- Risk Management Overview
- RF4 Planned Evolutions
- Pre-Outage Planning Evaluations
- RF4 Outage Evaluations
- MPLD Overview
- MPLD and PRA Integration Into Outage

# RISK MANAGEMENT

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## LEVELS OF RISK MANAGEMENT

- Common Sense Approach
  - Lessons - Learned
  - Industry Experience
  - Precursors
  
- Decision Making Based on Logic Trees
  - Master Plant Logic Diagrams (MPLDs)
  - Dependency Matrices
  
- Decision Making Based on PRA
  - Quantified Model
  - Change in Core Damage Frequency (CDF)  
or Release Frequency

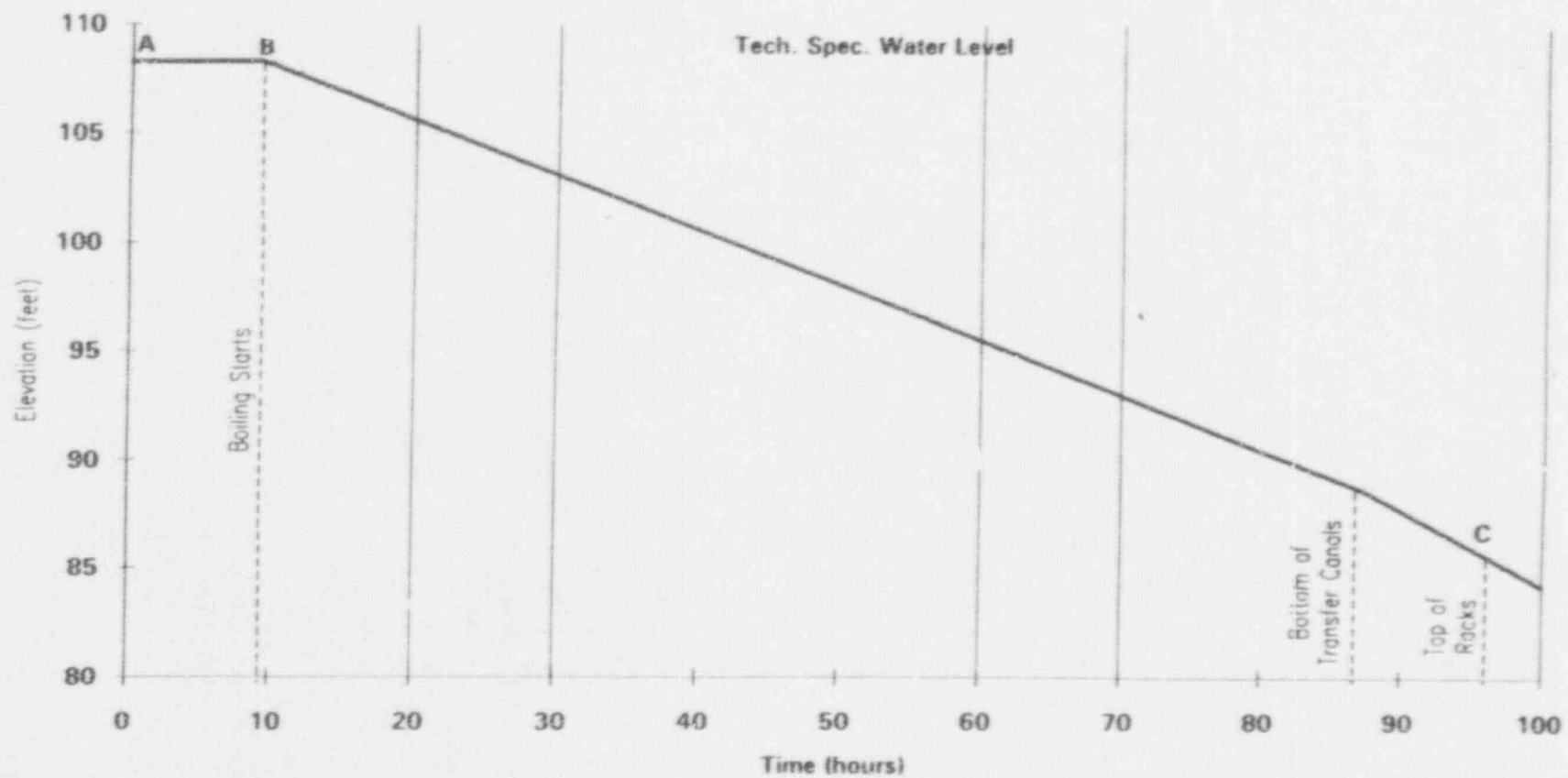
## RF4 EVOLUTIONS

- Full Core Off-Load
- One Emergency Diesel Generator Operable
- Service Water Cleaning
- Service Water Piping Replacement
- Normal Service Water Closure
- RPV Nozzle N4A Replacement
- Source Term Reduction
- Routine Outage Testing/Repairs

## PRE-OUTAGE PLANNING PRE-EVALUATION

- Evaluations Performed Prior to Final Outage Approach
- Evaluation of Outage Options for Mode 6
  - \* External flooding PRA of G-tunnel
  - \* Tornado strike probability
  - \* Tornado missile strike probability
  - \* Tornado depressurization studies
  - \* Seismic probabilities (OBE, SSE, and 0.5g)
  - \* Potential for loss of offsite power
- Evaluation of Mode 6
  - \* Decay heat load from core offload
  - \* Thermal-hydraulic analysis of Spent Fuel Pool
  - \* PRA for loss of SFC, pool boiling, and fuel uncovering

## Spent Fuel Pool Water Level After Loss of Cooling



- A. Probability for loss of spent fuel cooling:  $8.48 \times 10^{-3}$
- B. Probability for fuel pool boiling:  $1.12 \times 10^{-3}$
- C. Probability of fuel uncovering:  $1.93 \times 10^{-8}$



## RF4 OUTAGE EVALUATIONS

- Preferred Transformer Outage PRA
- Developed Mode 5 PRA Models  
(Fuel Bldg HVAC, Spent Fuel Pool Cooling)
- Provide Training To:
  - \* Outage Management
  - \* Operations (5-crews in Requal.)
  - \* NSAG
- Level 1 PRA Models for RHR, Service Water, Electric Power, etc.
- Radiological Analysis for Fuel Building due to Pool Heatup
- Provide Guidance to Operations on Loss of SFC (AOP-0051)
- Application of PRA Models
  - \* On-going RF4 Support using MPLDs and quantifying risk

Challenge:

To develop a set of technically sound controls and tools which allow effective risk management when important safety systems are unavailable

Objectives:

Develop a shut-down Risk Management Program for the RF4 outage

A Master Plant Logic Diagram (MPLD) was developed for the outage

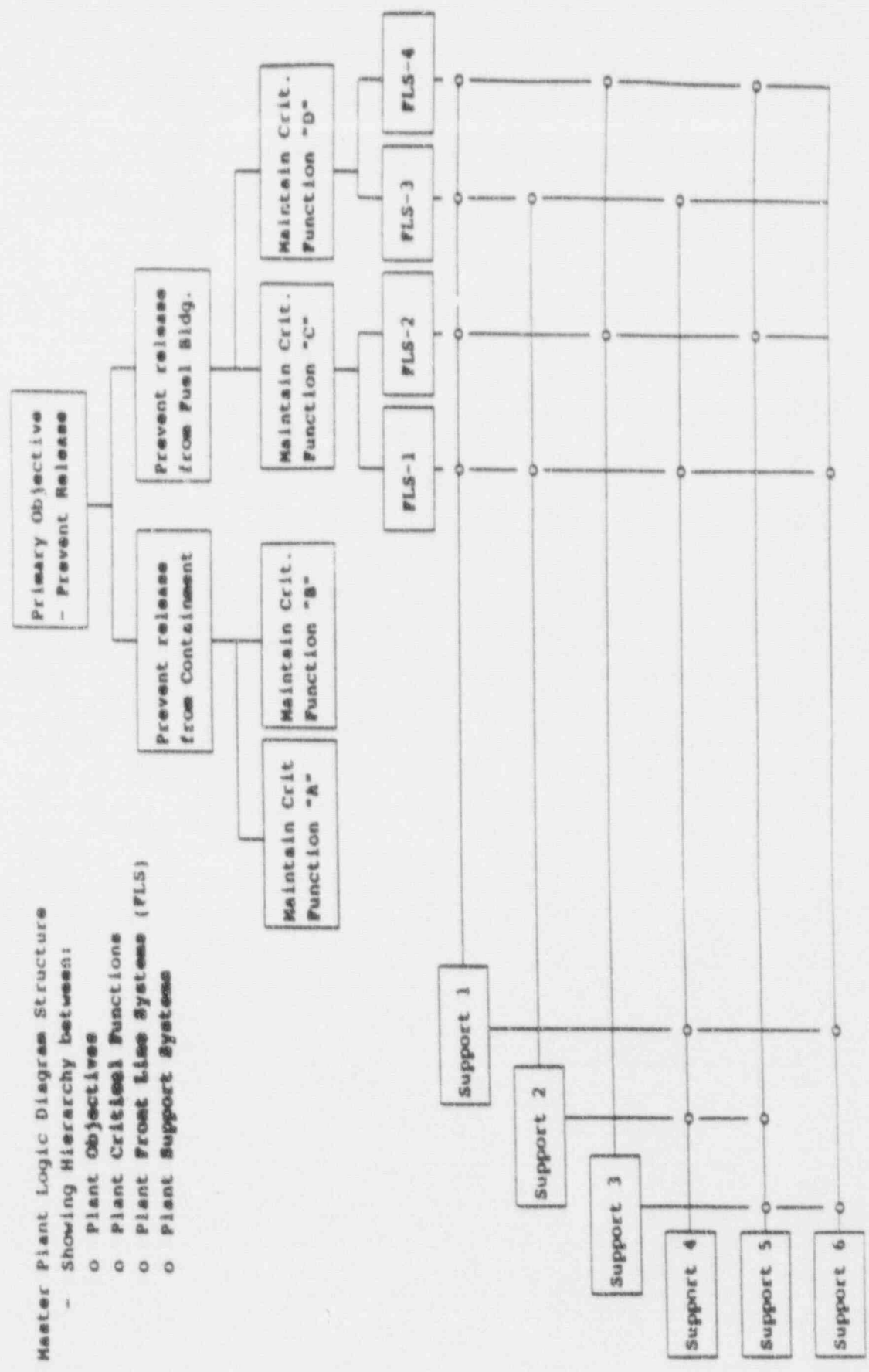
This diagram identifies the relationships between:

- Critical Functions which must be achieved to protect the core and containment and prevent release
- Primary plant systems (front-line systems) which provide needed success paths for the critical functions
- Support systems which must operate to maintain the capability of the front-line systems:
  - motive power
  - cooling
  - actuation and control
  - lubrication

The inter-system relationships are graphically presented as a hierarchy to allow inference of cause-consequence relationships.

- The general structure of the MPLD is shown in Figure 1

Master Plant Logic Diagram Structure  
 - Showing Hierarchy between:  
 o Plant Objectives  
 o Plant Critical Functions  
 o Plant Front Line Systems (FLS)  
 o Plant Support Systems



Structure of MPLD - Figure 1

## MPLD AND PRA INTEGRATION INTO OUTAGE

- PRA and NSAG Engineers On-Shift in Outage Management Center or Control Room for Support
- MPLD Developed with Comments from Operations, NSAG and Outage Management
- MODE 5 PRA Developed
- Check Level II Outage Schedule Against PRA

## CONCLUSIONS

- First Step to Outage Risk Management Using PRA Methods
  
- MPLDs May Not Be 100% Usable This Outage
  - Test/Evaluate
  
- Other Plants are Interested in Developing MPLDs
  
- Outage Team Evaluations Will Minimize Risk in RF4
  
- Programs Developed Will Reduce Risks for Future Outages

**OPERATIONS OVERVIEW - RF-4**

**J.P. SCHIPPERT**

**ASSISTANT PLANT MANAGER -  
OPERATIONS, RADWASTE AND CHEMISTRY**

- **Risk Management**

- **NUREG 1410**
- **Risk Management Concepts Embedded Into Schedule**
- **Operations Personnel Familiar With Schedule**

- **Schedule Execution And Overview**

- **Operations Organization**
- **NSAG Review**
- **MPLD Utilization**
- **NUMARC Comparison**



- **Operations Training**

- Pre-Outage Review
- Ongoing Outage Training
- Startup Training

- **Risk Minimization**

- Switchyard Control
- Diesel Generator Rounds
- Freeze Seal Contingencies
- Diesel Backup Alternate SFC Cooling
- Emergency Fire Protection Water Supply To SFC Cooling
- MR Adverse Impact Reviews
- Emergency Plan Drill

- Contingencies

- Inventory Makeup
- Decay Heat Removal
- Electrical Power
- Containment Recovery - Primary/Secondary
- Reactivity Control

## ENGINEERING PROJECT CONTROL

### MAJOR ENGINEER PROJECTS FOR RF-4

- Service Water
  - Chemical Cleaning
  - Pipe Replacement
  - Closed System
- N4A Nozzle Safe End Repair
- Source Term Reduction
  - Ring Header Replacement
  - Chemical Decontamination
- MOV Testing

## PROJECT CONTROL ITEMS

- Pre-Planning
- Procedural
- Methodology/Technology

## EXAMPLES OF PRE-PLANNING

- Core Off-Load
- Series versus Parallel SWS Cleaning
- Clean versus Replacement of SWS Piping

## PROCEDURAL

- Project Organization
- Generic Modification Request (MR)
- Status Tracking

## Generic MR Process

### Design

- Approved for Work
- ① Design requirement document  
(What-How-Application Boundary)
- Estimated Cost
- ② Level II Schedules
- Design Proceeds

### Review

- System Engr - Oper - Maint Checklists
- ③ Post design review and adverse impact statement
- ④ Release to Maintenance & Planning

### Installation

- Plan Maintenance Work Orders
- Material Procurement Completed
- ⑤ MR Release for work
- Changes require FCN afterwards

### Close Out

- Critique
- ⑥ Transmittal to PPF

Modification Number: MR 55-0561

H. Khan/4553

Description: HVK CHILLERS HAVING

Design Engineer/ext.

Maintenance Planner/ext.

GHOST LIGHTS DUE TO SNEAK

W. Mc DOUGALD/4568

System Engineer/ext.

Maintenance Planner/ext.

CIRCUIT. ELIMINATE THEM USING  
BLOCKING DIODES

BOB JELISON/2295

Operations/ext.

Materials/ext.

NOTE: WORK ON CHILLER A, B, & D  
HAS BEEN COMPLETED.

HARVEY LACAS/3612

Maintenance Planner/ext.

TRUDY

WORK ON DIV I CHILLER C  
IS SCHEDULED FOR RF-4  
WHICH REQUIRES DEENERGIZATION  
OF EJS BUS

Activity	Responsibility	ECD	Status
Design Requirements	Design Engineer	12/21/90A	COMPLETED
BOM	Design Engineer	2/09/91A	COMPLETED
Post Design Review Meeting	Design Engineer	N/A	MR RELEASED FOR WORK
Release For Planning	Design Engineer	3/27/91A	COMPLETED
MWO's			
<u>MWO # R147303, R137364</u> <u>AND R146652</u>	Electrical Mechanical I&C	6/18/91A	COMPLETED
Stock Mat. Staged	Warehouse	2/19/91A	COMPLETED
Purchased Material (Req.)	Warehouse	3/5/91A	COMPLETED
Purchased Material (P.O.)	Purchasing	3/5/91A	COMPLETED
Purchased Material Staged	Warehouse	5/4/91A	COMPLETED
Material Verification	Design Engineer	5/19/91	COMPLETED
RFW	Design Engineer	3/28/91A	COMPLETED
Installation By MWO	Maintenance	4/18/92	
PCP's	Design Engineer	4/25/92	
Block 35	Design Engineer	_____	_____
Block 36	Design Engineer	_____	_____
PPF	DCC	_____	_____

Comments: NOT currently on RF-4 list as of 1-31-92  
ms. Harvey can still be planning contact however  
do not want to spend resources on unless its approved  
for RF-4 ms.



## APPLIED METHODOLOGY/TECHNOLOGY

- Fuel Shuffling - COSMOS
- Configuration - MPLD
- Welding Overlay Contingency
- Reactor Vessel/Nozzle Mock-Ups

# SERVICE WATER PROGRAM

## INTRODUCTION

- System Description Video

- System Condition

- Program Objective

- Program Organization

## ● System Condition

- ✓ Corrosion product buildup - degraded flow
- ✓ MIC - localized pitting
- ✓ Pinhole leaks - 11
- ✓ System structurally sound
- ✓ Safety performance requirements met

## ● Program Objective

Restore health of system so that system will last life of Plant with normal maintenance

- ✓ Off-Line chemically clean
- ✓ Selected pipe/valve/component replacement/refurbishment
- ✓ Selected mechanical cleaning
- ✓ Close system
- ✓ Implement aggressive water treatment program to significantly reduce corrosion rates and impact of MIC

# ● Organization

Service Water  
Program  
Project Manager  
KESuhrke

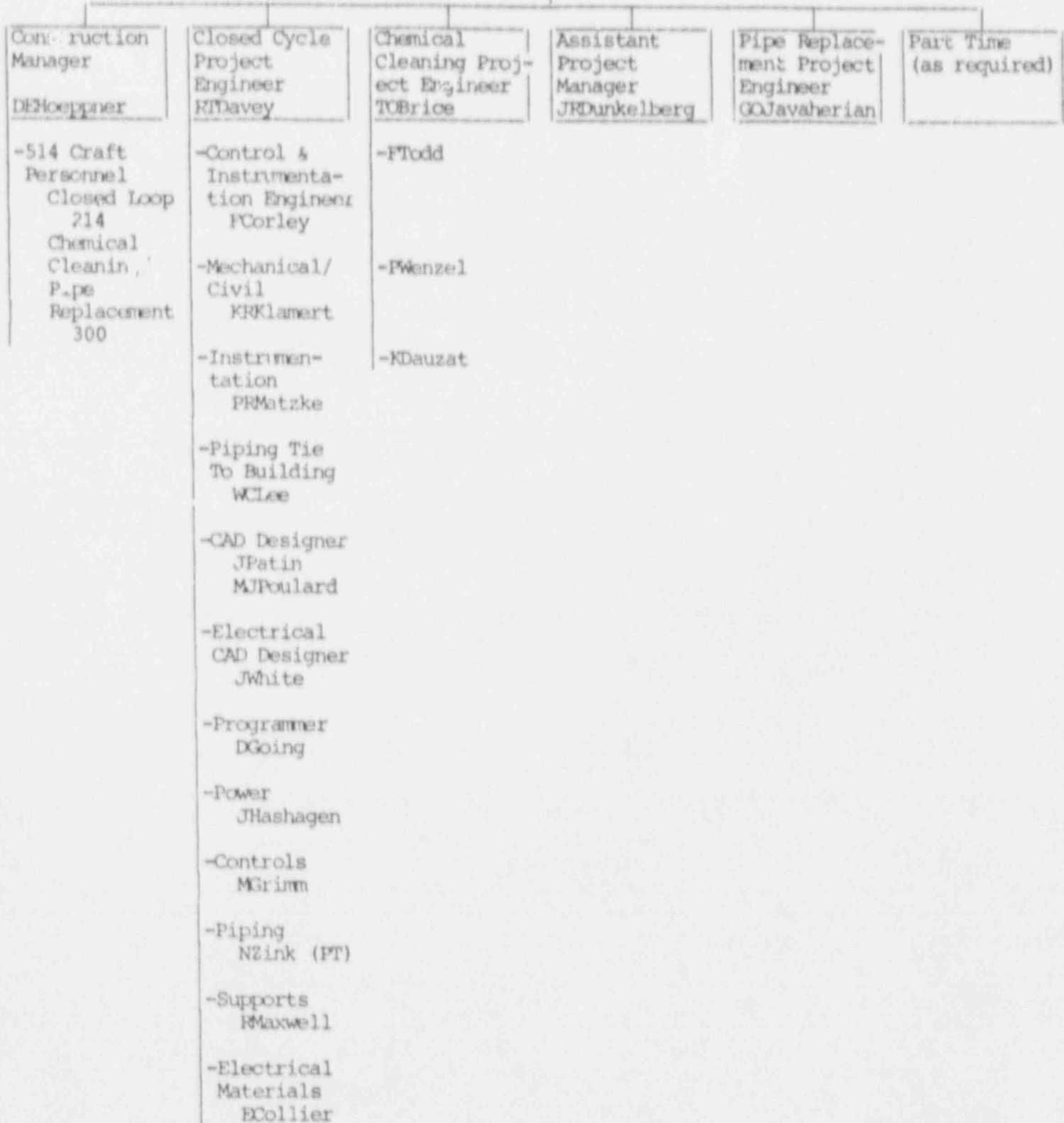
Construction Manager DEHoeppner	Closed Cycle Project Engineer RIDavey	Chemical Cleaning Project Engineer TOBrice	Assistant Project Manager JRDunkelberg	Pipe Replacement Project Engineer GOJavaherian	Part Time (as required)
-Construction Assistant SAlsandor	-Lead Mechanical Engineer CEDeweese	-GHughes	-Scheduler JGottschalck	-Pipe Design Engineer DNaike DReed	-Inspect/Clean/Replace Heat Exchanger JKHam
-Field Supervisor--Closed Cycle SGray	-Fluid System Engineer JACampbell	-GJMermigas	-Cost BIMorgan	-Pipe Support Engineer CFontain	-Clarifier NKariouk
-Field Supervisor--Chemical Cleaning/Pipe Replacement JSalmon	-Lead Civil Engineer TDRouns	-Dowell Schlumberger			-Procurement EPBell
-Planners (Functionally report to Maintenance) EBarrass DCampanile JCowart RHelton ELoveday PSullivan	-Major Equipment Piping Layout CGirgis	-UE&C			-Outage Management/Operations TPLacy
-Field Coordinator/Engineer --Pipe Replacement/Chemical Cleaning BBurgess BLoftin MRutherford Rhibaudeau DTurney	-Lead Electrical Engineer TOMoffitt	-WSullivan			-Start-Up & Test-- Closed Cycle CAWomack Jr + 8 (FT) Test Personnel
-SWP Field Material Coordinator JClark	-Foundations BLenox	-NShah			

CONTINUED ON NEXT PAGE

# ● Organization

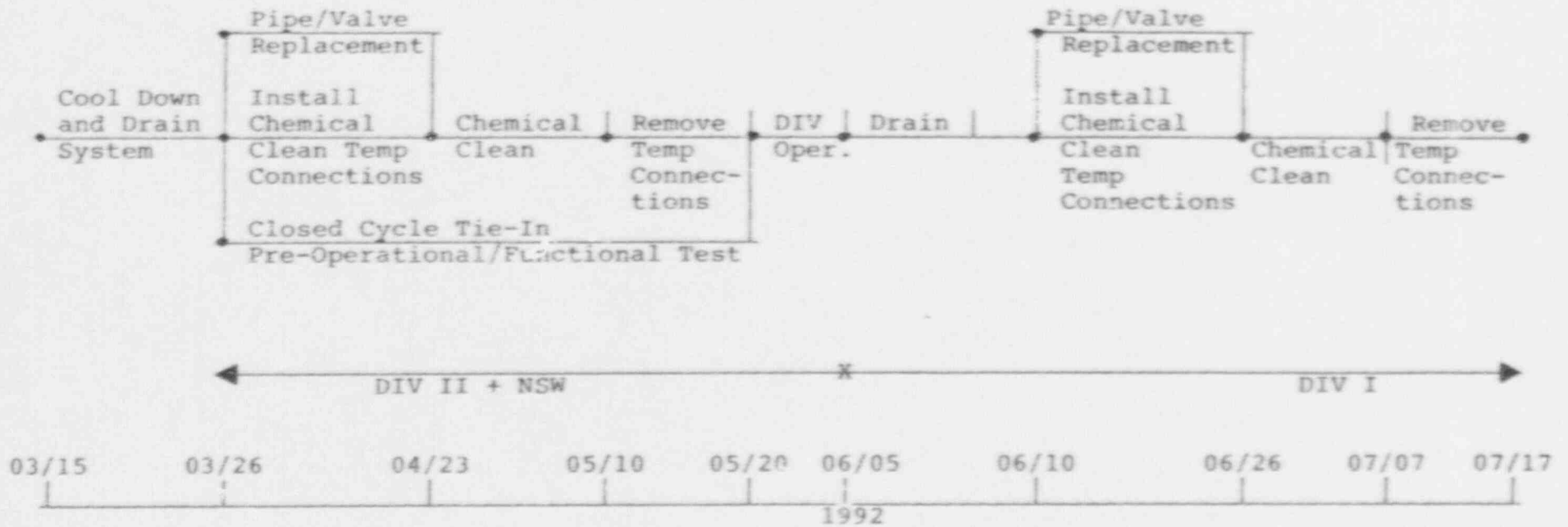
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Service Water Program Project Manager KESuhrke
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# ●RF-4 Schedule

## SERVICE WATER SUMMARY RF-4 SCHEDULE



40

**SERVICE WATER PROJECT  
CHEMICAL CLEANING**

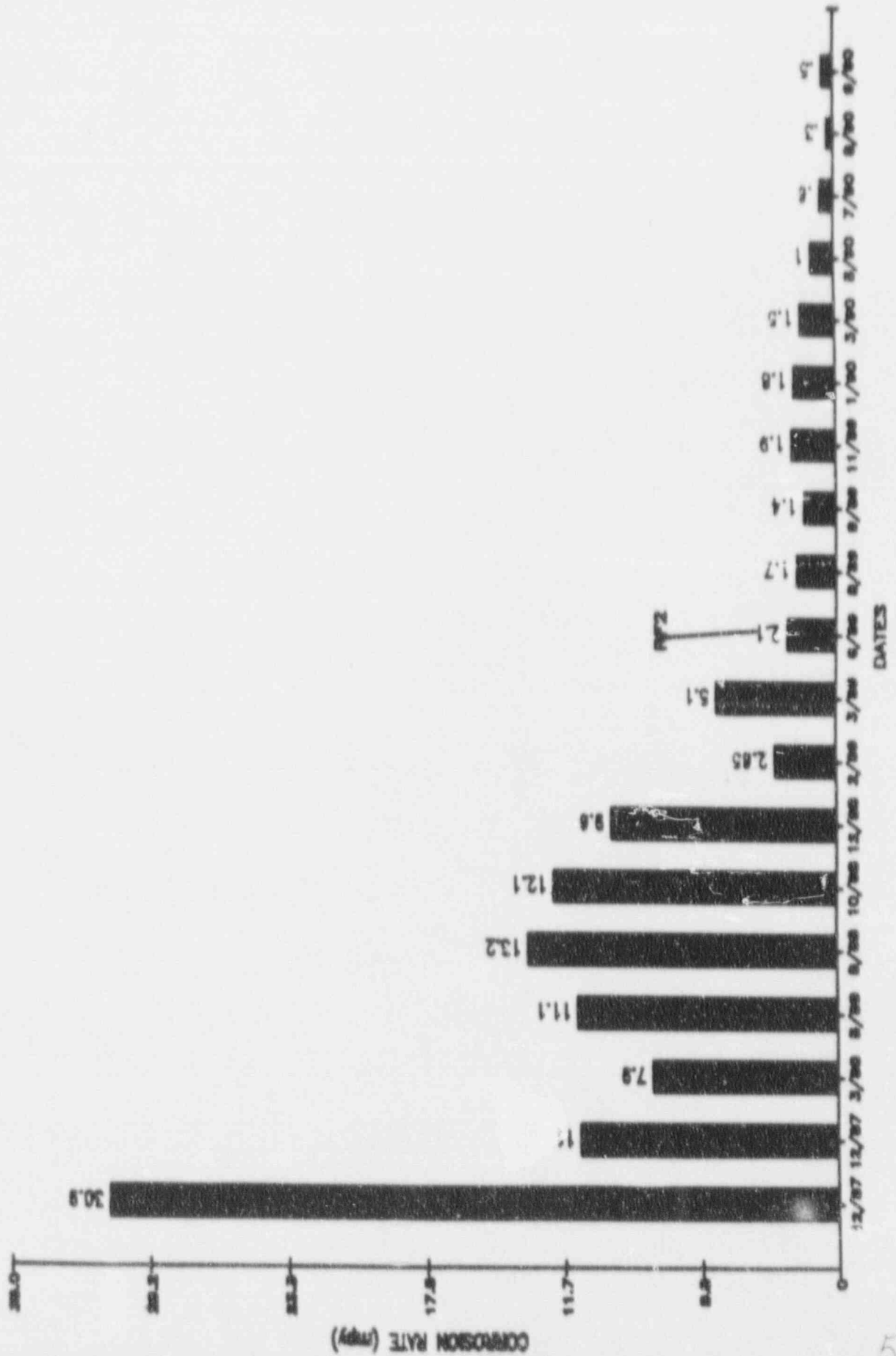
**TAMMY O. BRICE  
CHEMICAL CLEANING PROJECT ENGINEER**



# ASSESSMENT OF SYSTEM CONDITION

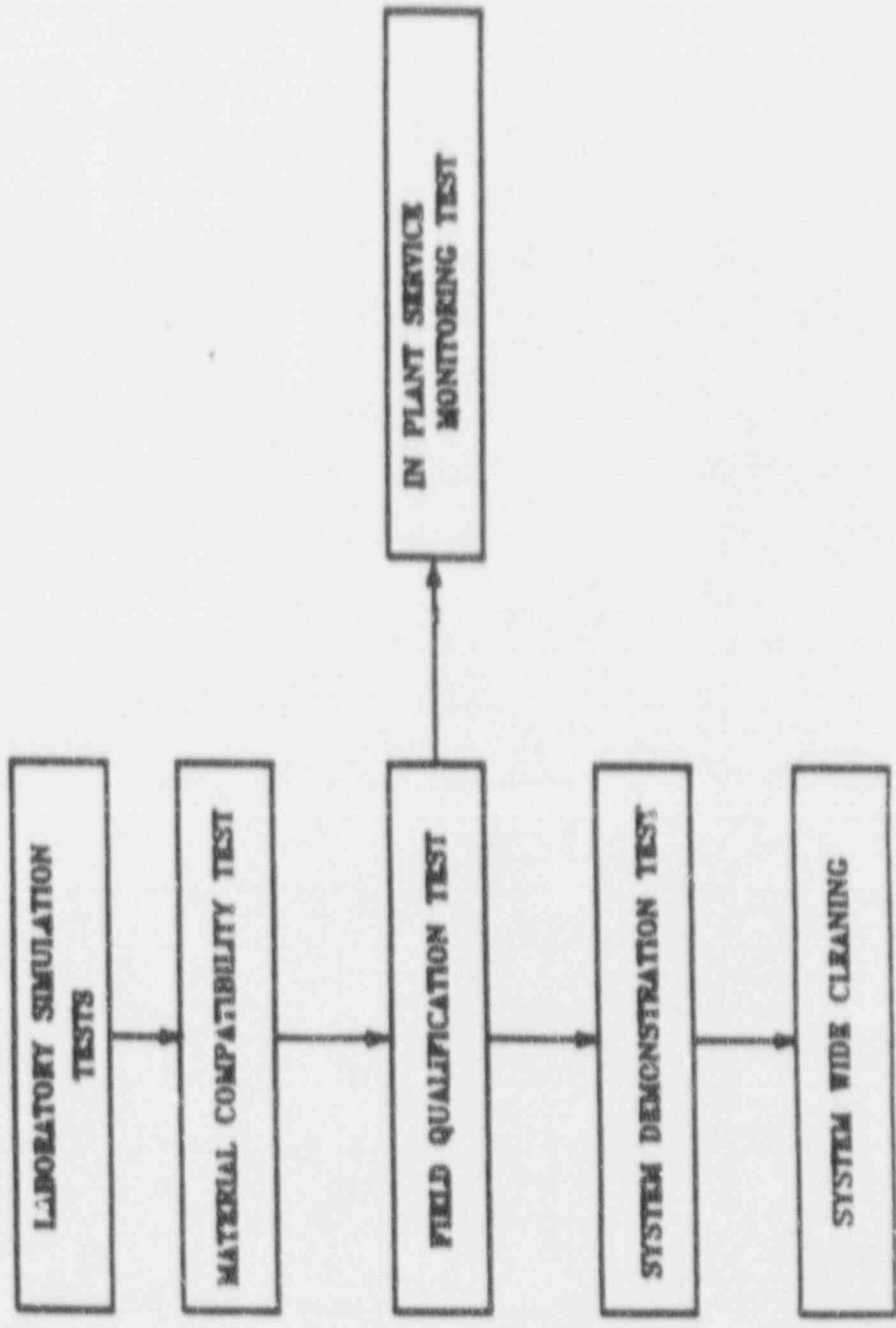
- CORROSION PRODUCT BUILDUP
- MICROBIOLOGICALLY INDUCED CORROSION
- LEAKS - 12
- LOCALIZED PITTING
- DEGRADED FLOW
  - Fouling of valve seats
  - General corrosion product buildup obstructing flow
  - Blocking of heat exchanger tubes
- MUD/SILT ACCUMULATION

GULF STATES UTILITIES - RIVER BEND STATION  
 STEEL CORROSION RATES  
 DECEMBER 1987 - SEPTEMBER 1990



Polylocate till 10/88; Zn till 7/89; Zn & PO4 since 7/89

# PROCESS DEVELOPMENT PLAN (PHASED APPROACH)



## **Finalized Multi-Stage Chemical Cleaning Process**

- **Iron Removal Stage**
- **Alkaline Stage**
- **Copper Removal Stage**
- **Passivation Stage**

## **Qualification of the Chemical Cleaning Process**

- **Quality assurance requirements for chemical cleaning as a "special process" are met**
- **Material compatibility studies**
- **A field qualification test**
- **A high velocity corrosion rate test**

# System Demonstration Test

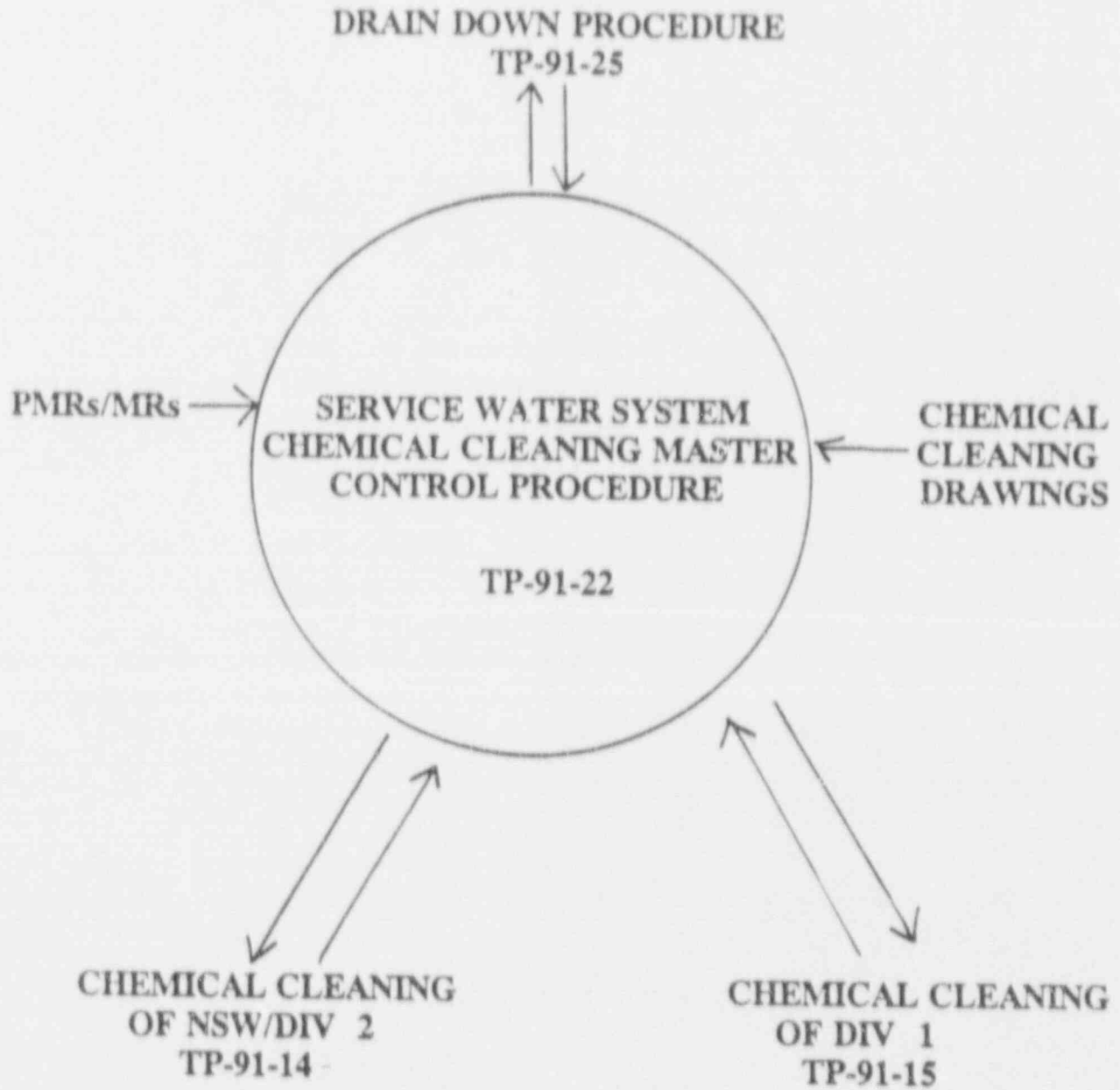
- Experience with a larger scale chemical cleaning job on site
- Provides additional in plant experience with the qualified cleaning process
- The overall logistics will be tested since the transport of chemicals, waste handling and manpower loading will all be exercised
- Provides additional data on the type of waste that should be generated

## **FULL SYSTEM CHEMICAL CLEANING (RF-4)**

- **SYSTEM VOLUMES  
(2 CLEANING LOOPS)**
  - ▶ **NORMAL SERVICE WATER/DIVISION 2:  
≈ 175,000 GALLONS**
  - ▶ **DIVISION 1 : 85,000 GALLONS**
- **ESTIMATED WASTE VOLUME:  $2.6 \times 10^6$   
GALLONS**
- **TEMPORARY DELIVERY SYSTEM**
- **JUMPERING HEAT EXCHANGERS**
- **3 WASTE STORAGE TANKS ( $1.2 \times 10^6$   
GALLONS EACH)**



# WORK CONTROL DOCUMENTS





## WASTE PROCESSING

- FQT WASTE CHARACTERIZATION
- DEMONSTRATION TEST WASTE CHARACTERIZATION AND ON SITE PROCESSING
- SYSTEM WIDE CHEMICAL CLEANING

## CONCLUSIONS

The lab simulation testing, the material compatibility studies, the field qualification test, the high velocity, and the radwaste cleaning are a series of activities which will develop a cleaning process..

To effectively chemically clean the Service Water System with a qualified chemical cleaning process which is compatible with the SWS material.

**SERVICE WATER PIPE-REPLACEMENT  
PROGRAM**

**GULF STATE UTILITIES COMPANY**

**BOP Supervisor: Mike Stein**

**Ext. 4675**

**Date: 3-4-92**

## TYPICAL FINDINGS FROM INSPECTIONS

- A. REDUCTION OF WATER FLOW RATE TO SOME EQUIPMENT. AS A RESULT, CHEMICAL CLEANING CAN NOT BE PERFORMED.
  
- B. DRAIN / VENT / INSTRUMENT VALVES CAN NOT BE OPERATED DUE TO BLOCKAGE OF PIPELINES AND VALVES. AS A RESULT, VALVES REQUIRED FOR FILLING AND VENTING OF CHEMICAL CLEANING AND FOR NORMAL OPERATION MUST BE CHANGED.
  
- C. WELD JOINTS BETWEEN A FEW VALVES AND PIPES HAVE LEAKED DUE TO CORROSION. THIS REPRESENTS A RISK OF LEAKAGE DURING CHEMICAL CLEANING.

## GOALS AND PRINCIPLES FOR PIPE-REPLACEMENT

1. IMPROVE THE SERVICE WATER SYSTEM EQUIPMENT PERFORMANCE TO MEET OR EXCEED SAFETY REQUIREMENTS AND/OR TO ENSURE VITAL EQUIPMENT AND PLANT RELIABILITY.
2. ESTABLISH AND IMPLEMENT A SAMPLE PLAN FOR ULTRASONIC TEST INSPECTIONS OF LARGE BORE ASME PIPING THAT IS MOST SUSCEPTIBLE TO CORROSION.
3. REPLACE / REPAIR PIPELINES TO REDUCE PROBABILITY OF ASME MINIMUM WALL VIOLATIONS, IF INSPECTION-SAMPLE TESTING SO INDICATES.
4. REPLACE / REPAIR VALVES NECESSARY TO SUPPORT PLANT OPERATION AND CHEMICAL CLEANING.
- 5- ALL PIPE-REPLACEMENT NEED NOT NECESSARILY BE ACCOMPLISHED DURING CYCLE-4 OR RF-4.
6. THE PRIORITY SYSTEM USED IN ESTABLISHING THE PIPE REPLACEMENT SCHEDULE MUST BE BASED ON MEETING SAFETY AND PLANT-RELIABILITY STANDARDS.
7. THE NEW CLOSED-CYCLE SERVICE WATER SYSTEM MUST ADDRESS AND ACCEPT RESIDUAL CORROSION PRODUCTS IN THE 1% OF THE PIPING THAT WILL NOT BE REPLACED OR CLEANED.

## PIPE-REPLACEMENT SCOPE

1. REPLACE SMALL-DIAMETER PIPING THAT CAN NOT BE CLEANED DUE TO BLOCKAGE.
2. REPLACE DRAIN / INSTRUMENT / VENT VALVES THAT CAN NOT BE OPERATED DUE TO BLOCKAGE.
3. FINALIZE THE PIPE-REPLACEMENT PROGRAM BY IMPLEMENTING AN ULTRASONIC TEST-SAMPLE INSPECTION PLAN

## PIPELINES THAT ARE SCHEDULED TO BE REPLACED

<u>SIZE</u>	<u>LENGTH</u>	<u>ASME</u>	<u>NON-ASME</u>
1/2"	90'	N/A	90'
3/4"	700'	600'	100'
1"	15'	15'	N/A
2"	511'	400'	111'
3"	447'	N/A	447'
4"	170'	40'	130'
<b>TOTAL</b>	<b>1,933'</b>	<b>1,055'</b>	<b>878'</b>

**NOTE:**

950 FEET OF THE ABOVE 1,933 FEET OF PIPE WERE REPLACED DURING PLANT NORMAL OPERATION AND PAST FORCED OUTAGE.

**VALVES THAT ARE SCHEDULED TO BE  
REFURBISHED / REPLACED  
WITHIN PIPELINES**

**230 VALVES**

<u>SIZE</u>	<u>ASME</u>	<u>NON-ASME</u>
1/2"	2	N/A
3/4"	98	67
2"	2	2
2.5"	48	21
<b>TOTAL</b>	<b>150</b>	<b>80</b>

**NOTE:**

**120 OF THE ABOVE 230 VALVES WERE REPLACED  
DURING PLANT NORMAL OPERATION AND PAST  
FORCED OUTAGE.**



**DRAIN / VENT / INSTRUMENT  
VALVES THAT ARE SCHEDULED  
TO BE REPLACED**

**182 VALVES**

<u>SIZE</u>	<u>ASME</u>	<u>NON-ASME</u>
1/2"	24	6
3/4"	111	41
TOTAL	135	47

**NOTE:**

**73 OF THE ABOVE 182 VALVES WERE REPLACED  
DURING PLANT NORMAL OPERATION AND PAST  
FORCED OUTAGE.**

## ULTRASONIC TEST-SAMPLE INSPECTION PLAN

### PURPOSE:

- TO ENSURE THE STRUCTURAL INTEGRITY OF THE PIPELINES.
- TO DETERMINE THE GENERAL CONDITION OF THE PIPELINES.  
(FOR ASME CODE COMPLIANCE)
- TO DEVELOP A METHOD OF SAMPLE INSPECTION, WHICH  
WILL PROVIDE VERY HIGH LEVEL OF CONFIDENCE THAT  
THE S.W. PIPING IS IN ACCEPTABLE CONDITION.

# ULTRASONIC TEST-SAMPLE INSPECTION PLAN

## INSPECTION PROGRAM (TP-91-037)

### 1 - PIPELINES SELECTION:

THOSE SELECTED WERE CONSIDERED TO BE THE MOST SUSCEPTIBLE TO CORROSION.

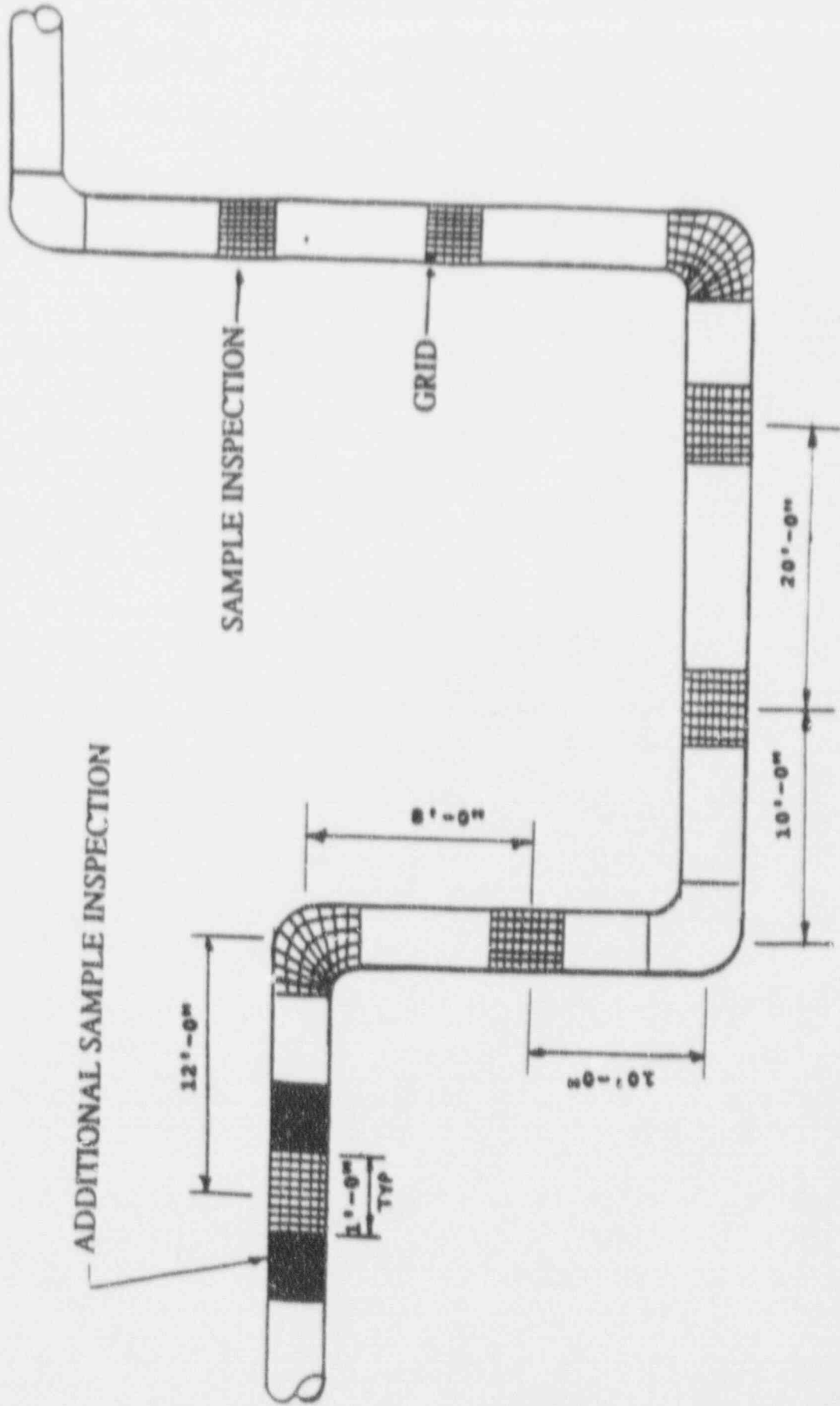
### 2 - SAMPLE SELECTION:

ONE SAMPLE FOR EVERY 20 FEET MAXIMUM WAS SELECTED.  
ONE SAMPLE FOR EVERY OTHER FITTING WAS SELECTED.

### 3 - ACCEPTANCE CRITERIA:

- PIPE STRUCTURAL INTEGRITY COULD NOT BE VIOLATED.  
(VERIFIED BY MEANS OF CALCULATION).
- ONE ASME MINIMUM WALL VIOLATION WAS ALLOWED PER  
SAMPLE WITH LOCAL REPAIR/REPLACEMENT REQUIRED.
- IF REMAINING WALL THICKNESS IS LESS THAN  
CALCULATED 35 YEARS WALL THICKNESS, THE AFFECTED  
PIPELINES TO BE MONITORED DURING EACH CYCLE.

# EXAMPLE



## SAMPLE PLAN SUMMARY

SIZE	LENGTH OF ASME PIPE	NUMBER OF SAMPLE	NUMBER OF GRID SQUARE	NUMBER OF GRID SQUARE < SPEC.	NUMBER OF GRID SQUARE < ASME
3"	80'	7	484	5	NONE
6"	25'	5	367	13	NONE
8"	500'	38	3115	195	1
12"	600'	41	3480	80	1
TOTAL	1,205'	91	7446	293	2

## ULTRASONIC TEST-SAMPLE INSPECTION RESULTS

- 1 - THE REMAINING WALL THICKNESSES OF TWO GRID SQUARES WAS LESS THAN THE ASME MINIMUM WALL THICKNESS. REPAIRED COMPLETED.
- 2 - THE WALL THICKNESSES OF 293 GRID SQUARES HAVE BEEN REDUCED TO BELOW THE SPECIFICATION MINIMUM WALL THICKNESS.
- 3 - EXCEPT FOR THE TWO GRID SQUARES NOTED ABOVE, REMAINING PIPE WALL THICKNESSES WERE GREATER THAN 35 YEAR CALCULATED WALL THICKNESS.
- 4 - CALCULATIONS VERIFY THAT THE EXISTING PIPE WALL THICKNESSES ARE SUFFICIENT TO MAINTAIN STRUCTURAL INTEGRITY.

**THE CLOSED CYCLE  
SERVICE WATER SYSTEM**

**Tom Davey**

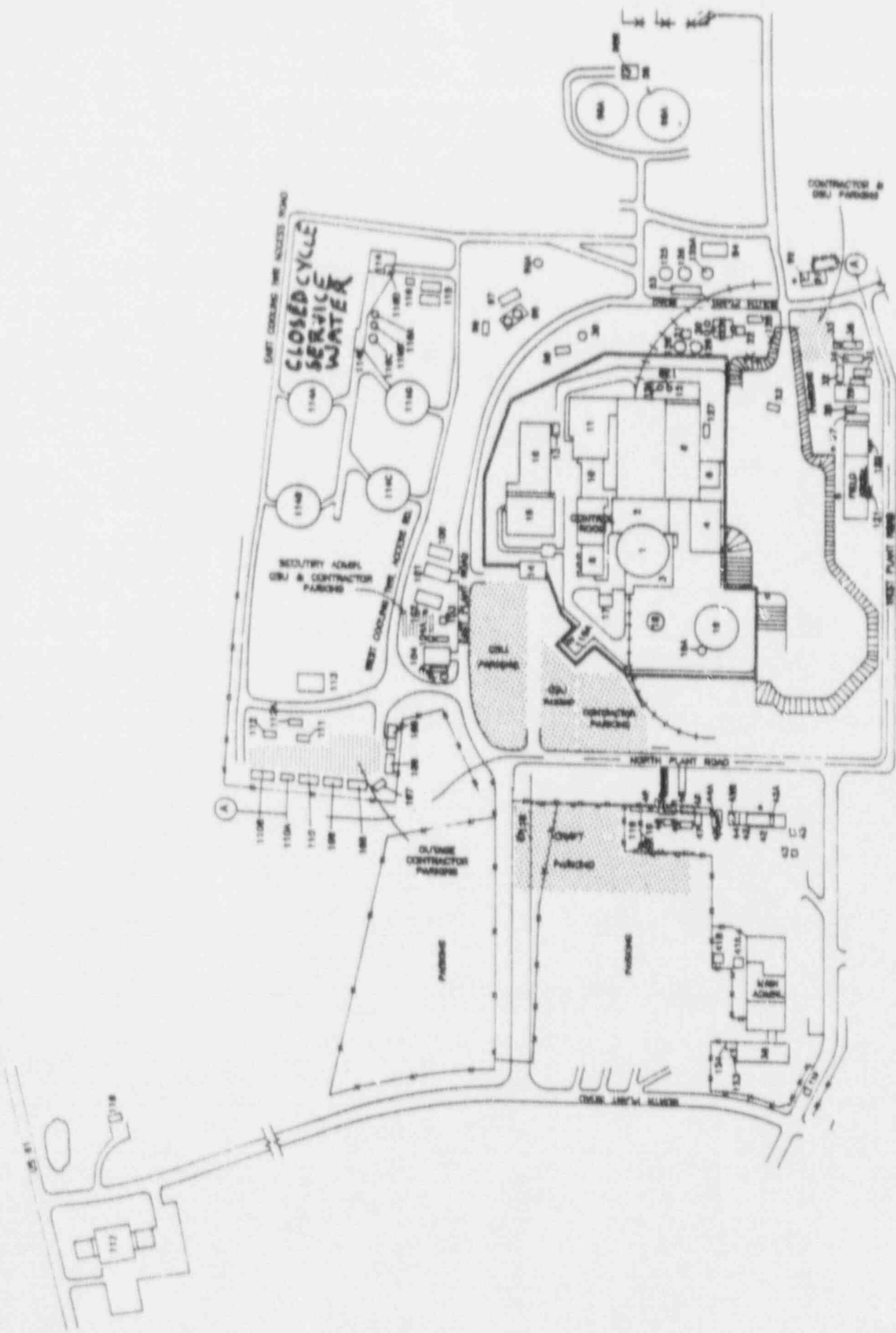
**Closed Service Water  
Project Engineer**



## SERVICE WATER: CLOSING THE SYSTEM

- OBJECTIVE:** Modify the existing service water system to a closed system
- TIME FRAME:** Closed system to be operational during the fourth refueling outage. Significant because groundbreaking occurred May 22, 1991
- ATTRIBUTES:**
- Trouble/accident free
  - Designed in-house
  - Accelerated design & construction time frame

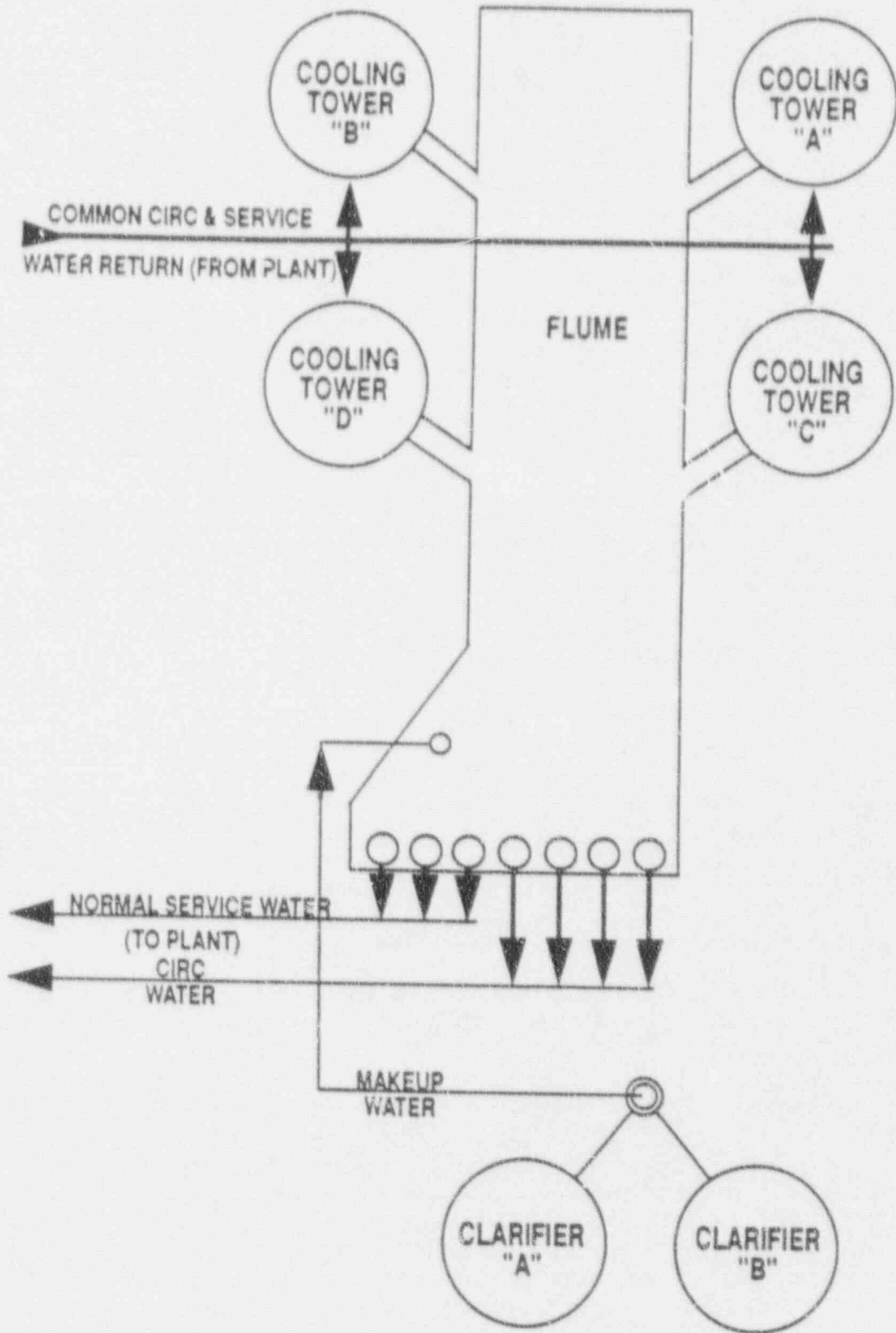




## THE EXISTING SERVICE WATER SYSTEM

- Service water is common with circulating water using mechanical draft wet cooling towers to lower water temperature
- Water quality is limited because of interaction with atmosphere in the 4 cooling towers and the use of clarified river water for makeup
- Chemical treatment of the common systems is utilized to control corrosion and biological fouling, but is costly because of the high water dilution factor
- Water treatment has to be restricted to avoid exceeding EPA discharge limits during blowdown to the river

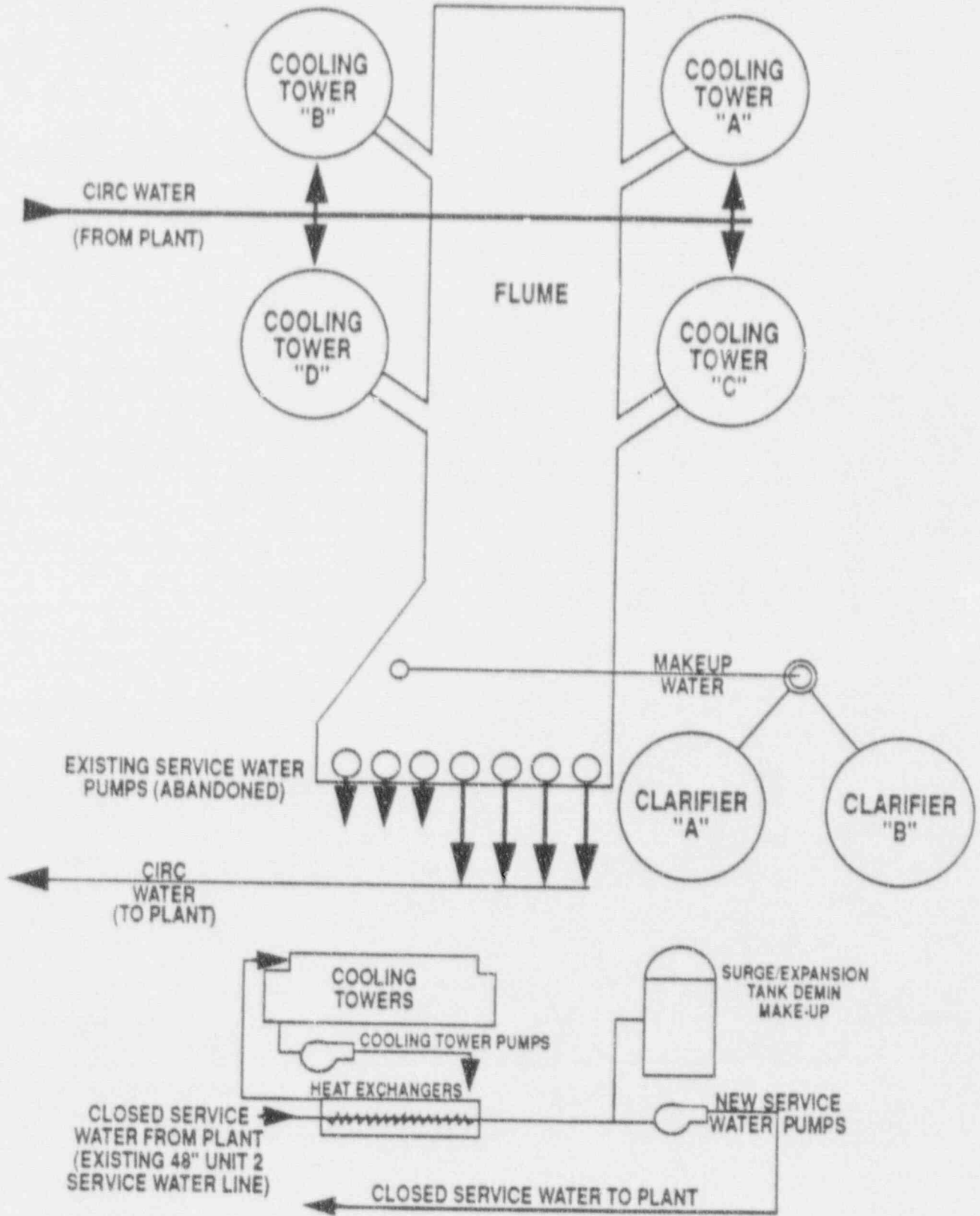
# EXISTING "OPEN" SERVICE WATER SYSTEM



## THE NEW CLOSED SYSTEM

- New design separates entire service water system from circulating water / service water system
- The new service water system will be cooled using new heat exchangers
- The heat exchangers will be cooled by water recirculating through a new cooling tower
- The design of the new system provides smooth interface to existing systems and precludes changes to safety systems. The new system is non safety related and does not affect the safety aspects of the Standby Cooling Tower system
- The new system allows aggressive water treatment because there is less volume to treat. EPA discharge requirements are easier to meet and since service water is no longer exposed to atmosphere, there are no evaporative losses

# NEW CLOSED SERVICE WATER SYSTEM



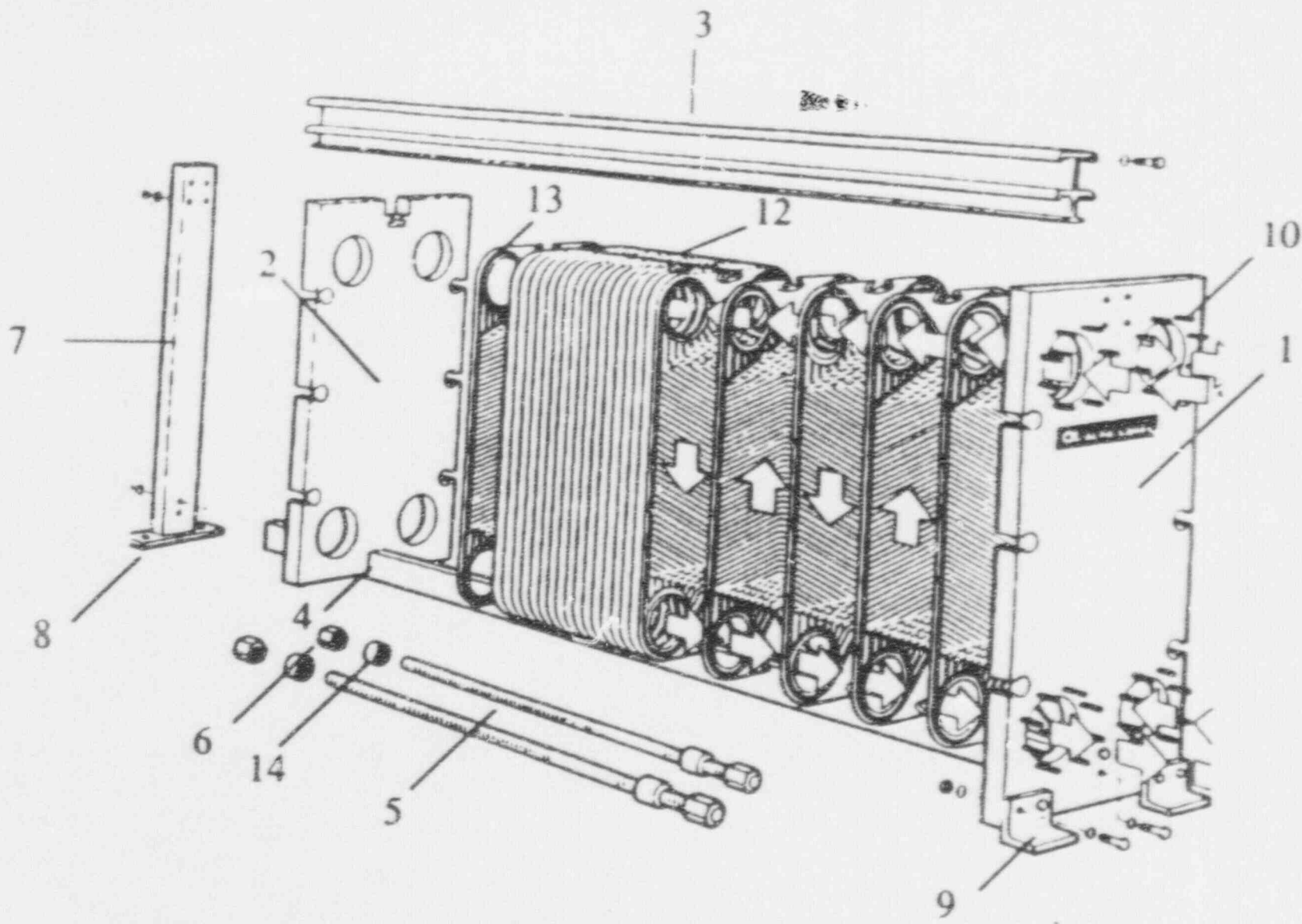


## MAJOR COMPONENTS

- Plate heat exchangers
- Pumps (Cooling tower pumps and in-line service water pumps)
- Cooling tower
- Approximately 4000 feet of 48 inch diameter pipe
- Surge/expansion tank w/ approximately 40,000 gallon capacity

## PLATE HEAT EXCHANGERS

- Low cost (compared to other heat exchanger types)
- Less space requirements than shell and tube
- Easily maintained
- Easily expanded - modular design
- Good operating experience in plants using Mississippi river water for cooling
- Eight heat exchangers measuring approximately 4 feet wide by 15 feet long by 10 feet high



- |                      |                     |                   |
|----------------------|---------------------|-------------------|
| #1 - Fixed Cover     | #6 - Tightening Nut | #11 - Bearing Box |
| #2 - Movable Cover   | #7 - Support Column | #12 - Plate Pack  |
| #3 - Carrying Bar    | #8 - Support Foot   | #13 - Gasket      |
| #4 - Guide Bar       | #9 - Frame Foot     | #14 - Lock Washer |
| #5 - Tightening Bolt | #10 - Stud Bolt     |                   |



# PUMPS

## COOLING TOWER PUMPS

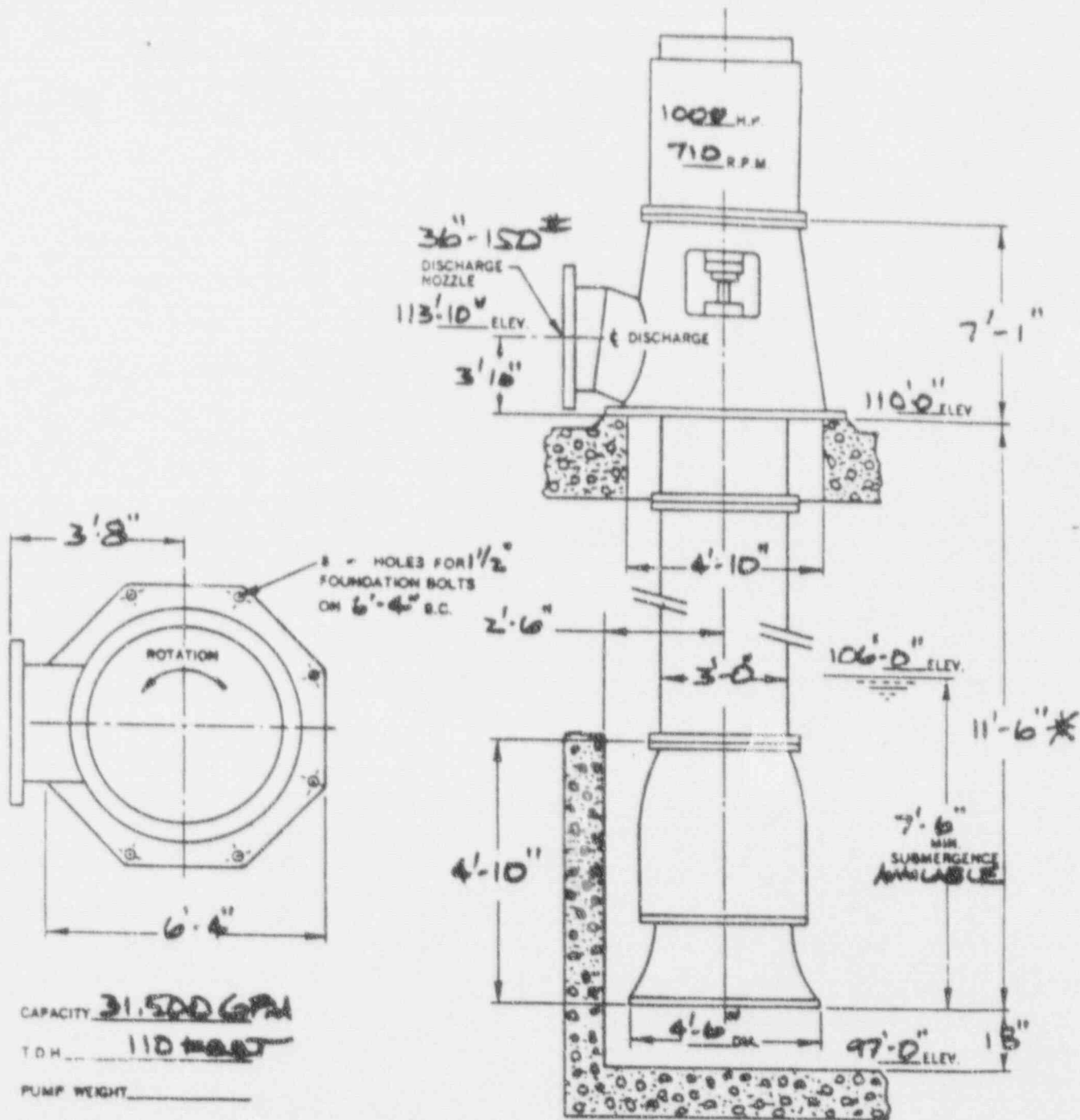
- Three 50% capacity vertical pumps (approximately 31,500 GPM each) at 1000 HP each
- Cooling tower flume (about 300 feet long by 60 feet wide) with pump pit

## IN LINE PUMPS

- Three 50% capacity horizontal pumps (approximately 31,500 GPM each)
- Each pump motor combination measures about 20 feet long by 10 feet wide by 10 feet high and is rated at 2000 HP
- Design utilizes expansion tank to control surges

# VERTICAL CIRCULATING PUMP

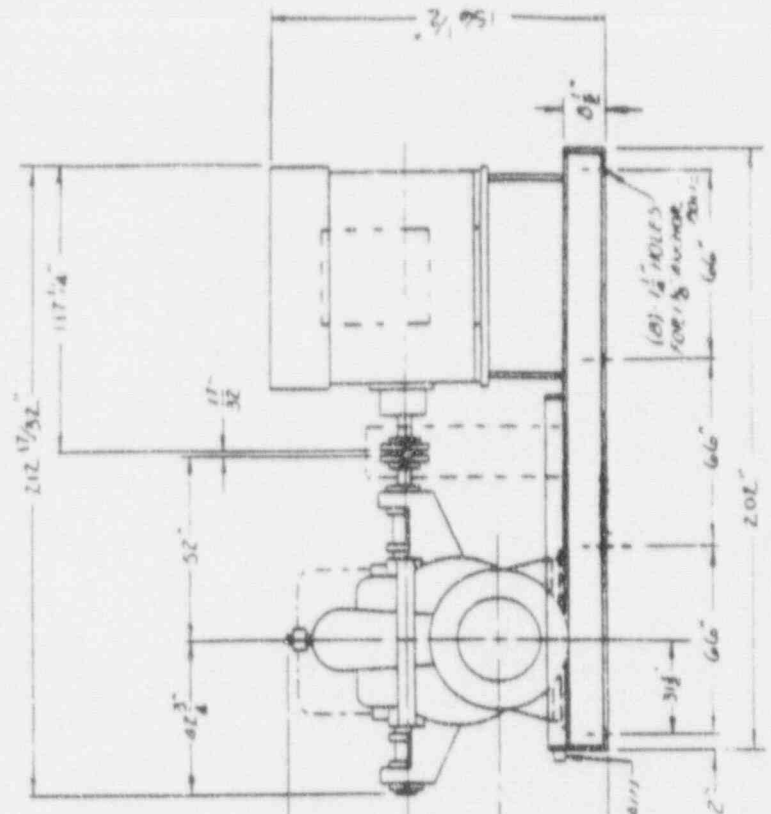
## Preliminary Outline Dimensions



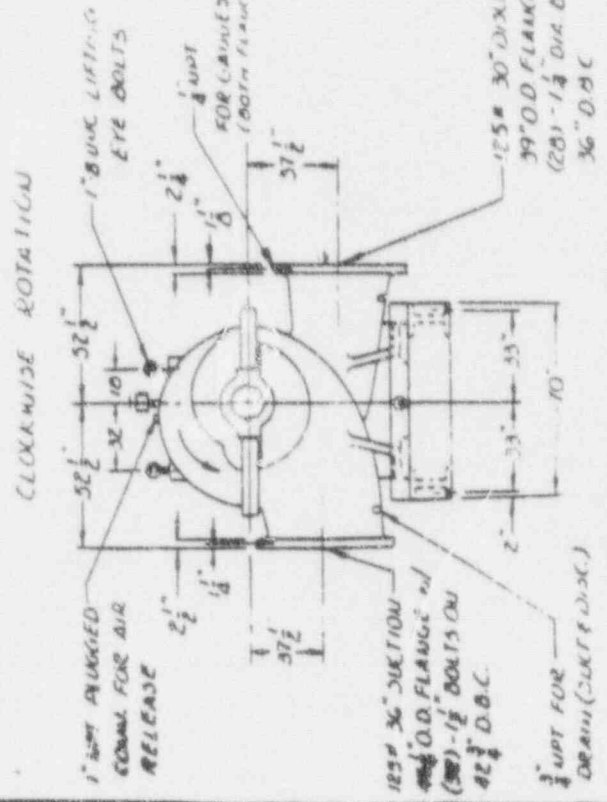
NOTE: Dimensions are preliminary and are not to be used for construction purposes.

USER: <b>GULF STATES UTILITIES</b>	DRAWN BY: <b>EA</b>	DATE: <b>5/20/81</b>	PROP. NO.: <b>AN95160</b>
LOCATION: <b>RIVER BEND UNIT 1</b>	BWIP International, Inc.		DRAWING NO.: <b>SK01</b>
ENGINEER:	Byron Jackson Pump Division		
PUMP SIZE AND TYPE: <b>18 RVL-1 stage VCT</b>			

PART LIFT		ORDER NO.		REV.	
UNIT		OUR ORDER NO.		DRAWING NO.	
JOB		CUSTOMER ORDER NO.		PATTERSON PUMP COMPANY	
PUMP		CAPACITY		TOCCOA GA	
MOTOR		MAKE		X-30 MAX	
HOURS		PHASE		2000 HP MOTOR	
SPEED		RPM		REV. MIN.	
CONTROL		DATE		PART NO.	
CERTIFIED BY				PART	
				M	



COVER WT - 4,750 #  
 LOWER HALF KIT - 11,800 #  
 ROTOR KIT - 2,450 #  
 TOT PUMP WT - 19,000 #

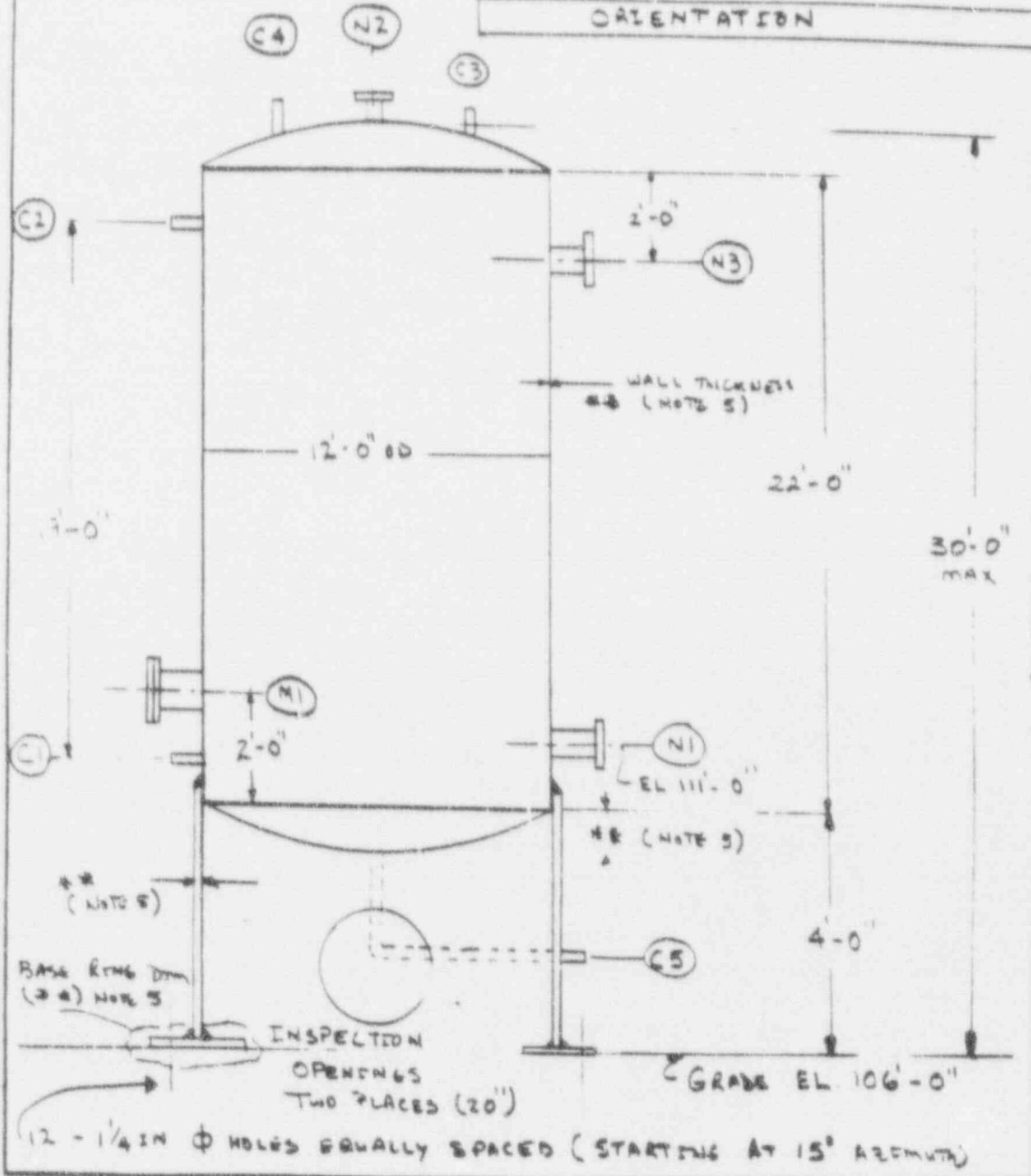
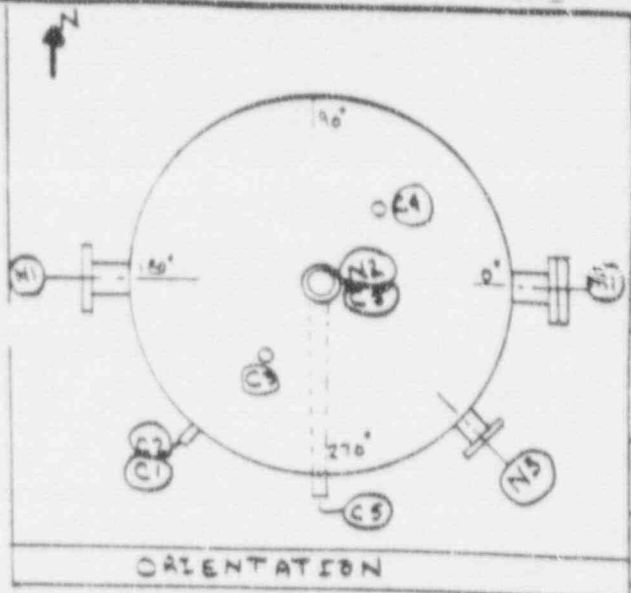


125# 30" DISCHARGE  
 39" O.D. FLANGE w/  
 (28) - 1/2" DIA BOLTS ON  
 36" D.B.C.

3/4" DIA. DRAIN (SEE P. 25-C)

ALL DIMENSIONS ARE APPROXIMATE, HOWEVER. NOMINAL CAPACITY AND MAXIMUM HEIGHT SHALL BE MAINTAINED.

PROPOSED TANK APPROX CAPACITY: 44,000 GALS



## COOLING TOWER

- Multiple cell, mechanical draft type
- Five individual concrete cells measuring approximately 50 feet square; the size is based on maximum total heat load
- Each tower to have single fan with two speed motors for efficiency (100 - 200 HP)
- GSU has good operating experience with this type cooling tower



240' 0" TOWER HEIGHT

220' 0" TOWER LENGTH

44' 0" TYP CELL LENGTH

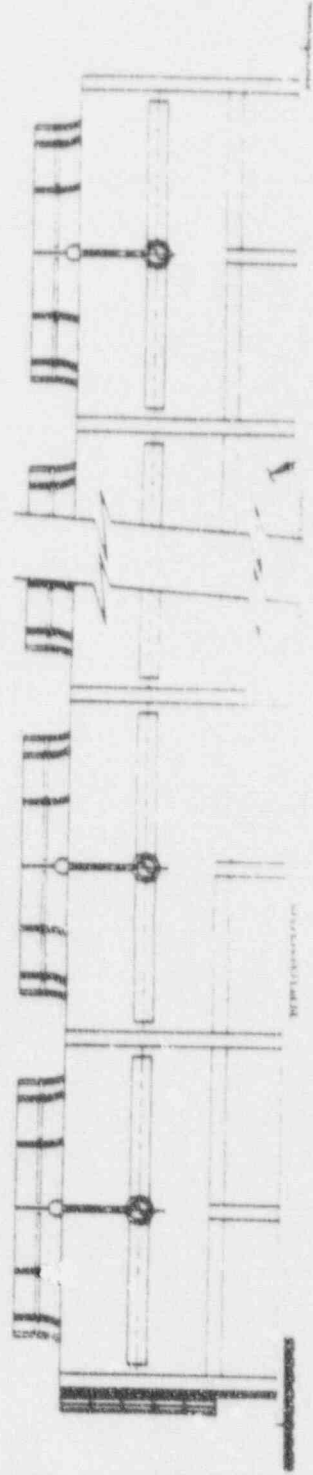
52' 0" TOWER WIDTH

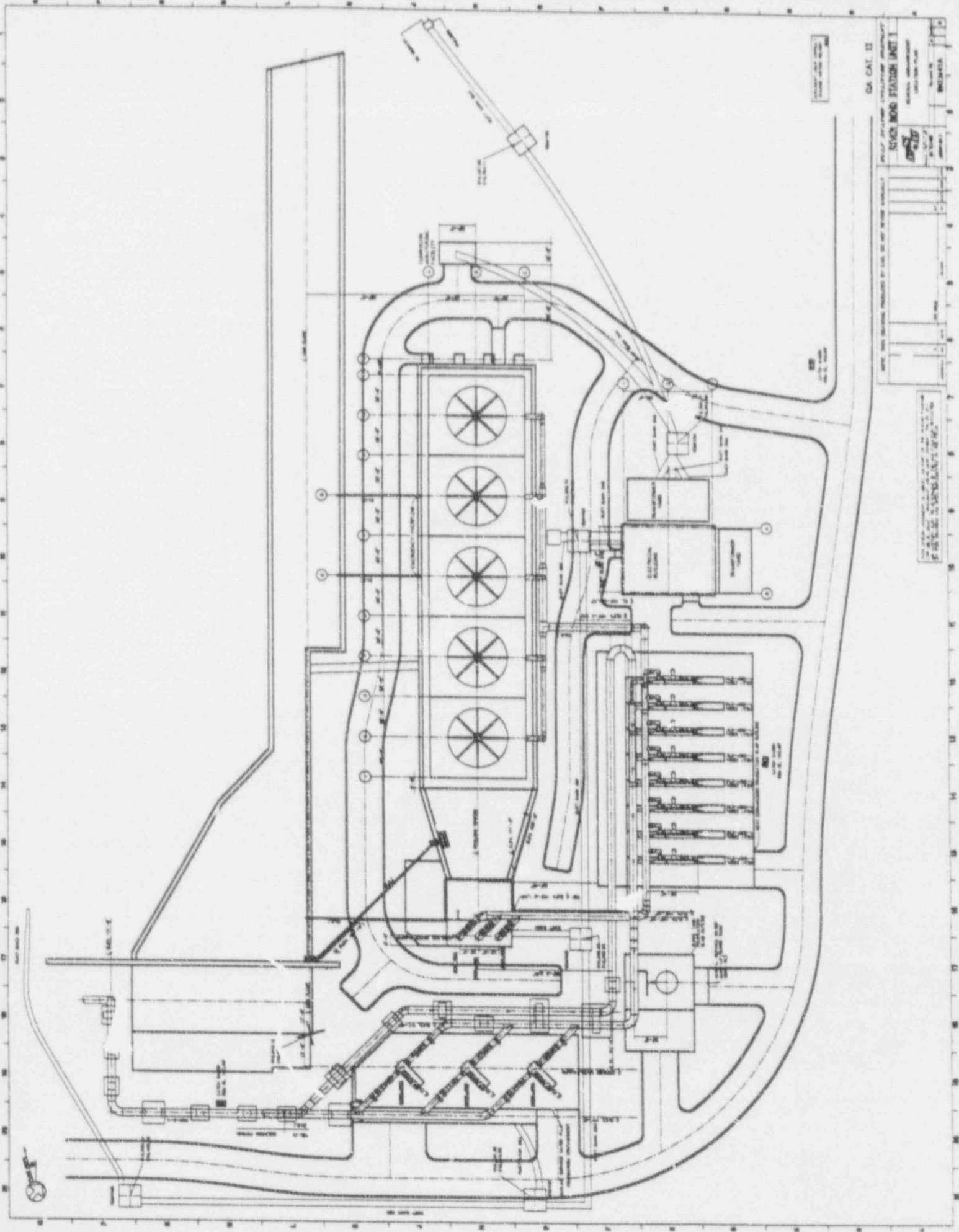
60' 0" INSIDE BASIN

HOT WATER INLET  
26" DIA, 125 LB FLANGE

CELL ACCESS HATCH  
TYP

PLAN VIEW





## TESTING/TURNOVER STATUS

SYSTEM	DESCRIPTION
SWC-001	<u>13.8KV transformers and 4.16KV switchgear:</u> Protective relaying, Station Service transformers 1STX-XS5A + 5B, Switchgear 6A + 6B
SWC-002	<u>Batteries and Chargers:</u> BUS battery and charger
SWC-003	<u>480V transformers, load centers and motor starters:</u> Protective relays, Switchgear breakers, Transformers, Cooling tower fan motors
SWC-004	<u>480V Motor control center:</u> None
SWC-005	<u>Cooling tower fans:</u> Fans 1A-1E
SWC-006	<u>Cooling tower:</u> Cooling tower valves, Cooling tower cells 1A-1E, Cooling tower flume, Cooling tower makeup water
SWC-007	<u>Heat Exchangers:</u> Strainer MOVs and diverter actuators, SWC heat Exchanger inlet and outlet temp inst,
SWC-008	<u>Cooling tower pumps:</u> Instruments for calibration
SWC-009	<u>Chemical Feed and injection:</u> None
SWC-010	<u>Multiplexer:</u> None
SWP-001	<u>Normal service water:</u> SWP temperature monitoring instruments



**GENERIC LETTER 89-10  
MOV PROGRAM  
RF-4 PREPARATIONS**

**RIVER BEND STATION - UNIT 1  
GULF STATES UTILITIES COMPANY**

**NRC PRESENTATION  
MARCH 4, 1992**

## **BACKGROUND**

- **PREPARATIONS DURING CYCLE 4**
  - **DESIGN BASIS REVIEWS**
  - **PURCHASE VOTES TEST EQUIPMENT**
  - **PROCEDURE UPGRADE**
  - **SPARE PARTS**
  - **TRAINING**

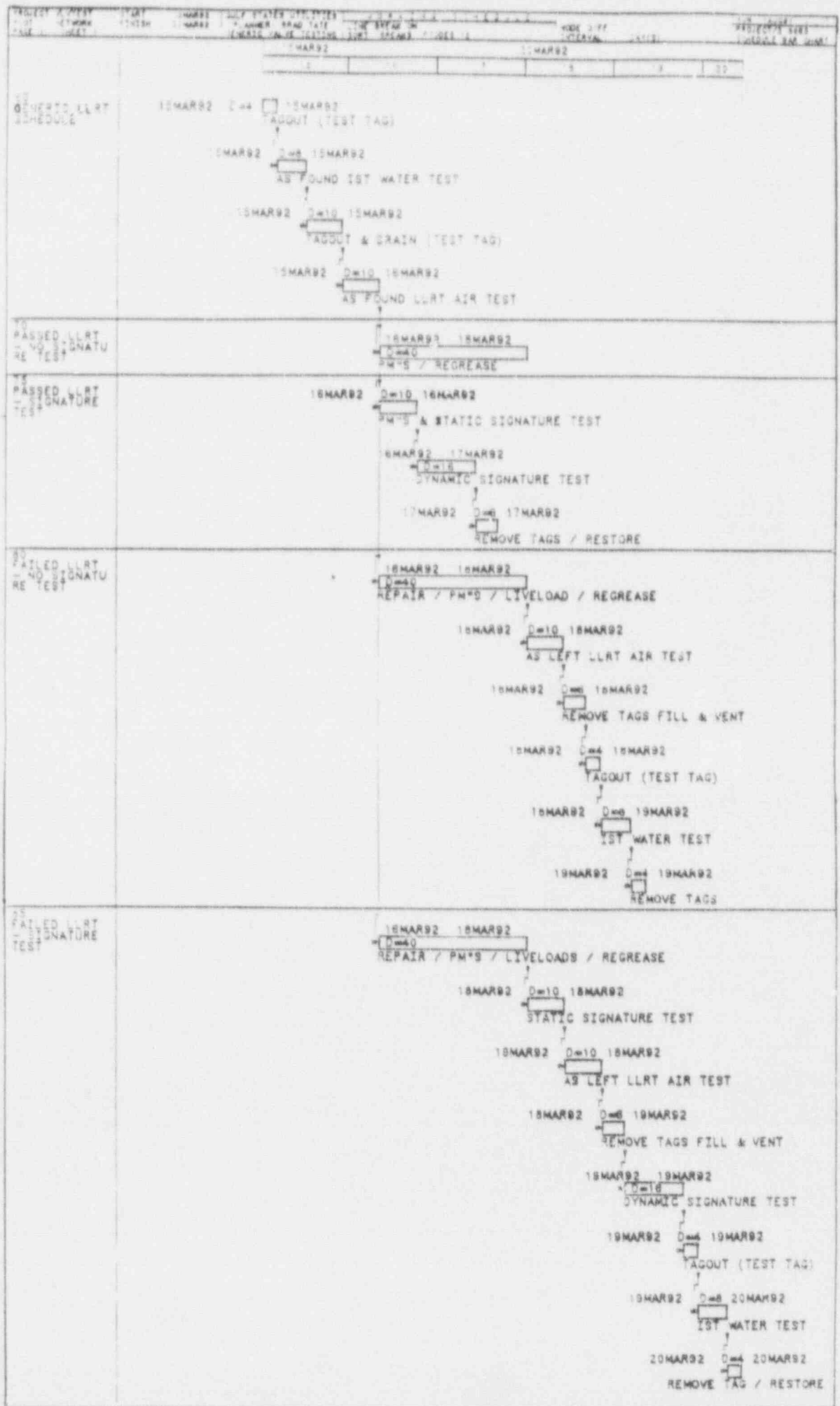
- **NRC INSPECTION DECEMBER 9-13, 1991**
  - **INSPECTORS CONCLUDED THAT GSU'S GL 89-10 PROGRAM REPRESENTED A GOOD INITIAL EFFORT TO MEET THE INTENT OF THE GENERIC LETTER.**
  - **GSU HAS ALLOCATED SIGNIFICANT RESOURCES IN BOTH MANPOWER AND EQUIPMENT TO DEVELOP THE PROGRAM**
  - **SEVERAL WEAKNESSES WERE IDENTIFIED, WHICH WHEN PROPERLY RESOLVED SHOULD ENSURE AN ACCEPTABLE PROGRAM WILL BE IN PLACE**
- **RESPONSE TO NRC INSPECTION REPORT 91-24**
  - **DUE MARCH 22, 1992**
  - **WILL ADDRESS EACH OF THE SEVEN WEAKNESSES**
  - **REPRESENTS A SIGNIFICANT EFFORT**

## TOTAL SCOPE

- 260 SAFETY RELATED MOV'S
- 4 OUTAGES (RF-3, 4.5.6)

## RF-4 SCOPE

- 115 MOV'S PLANNED FOR STATIC TESTING
- OF THOSE, APPROXIMATELY 60 WILL BE FLOW TESTED AT MAX ACHIEVABLE FLOW/PRESSURE
- APPROXIMATELY 40 MOV'S PLANNED FOR REFURBISHMENT PRIOR TO DIAGNOSTIC TESTING



## MOV TEST SEQUENCE

- TAGOUT (TEST TAG)
- AS-FOUND LLRT WATER TEST
- TAGOUT & DRAIN
- AS-FOUND LLRT AIR TEST
- VALVE REPAIR
- OPERATOR OVERHAUL
- LIVELOAD PACKING
- PREVENTIVE MAINTENANCE
- STATIC SIGNATURE TEST
- AS-LEFT LLRT AIR TEST
- REMOVE TAGS, FILL & VENT
- DYNAMIC SIGNATURE TEST
- TAGOUT (TEST TAG)
- IST WATER TEST
- REMOVE TAG/RESTORE



## RF-4 MOV TEST ORGANIZATION

- 3 CREWS, EACH WITH AN EXPERIENCED GSU LEAD IN CHARGE
- DEDICATED STAFF OF TEST ENGINEERS AND TECHNICIANS ASSIGNED TO PPT
- TRAINED IN BOTH ENGINEERING AND MAINTENANCE FUNCTIONS
- BENEFITS
  - QUALITY - PROGRAM EMPHASIS ON QUALIFICATIONS AND TRAINING
  - CONSISTENCY - ONE GROUP RESPONSIBLE FOR GL 89-10 ACTIVITIES
  - EFFICIENCY - WORK PACKAGES DO NOT HAVE TO TRANSFER FROM GROUP TO GROUP
- CLOSE INTERFACE WITH SYSTEM ENGINEERING VALVE COORDINATOR
- DESIGN ENGINEERING SUPPORT
  - CALCULATION PREPARATION AND REVIEW
  - ONSHIFT OUTAGE COVERAGE
  - REVIEW/FEEDBACK/TREND TEST RESULTS

## CONCLUSIONS

- GL-89-10 IS A VALID PROGRAM
- GSU HAS A WELL MANAGED & DEDICATED MOV TEAM
- NRC INSPECTION INDICATED GSU'S PROGRAM IS A GOOD INITIAL EFFORT TO MEET THE INTENT OF GL-89-10
- MOV TESTING, INSPECTION AND REFURBISHMENT REQUIRES AN INTEGRATED AND DETAILED APPROACH
- GSU IS DEDICATED TO A GOOD FAITH EFFORT TO TEST 115 VALVES DURING RF-4



**RADIATION  
SOURCE TERM  
REDUCTION**

**RIVER BEND STATION**

**MARCH 1992**

## OBJECTIVE

Reduce the Source Term as necessary  
to Minimize Man - Rem Expenditures consistent  
with River Bend ALARA Goals.

## OVERALL GOAL

Reduce RBS Man - Rem Expenditures  
to the BWR Industry Goal of 255 Man - Rem  
for a 3 Year Average  
by 1995.

## ACTION REQUIRED

Reduce current Man - Rem Expenditure by 30%.

# SOURCE TERM REDUCTION PROGRAM OVERVIEW

Chemical Decontamination

RWCU Ring Header Replacement

-----<>-----

Maximize Pre - Startup Condensate Cleaning Duration  
with Condenser Vacuum

Soft Shutdown with Maximized RWCU Availability

Temporary Shielding and Component Flushing

-----<>-----

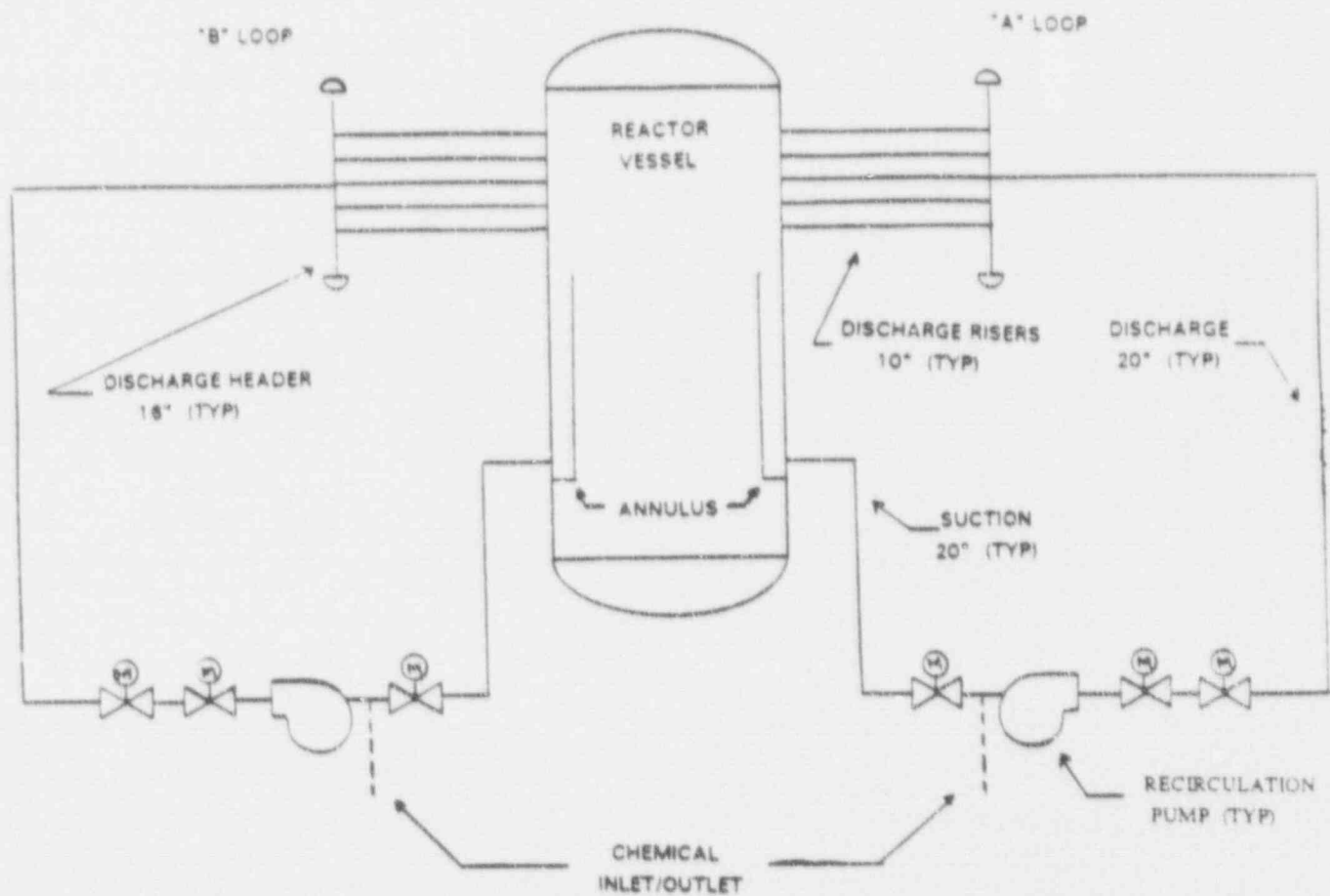
Stellite (Cobalt) Reduction

Low Cobalt Parts Program

Use of NOREM for Valve Repair

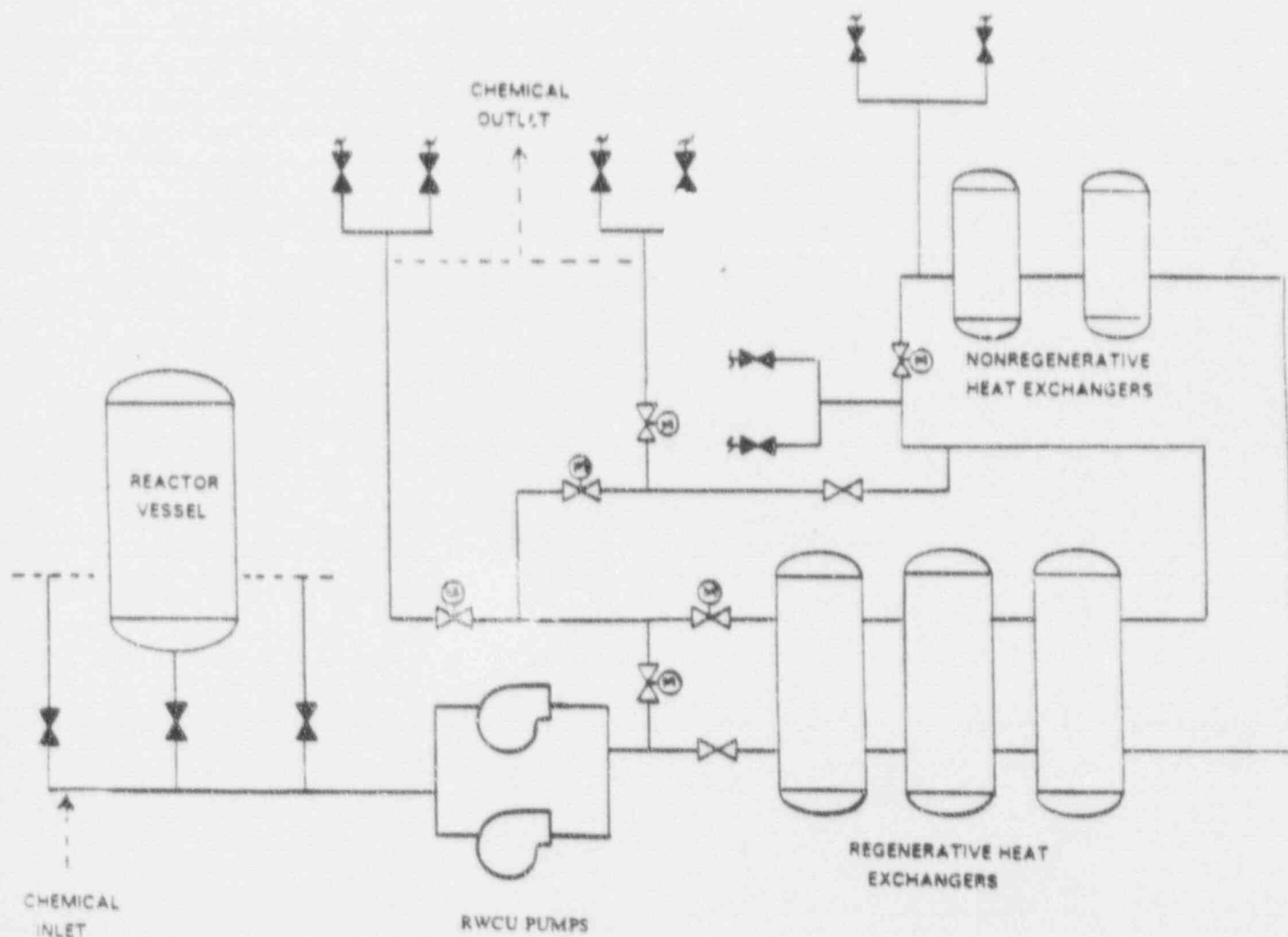
Condensate Filter Addition Study

# CHEMICAL DECONTAMINATION REACTOR RECIRCULATION SYSTEM



Low Oxidation - State Metal Ion (LOMI) Process  
Non - Regenerative (Batch) Type Process  
Best for Systems with Small Surface Area  
to Volume Ratio  
Approved for Use in the Reactor Vessel  
Decontamination Factor - 10 Target

# CHEMICAL DECONTAMINATION REACTOR WATER CLEANUP SYSTEM



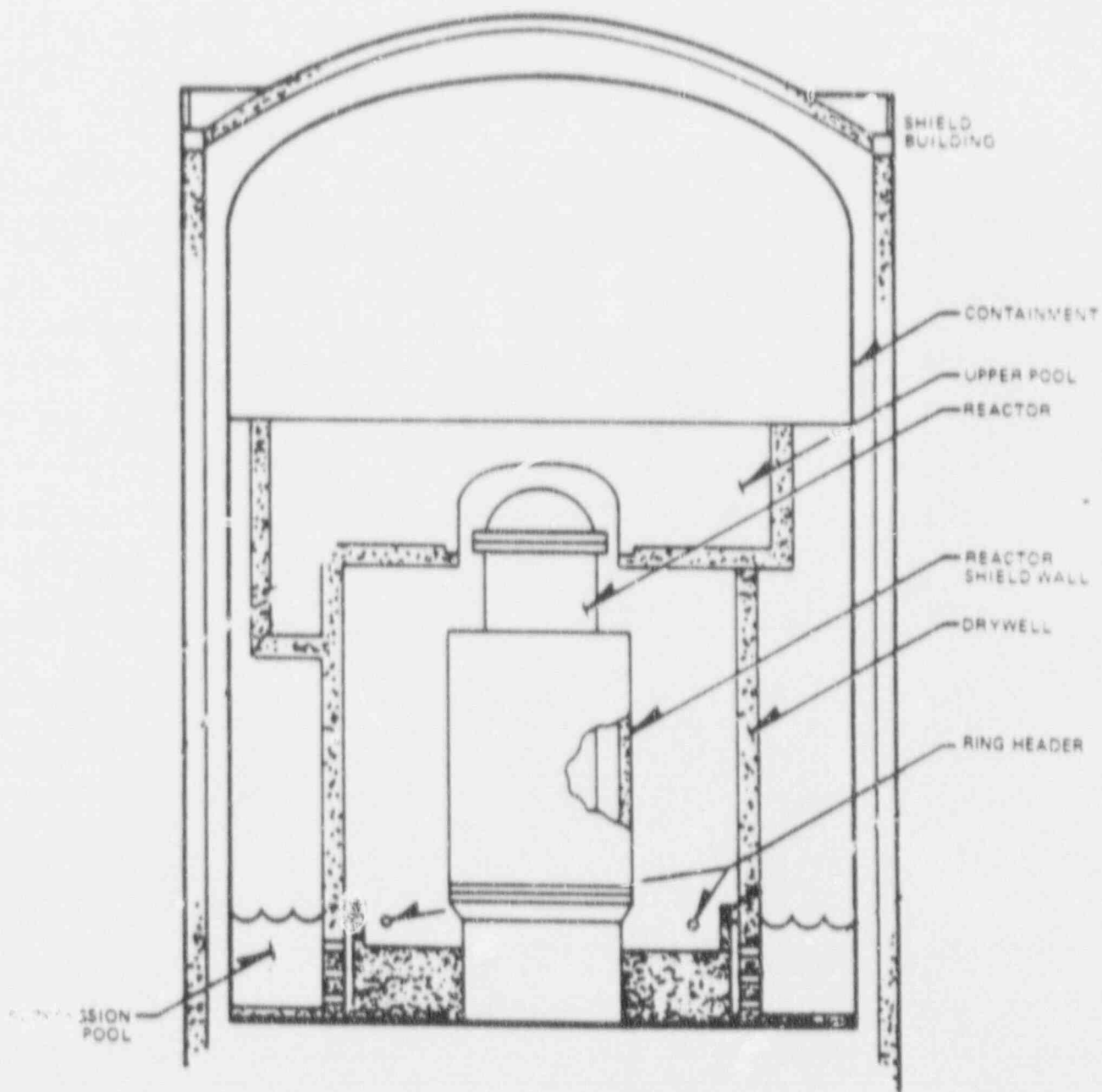
Can - Derem Process

Regenerative (Continuous) Type Process

Best for Systems with Large Surface Area  
to Volume Ratio

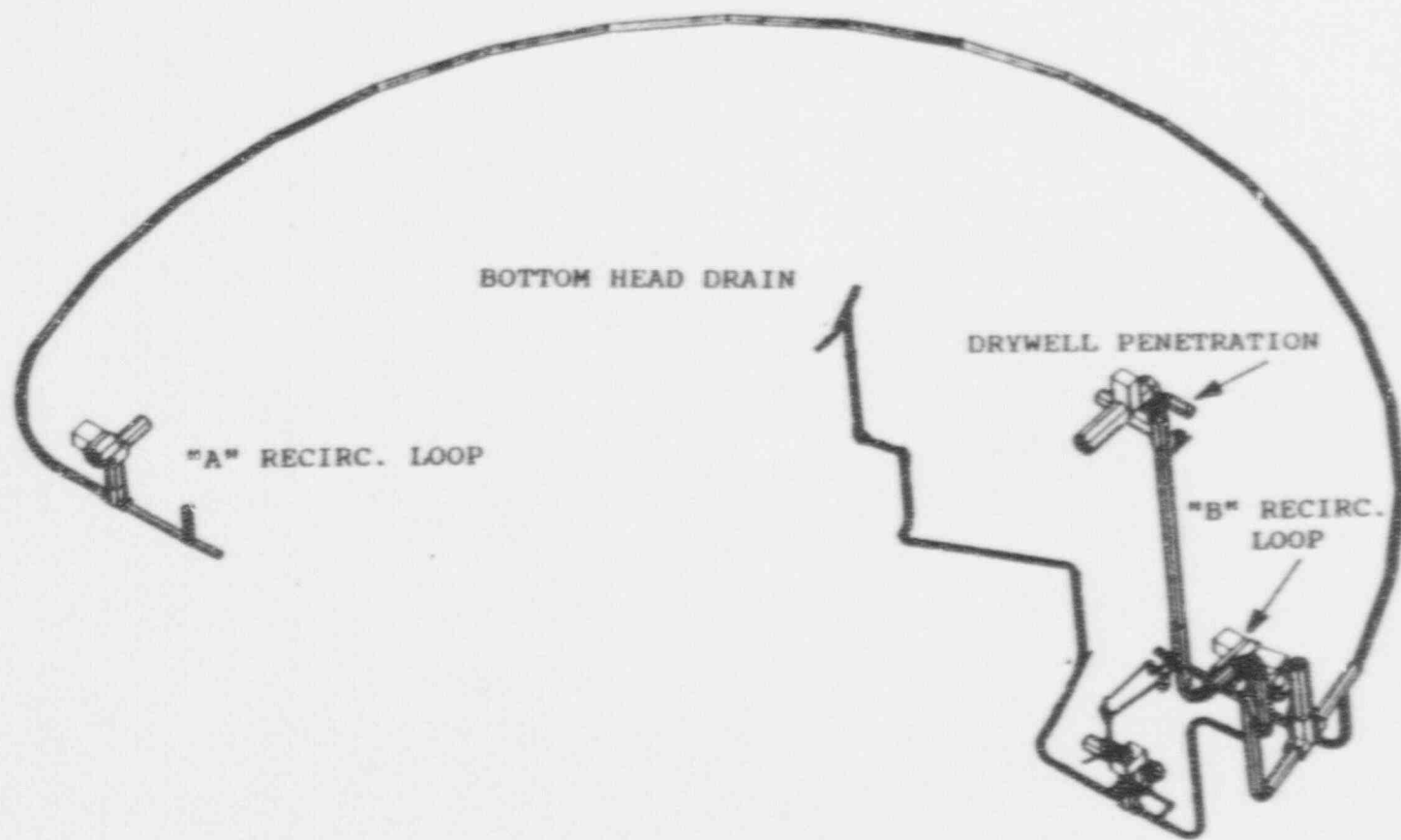
Decontamination Factor - 10 Target

# RWCU PIPE REPLACEMENT GENERAL LOCATION



*Reactor Building (Mark III) Containment and Shield Building)*





RBS RWCU RING HEADER

110.

MATERIAL SELECTION  
AND PROCESSING

Selected 316L Stainless Steel

Corrosion Resistant

Not Susceptible to IGSCC

**Corrosion Resistance Enhancements:**

Finished Spool Pieces will be ELECTROPOLISHED.

Finished Spool Pieces will be PREOXIDIZED.

**IGSCC Resistance Enhancements:**

All Piping will be SOLUTION ANNEALED  
following Fabrication.

Field Welding will be Performed using  
HEAT - SINK WELDING.

Meets and Exceeds NUREG 0313 Requirements for  
Category "A" Weld Inspection.



## BENEFITS OF REPLACEMENT

Eliminates Approximately 5 Whip Restraints

Eliminates Approximately 21 Seismic Snubbers

Reduced Maintenance and Inspection Requirements

Reduced Shielding Requirements

Removes "Dead Legs" and "Hot Spots"

Adds Permanant Chemical Decontamination Connections

Cobalt Reduction (No - Cobalt Valves)

Low Recontamination Rate

(Approximately 100 mRem/hr Maximum)

Adds Maximum Support Jacket Insulation

## CONCLUSION

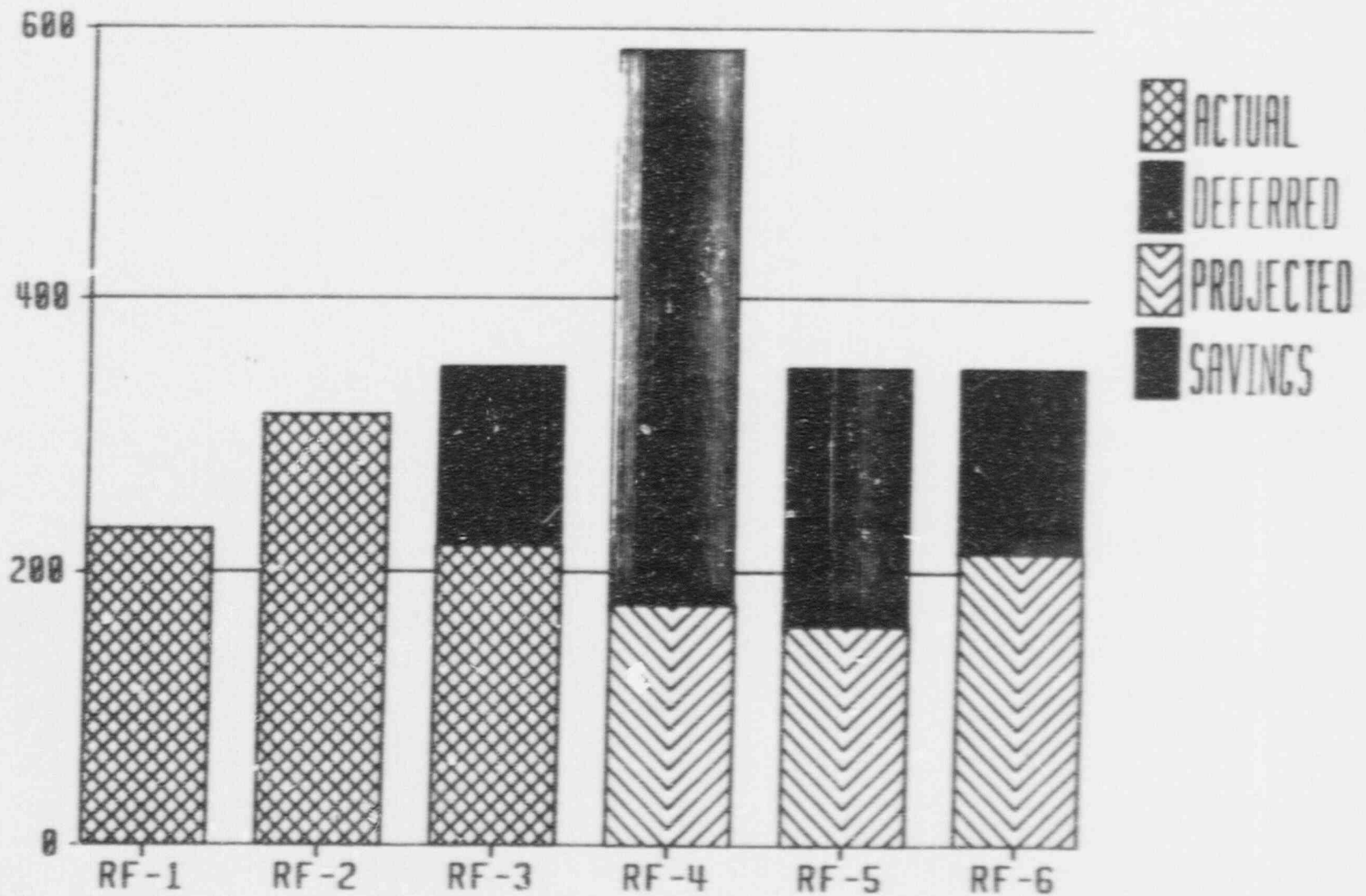
Savings of over 650 Man - Rem  
Projected through 1995

Developed Comprehensive Program

Established Aggressive Goal

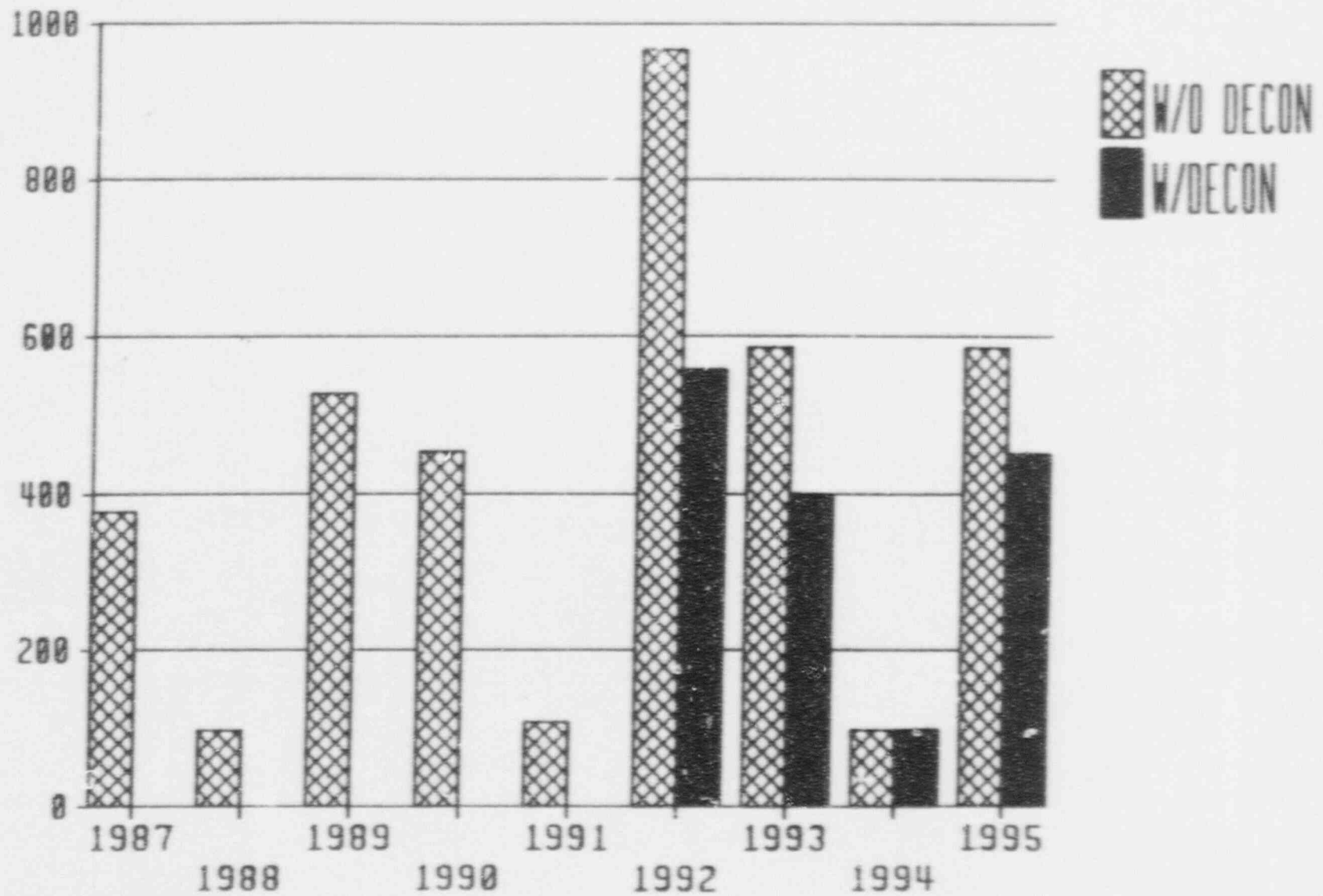
Ongoing Team Effort

# RBS DRYWELL MAN-REM



f11

# RBS MAN-REM EXPENDITURE



## FEEDWATER NOZZLE REPAIR

- o INTRODUCTION
- o EXISTING SAFE END/THERMAL SLEEVE DESIGN CHARACTERISTICS
- o INDICATION FOUND IN FEEDWATER NOZZLE N4A SAFE END TO NOZZLE WELD DURING RF-2 IN MARCH 1989
  - o INDICATION HAS BEEN MONITORED DURING REFUELING AND MIDCYCLE OUTAGES SINCE DISCOVERY
  - o FLAW ANALYSIS WAS PERFORMED FOLLOWING EACH INSPECTION
  - o GROWTH RATE HAS BEEN CONSISTENT WITH PREDICTIONS FOR IGSCC FLAW
  - o PREDICTIONS SHOW SIGNIFICANT MARGIN WILL EXIST AT RF-4
- o SEVERAL REPAIR OPTIONS WERE CONSIDERED
  - o EXCAVATE WELD TO REMOVE DEFECT
  - o WELD OVERLAY
  - o LIKE FOR LIKE REPLACEMENT
  - o REPLACE WITH MODIFIED SAFE END AND THERMAL SLEEVE CONFIGURATION
- o MODIFIED DESIGN WAS DEVELOPED
  - o INSTALLATION TO BE IN ACCORDANCE WITH REPAIR REPLACEMENT PROVISIONS OF ASME SECTION XI
  - o NEW DESIGN SPECIFICATION ISSUED BY GSU
  - o COMPETITIVE BIDS SOLICITED FROM QUALIFIED DESIGN FIRMS
  - o CONTRACT AWARDED TO GE TO PROVIDE DESIGN AND ANALYSIS IN ACCORDANCE WITH DESIGN SPECIFICATION
  - o MATERIAL PROCUREMENT HANDLED BY GSU

- MODIFICATION DESIGN DETAILS
  - NEW SAFE END IS A DOUBLE THERMAL SLEEVE "TUNING FORK" CONFIGURATION
    - REQUIRES MINIMAL IN VESSEL WORK
    - ELIMINATES SEAL LEAKAGE
    - PROVIDES ADEQUATE THERMAL PROTECTION FOR THE RPV NOZZLE
    - UTILIZES EXISTING SPARGER
  - DESIGNED IN ACCORDANCE WITH ASME III, 1986
  - MATERIALS SELECTION BASED ON NUREG-0313 REV 2
    - NO IGSCC SUSCEPTABLE MATERIAL
    - ISI FREQUENCY REDUCED
  - POST WELD HEAT TREATMENT
- SUMMARY OF ENGINEERING PRESENTATION
  - SIGNIFICANT PROJECT FOR GSU
  - PERMANENT FIX
  - MEETS ALL DESIGN REQUIREMENTS FOR REACTOR PRESSURE VESSEL
  - REDUCED ISI FREQUENCY RESULTS IN EXPOSURE SAVINGS
- MAINTENANCE PRESENTATION  
(BY GARLAND MAHAN)



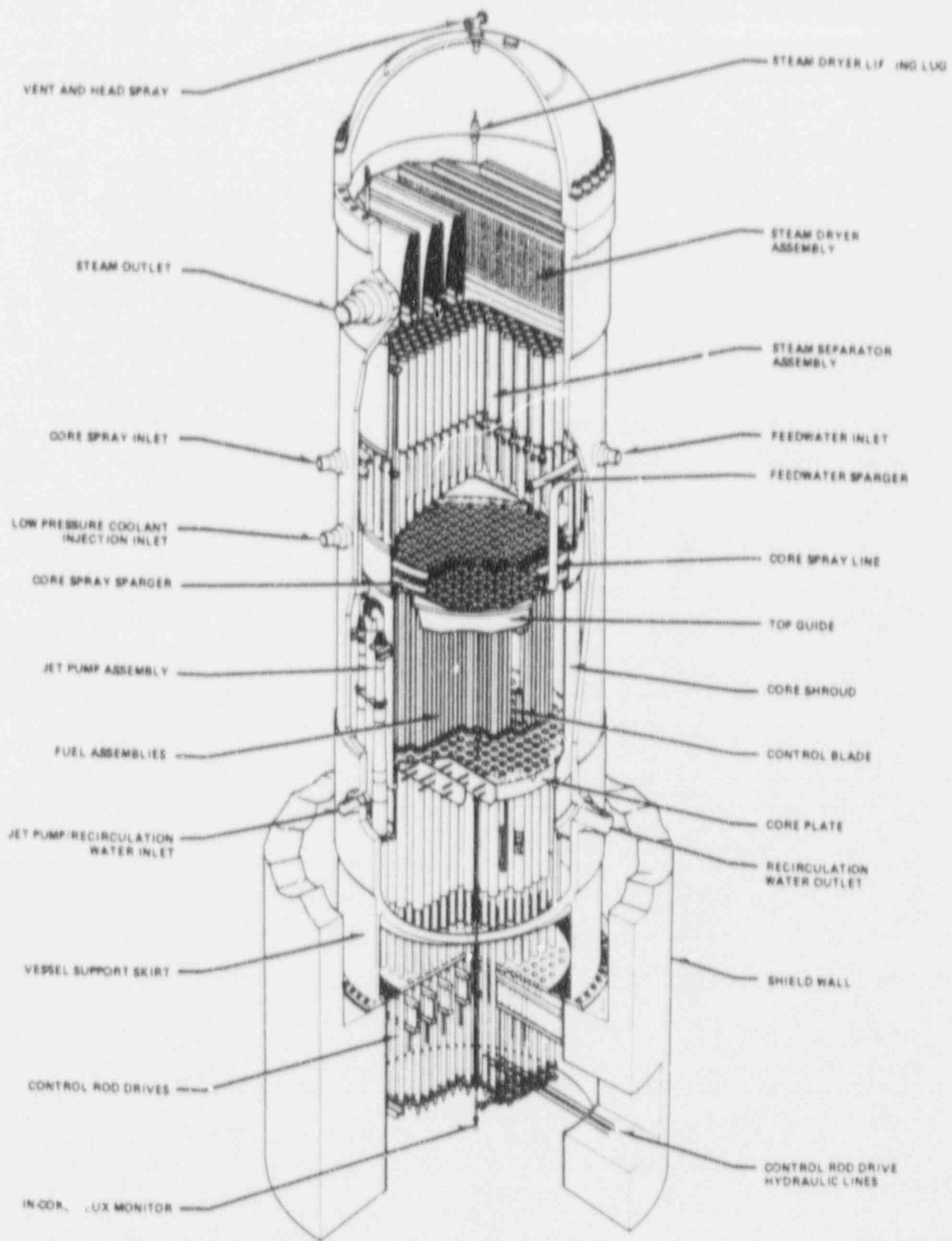
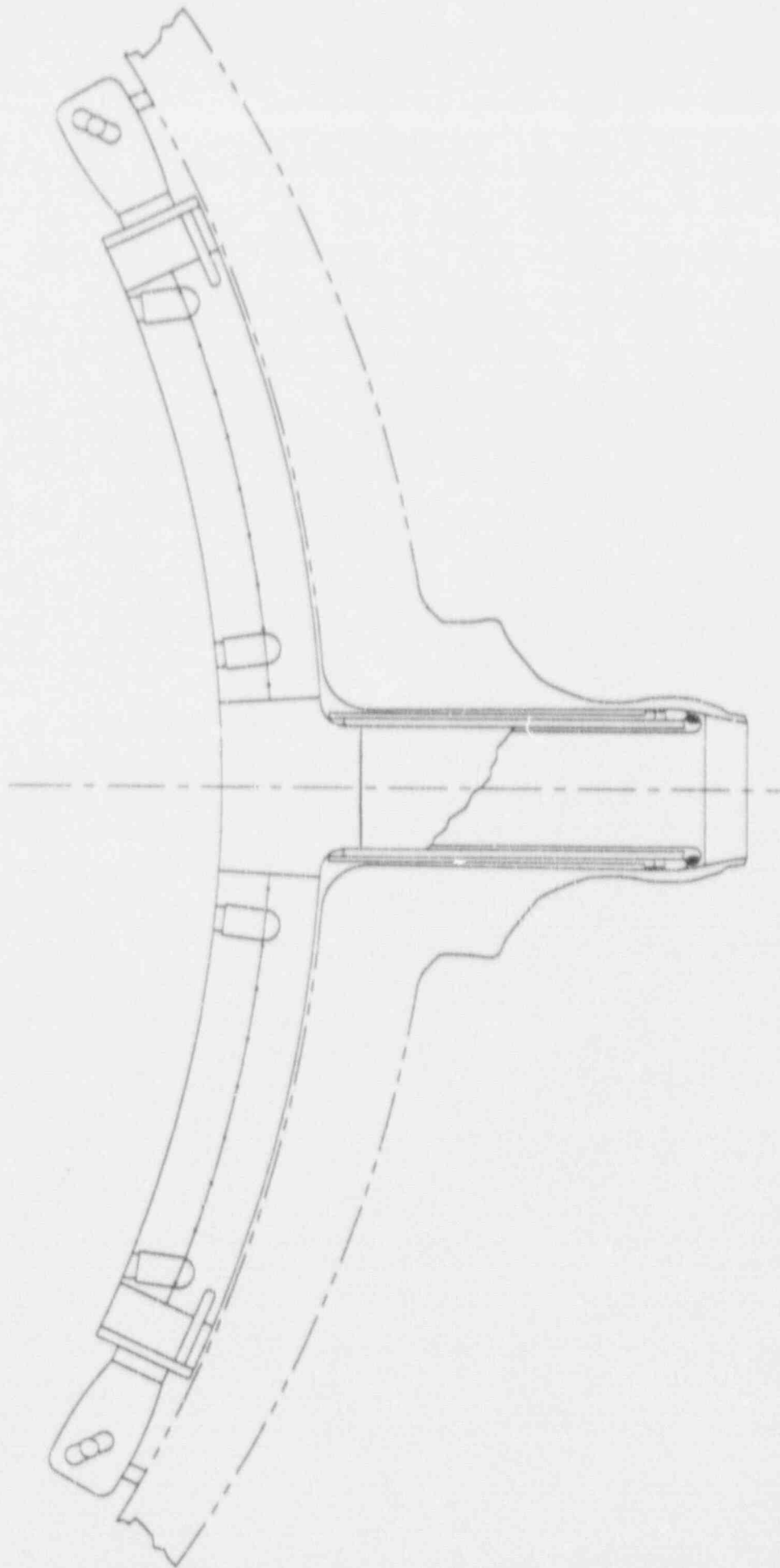
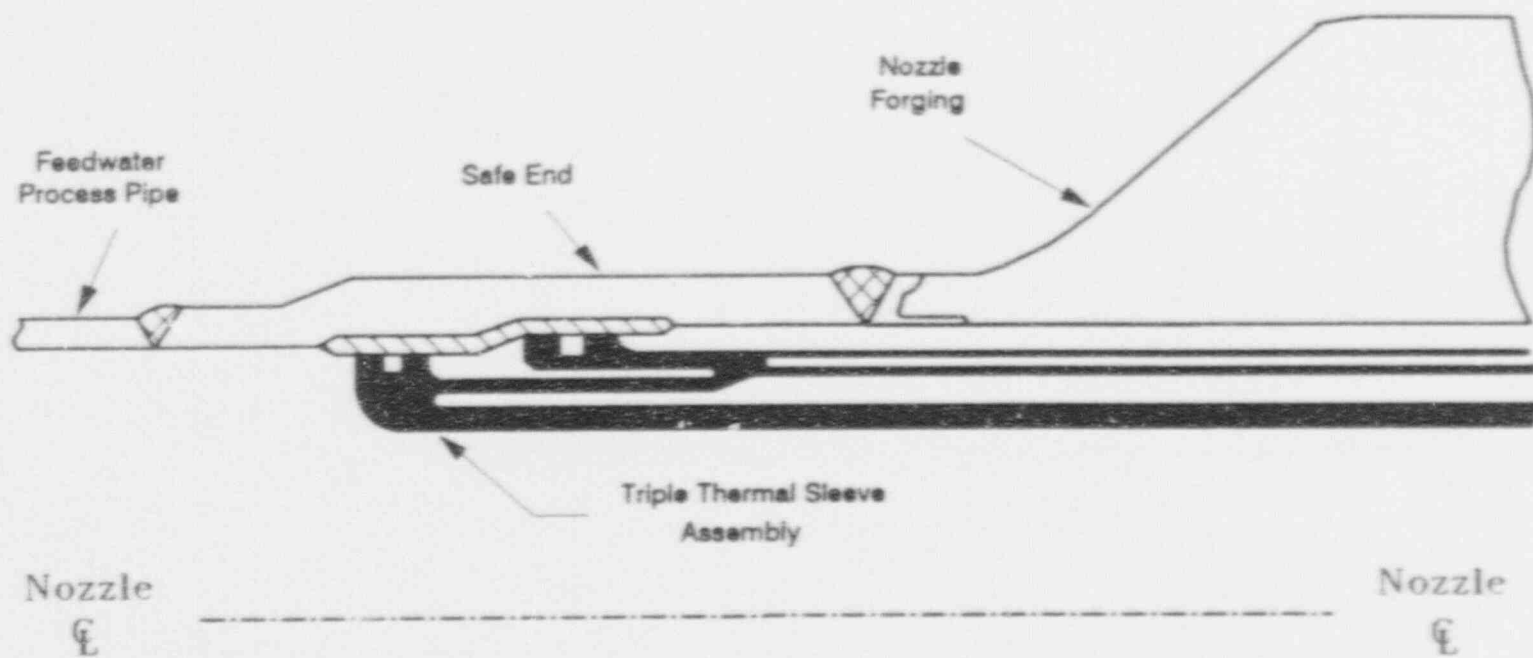


Figure 2-1. Reactor Assembly





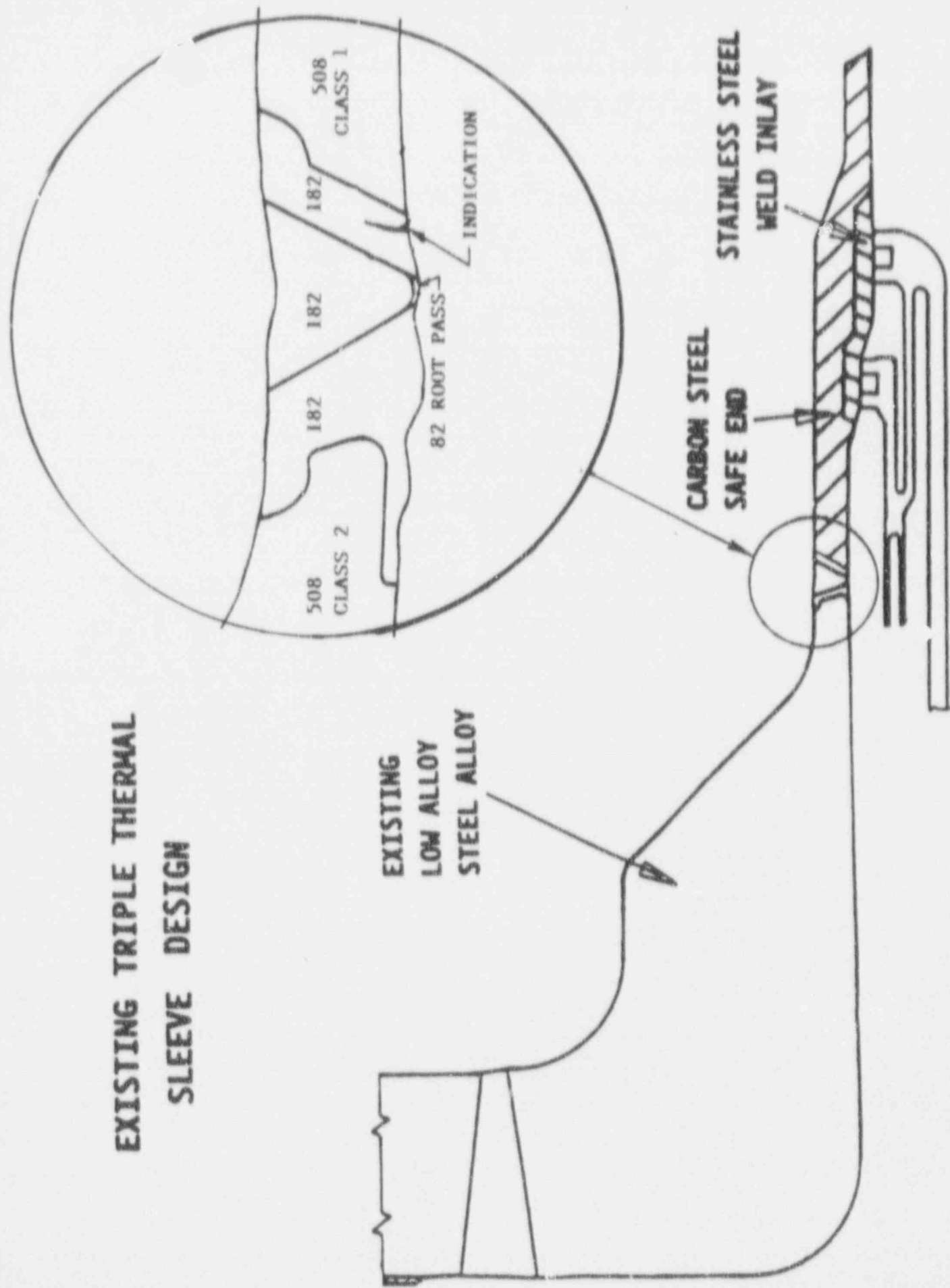
FEEDWATER SPARGER  
PLAN VIEW



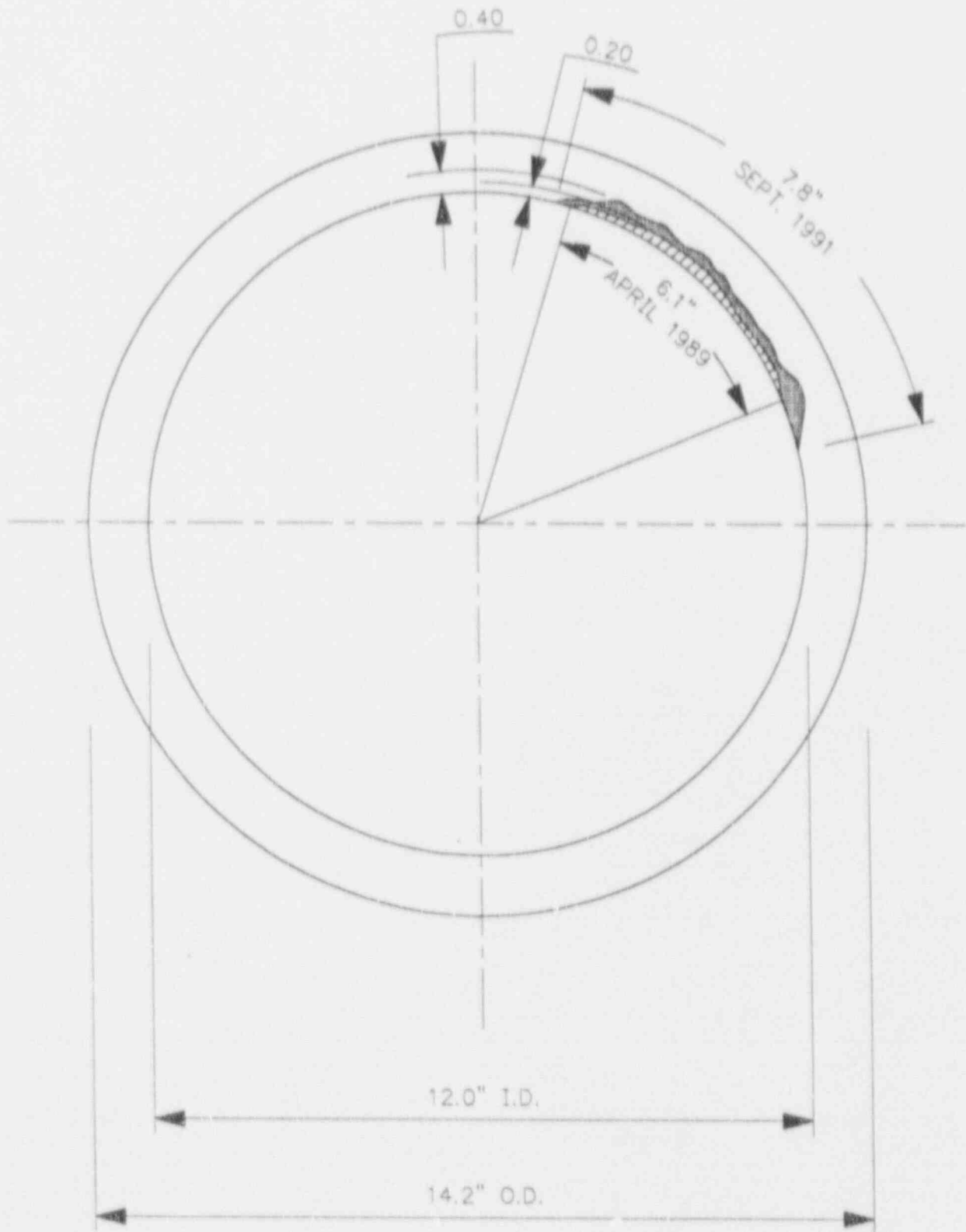
Current Existing Feedwater Safe End Design

125

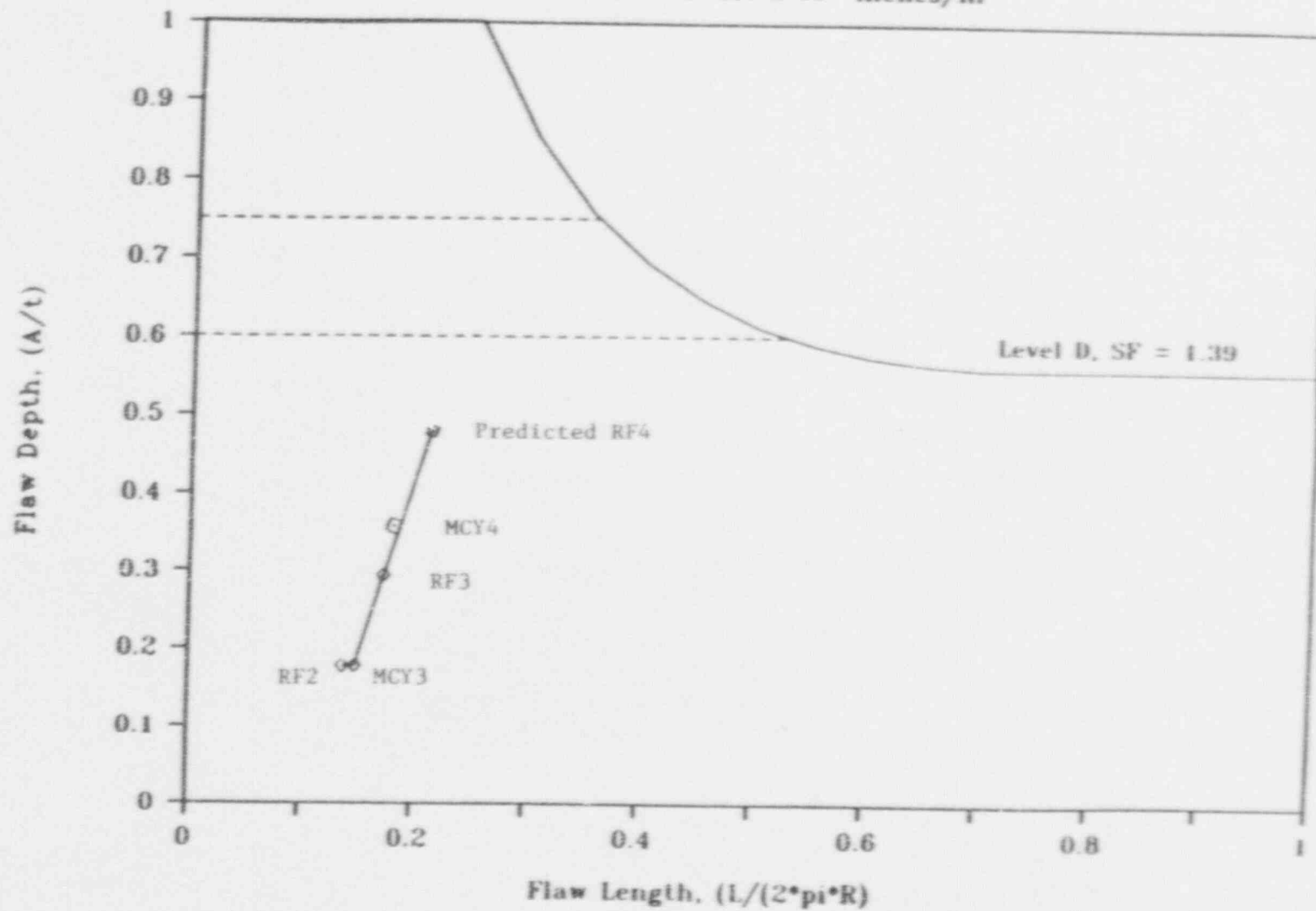
# EXISTING TRIPLE THERMAL SLEEVE DESIGN



# NOZZLE N4A CRACK PROGRESSION

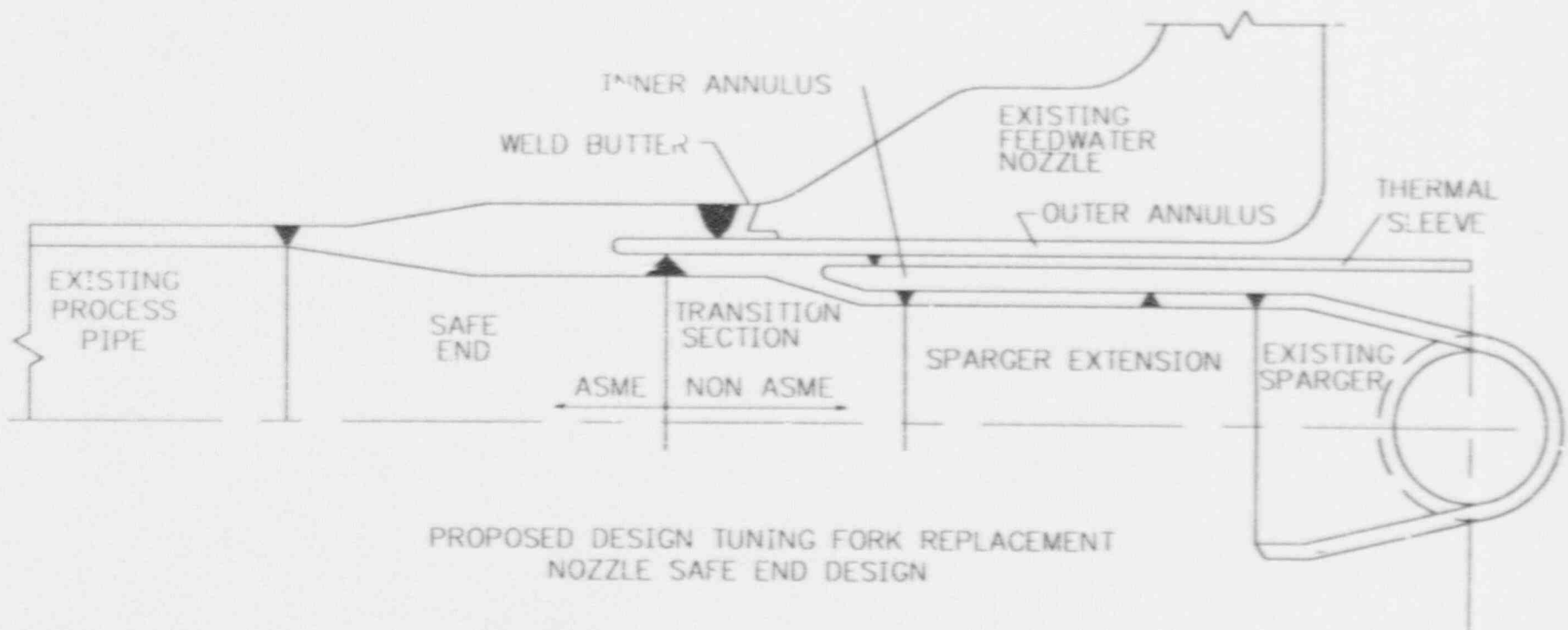


Level D Condition Flaw Assessment Diagram, River Bend - Feedwater Nozzle  
 Crack Growth Rate of  $3.0 \times 10^{-5}$  inches/hr



Flaw Assessment Diagram for Level D Conditions Based  
 on  $3 \times 10^{-5}$  in/hour Crack Growth Rate

123



PROPOSED DESIGN TUNING FORK REPLACEMENT  
NOZZLE SAFE END DESIGN

MATERIALS

SAFE END SA-508 CLASS 1

TRANSITION SECTION SA-508 CLASS 1

THERMAL SLEEVE SA-106 GR B

127

**N 4 A**

**NOZZLE**

**SAFE END**

**REPLACEMENT**



# LOCATION

ELEVATION 141' LEVEL

AZIMUTH 45°

# TRAINING

## INITIAL TRAINING

PROCEDURE REFINEMENT

EQUIPMENT CHECKOUT

ALARA REVIEW

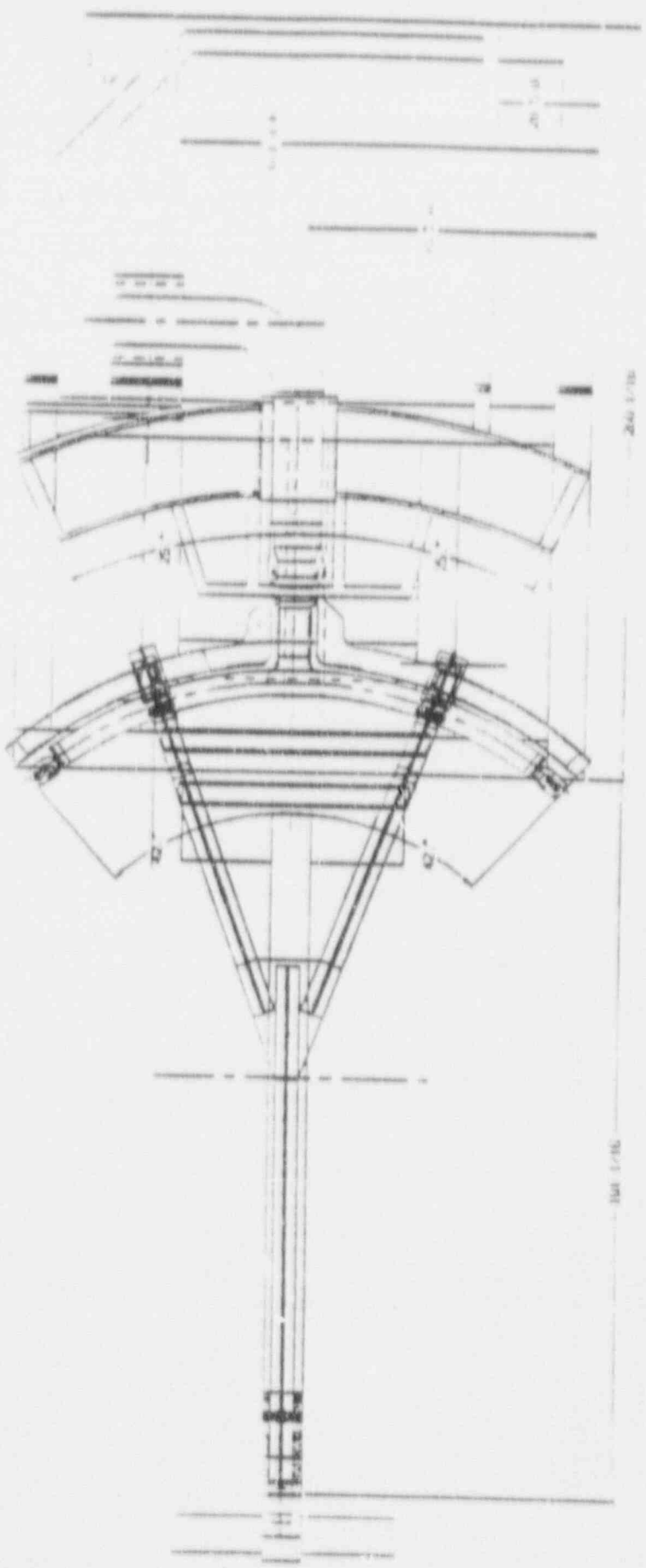
RADIOLOGICAL PROTECTION  
FAMILIARIZATION

OQC

## FINAL TRAINING

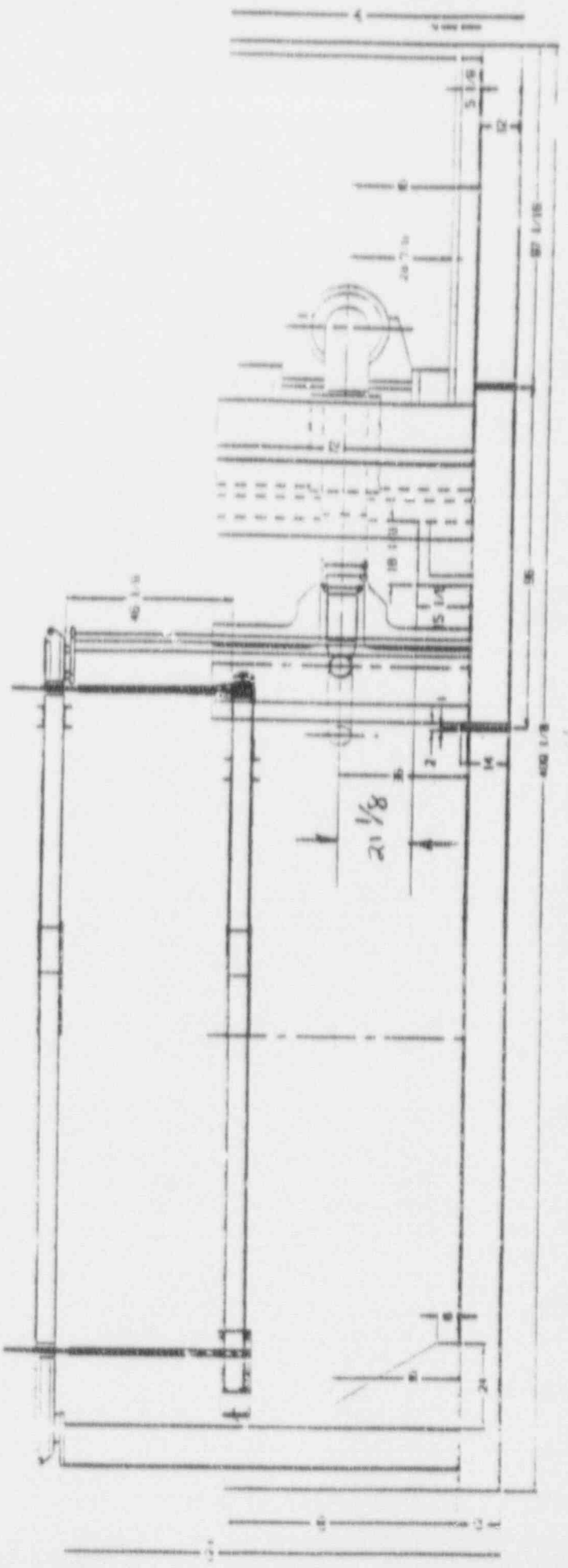
COMPLETE DRY RUN  
SIMULATING ALL DRYWELL CONDITIONS

ALL CRAFTS TO BE INVOLVED



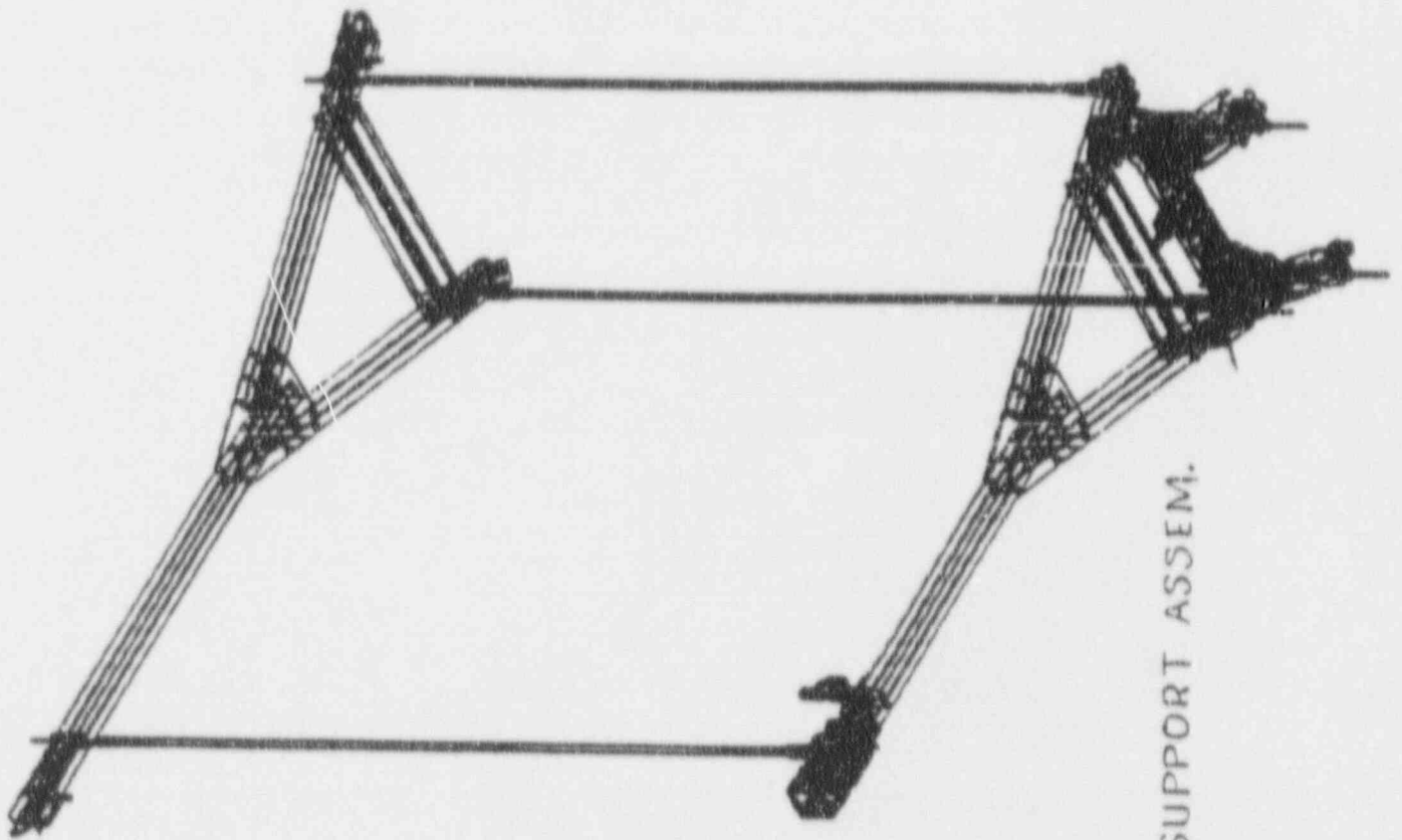
240 1/16

104 1/16

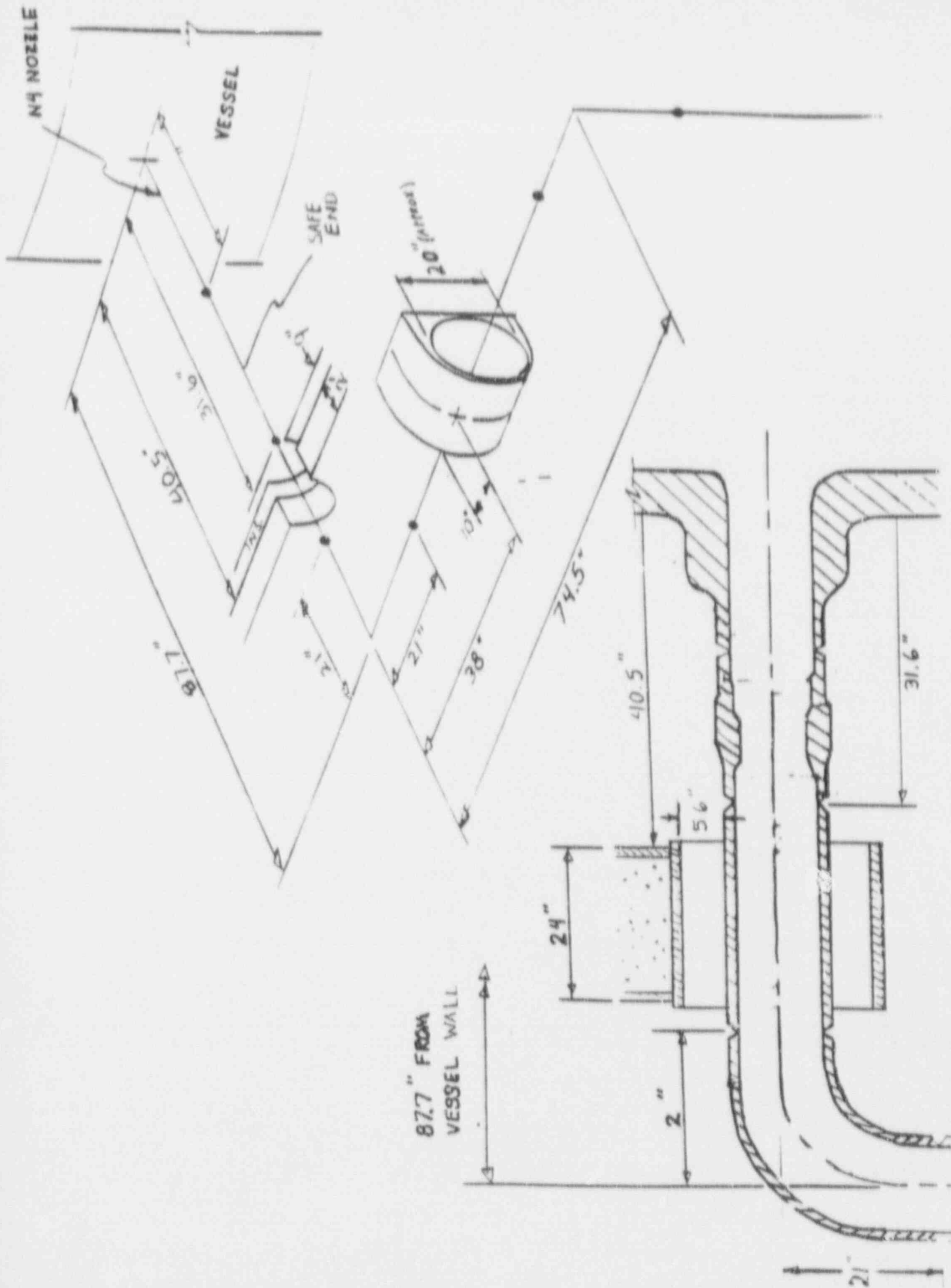


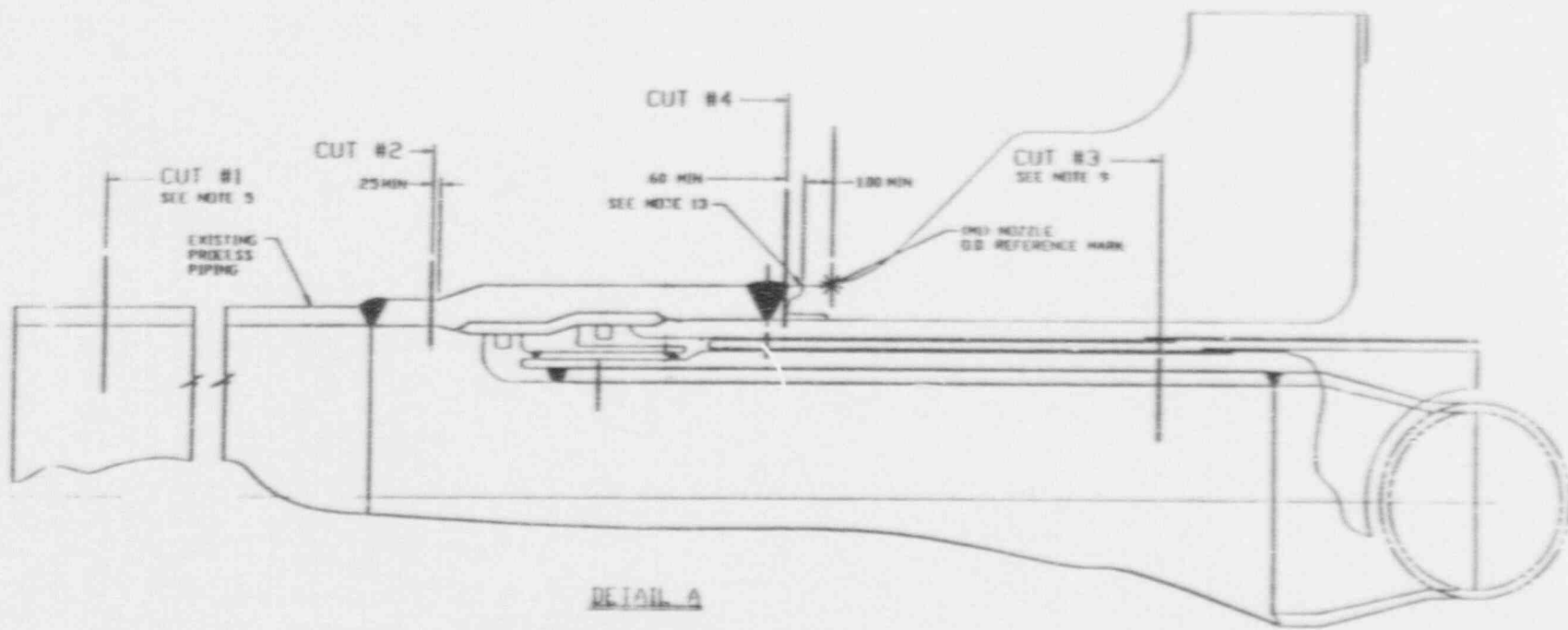
$2\frac{1}{8}$

# REPLACEMENT



SPARGER SUPPORT ASSEM.





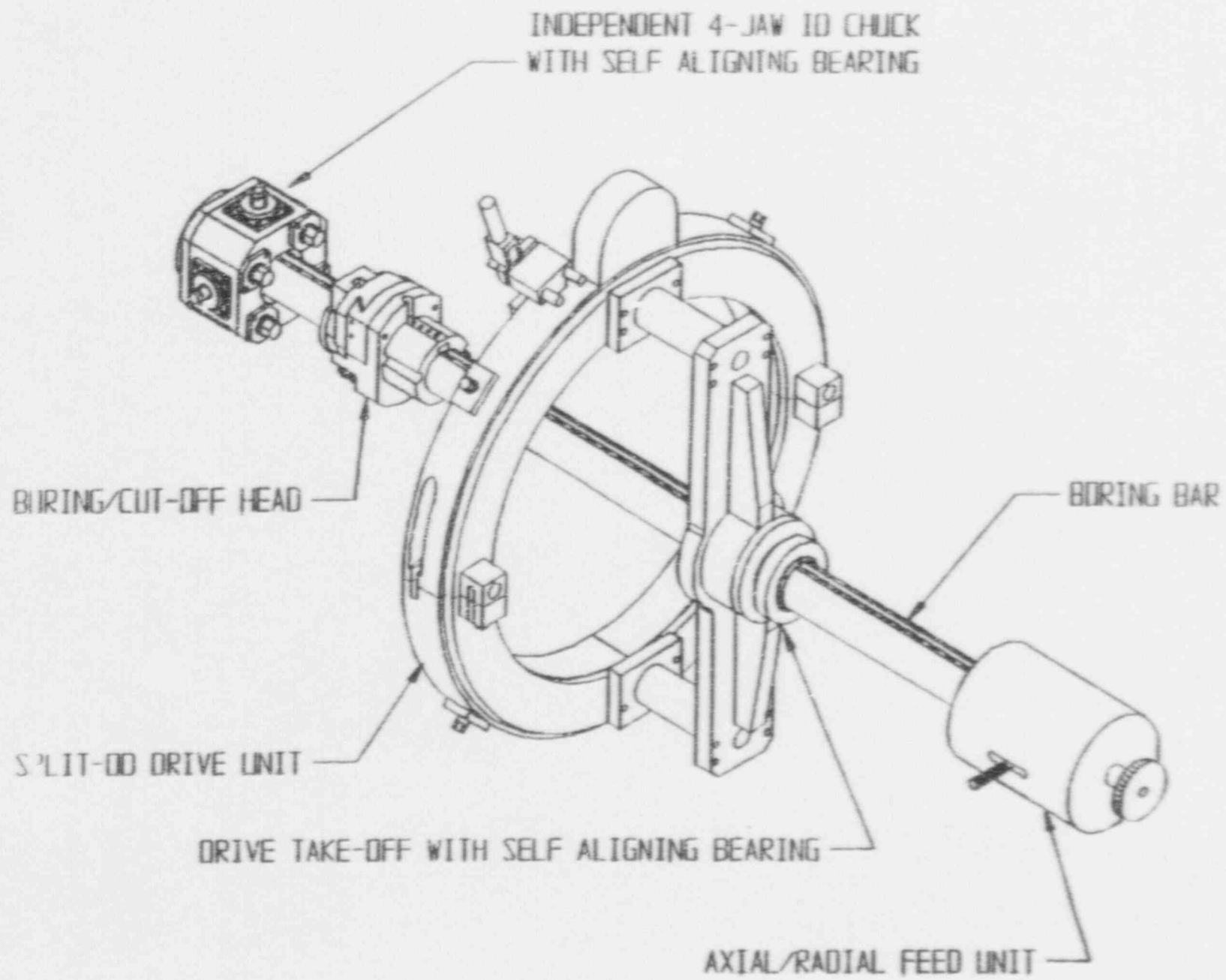
DETAIL A

STEP 1

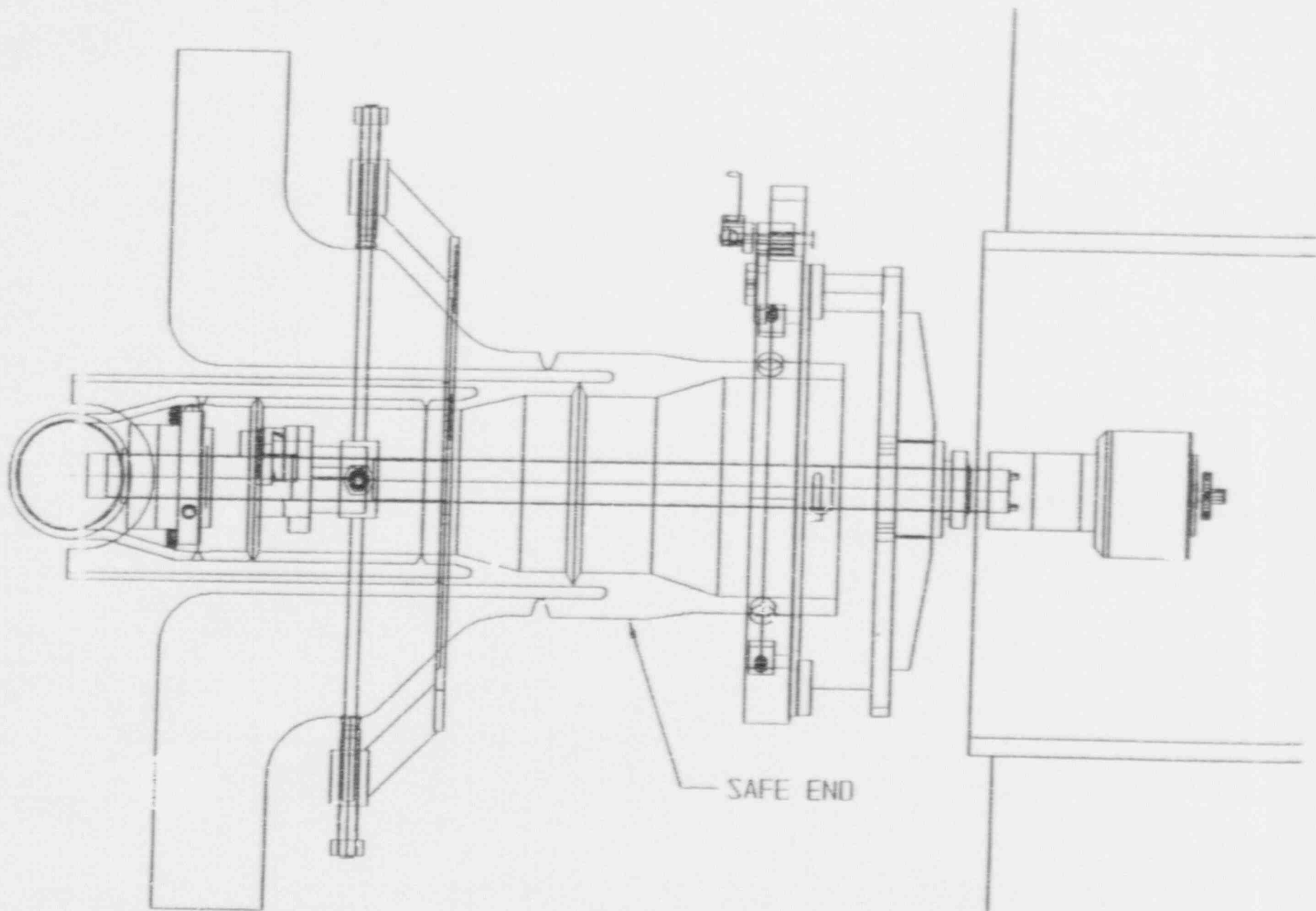
EXISTING SAFE END AND THERMAL SLEEVE

132

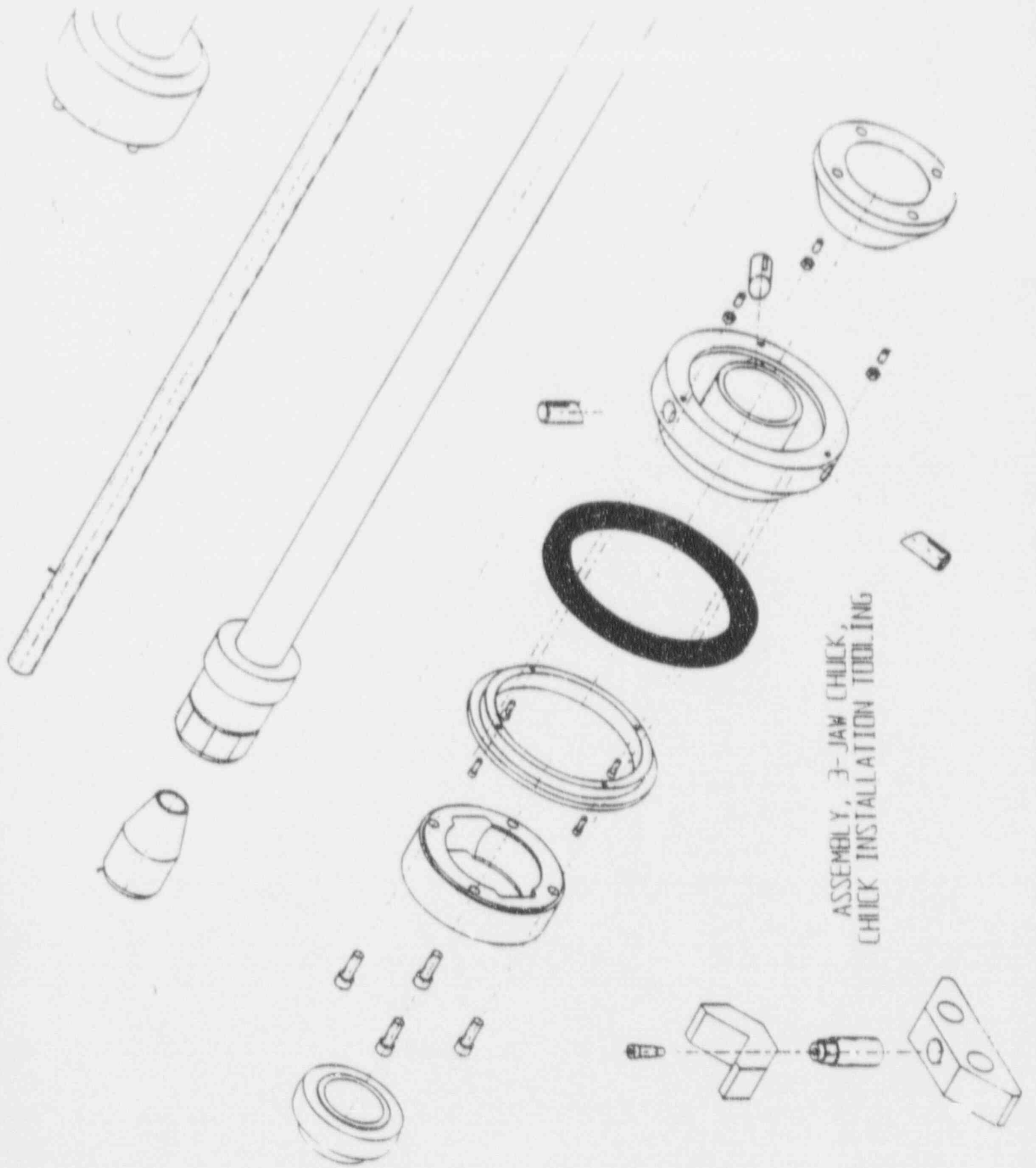




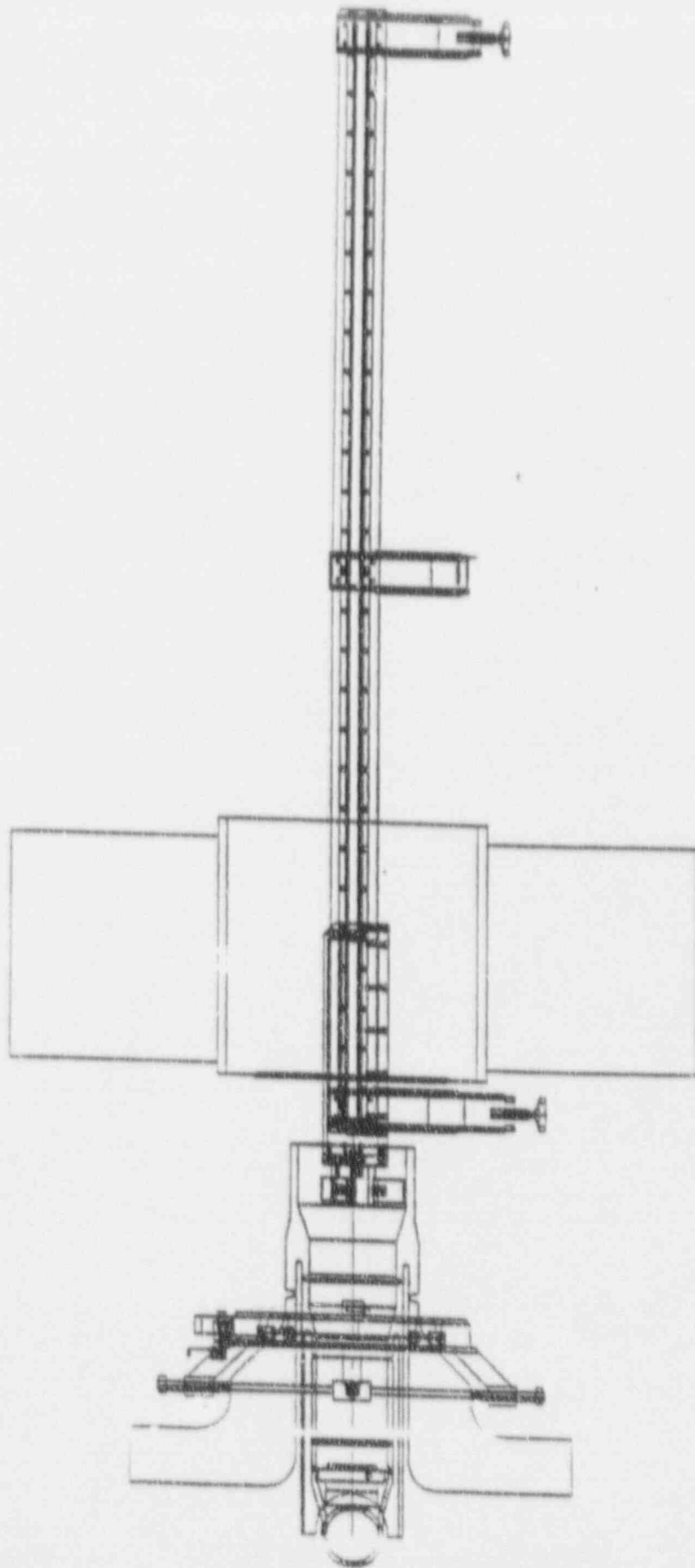
1/2/92



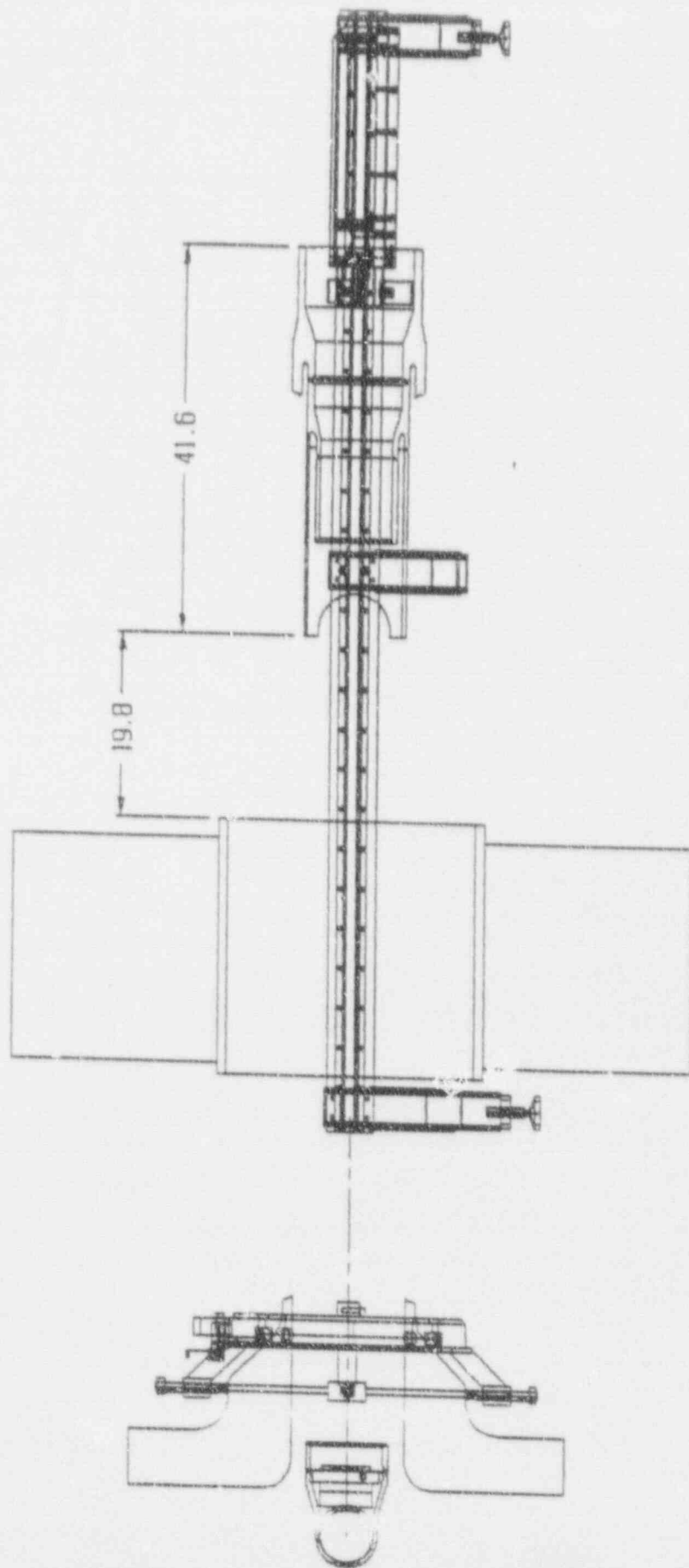
SEVER SPARGER USING ID BORING BAR



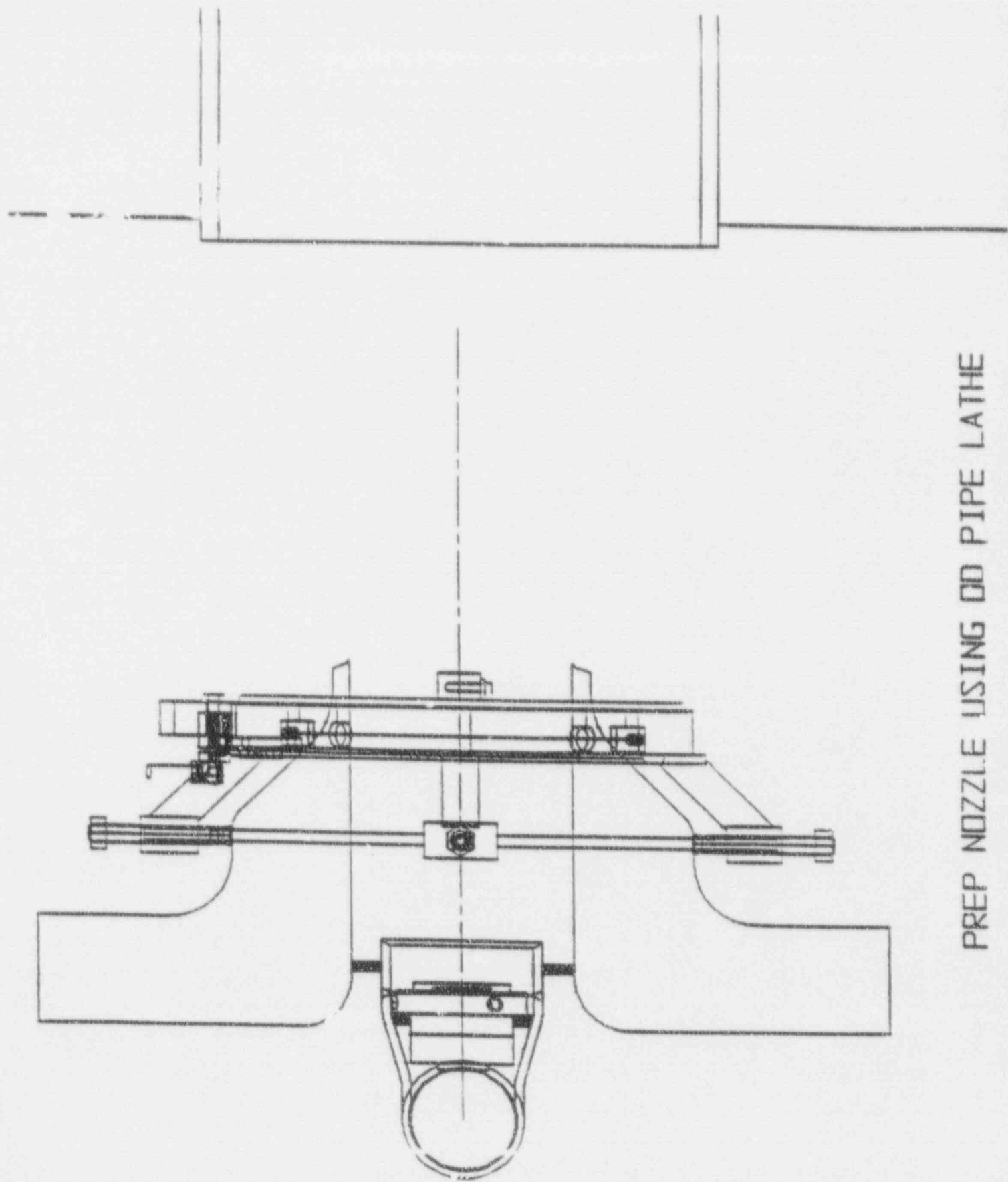
ASSEMBLY, 3-JAW CHUCK,  
CHUCK INSTALLATION TOOLING



SAFE END INSERTION POSITION

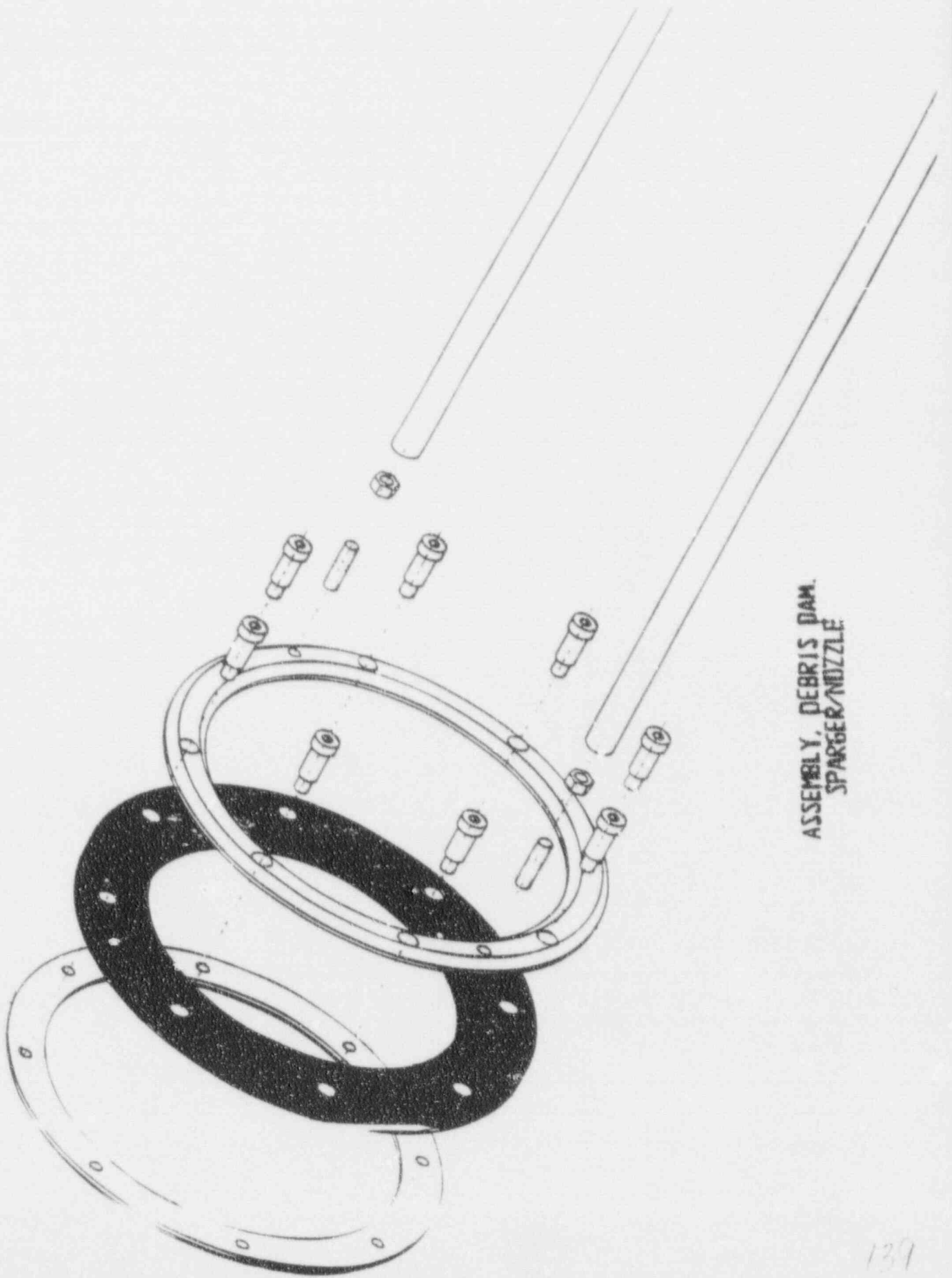


SAFE END RETRACT POSITION



PREP NOZZLE USING OD PIPE LATHE



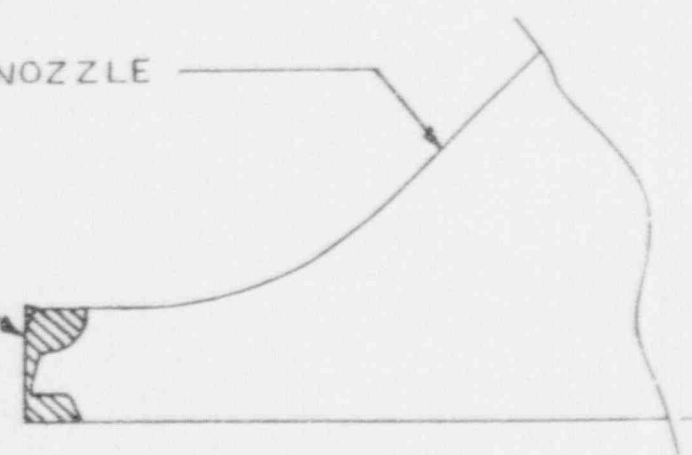


ASSEMBLY, DEBRIS DAM.  
SPARGER/NOZZLE



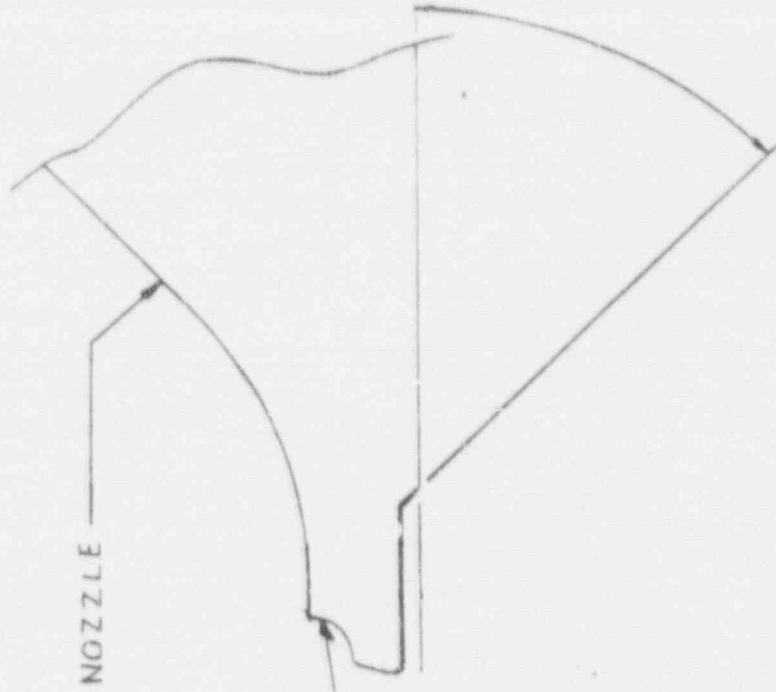
REMAINING ALLOY 182  
BUTTER AFTER  
SAFE END REMOVAL

NOZZLE



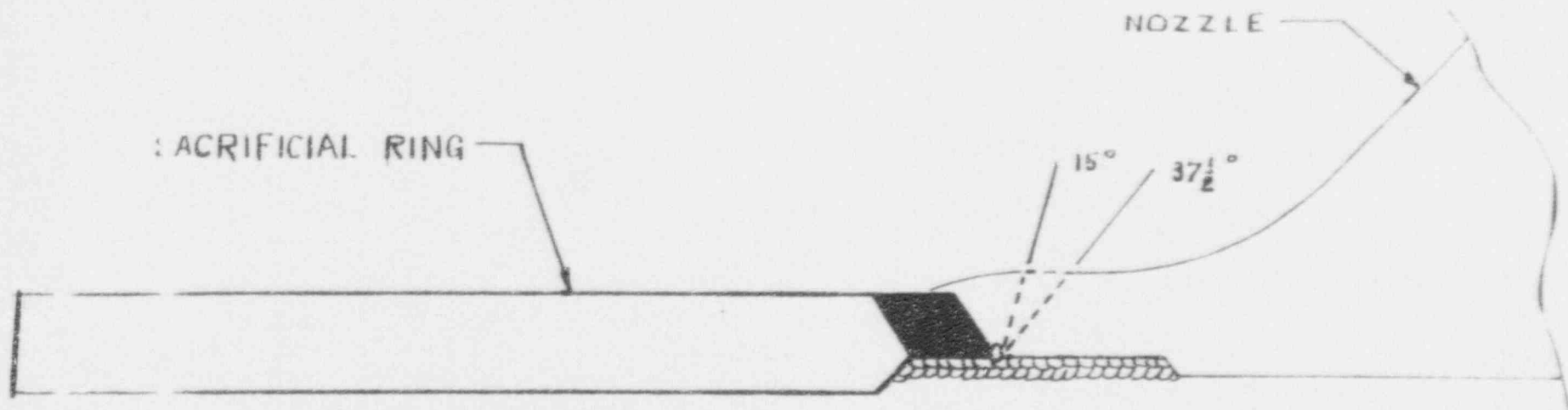
NOZZLE AFTER SAFE END REMOVAL

271



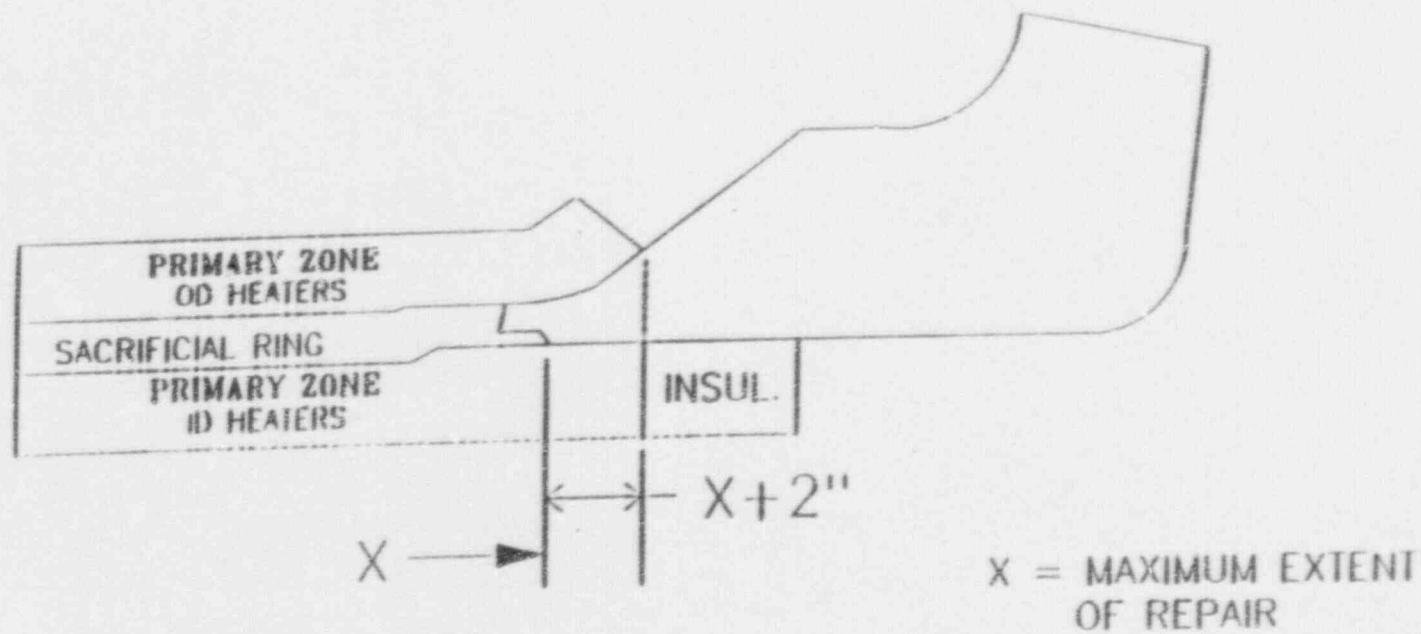
NOZZLE

ETCH  
TO VERIFY REMOVAL  
OF ALLDY 182



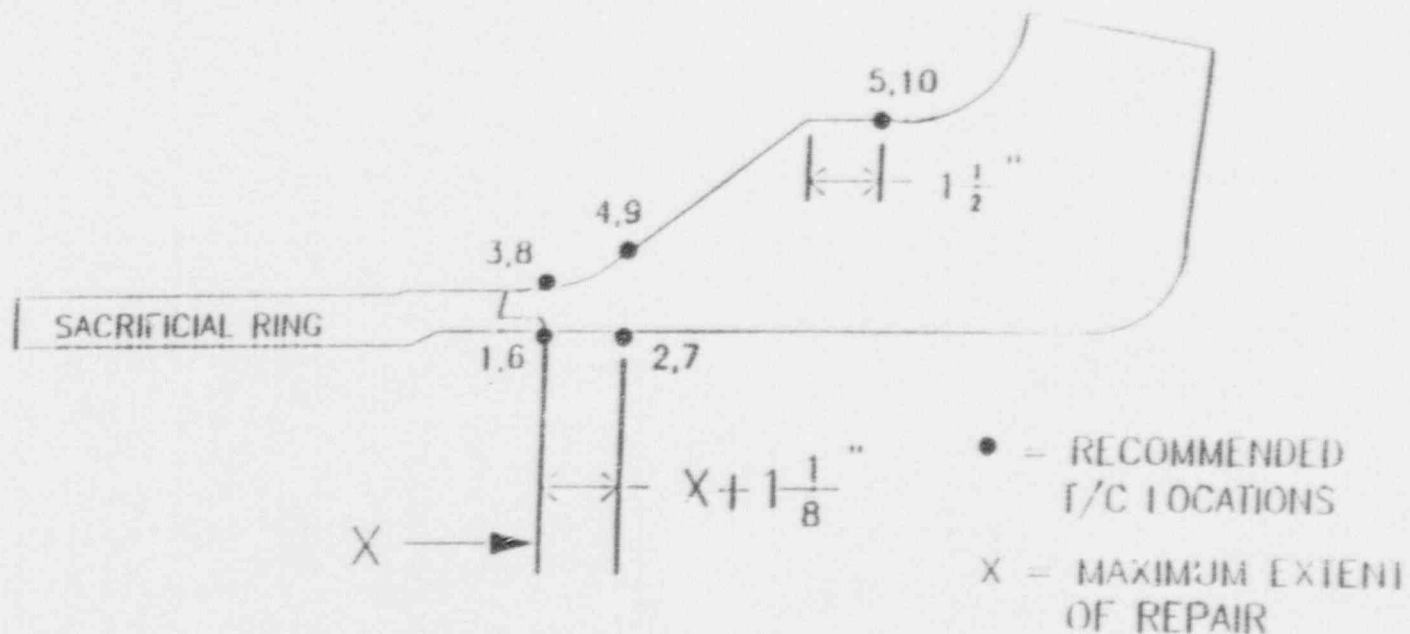
WELD JOINT DETAILS OF SACRIFICIAL RING TO NOZZLE

NOTE: PLACE HEATERS AND INSULATION CIRCUMFERENTIALLY



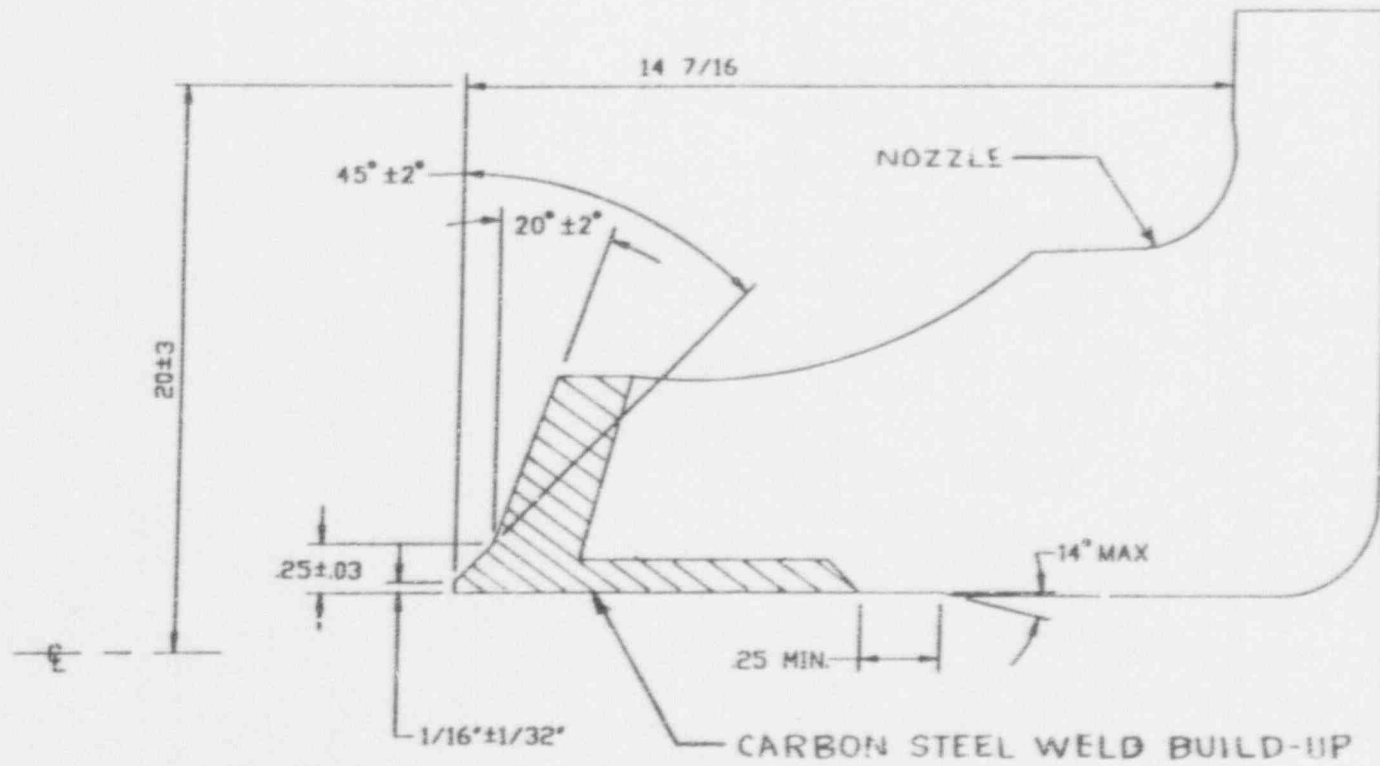
HEATER AND INSULATION LOCATIONS

NOTE: A MINIMUM OF 2 THERMOCOUPLES IS RECOMMENDED FOR EACH LOCATION (ONE AT THE TOP, OTHER AT THE BOTTOM)



RECOMMENDED THERMOCOUPLE LOCATIONS  
 THERMOCOUPLE READINGS ARE SHOWN IN TABLE 1

77-1

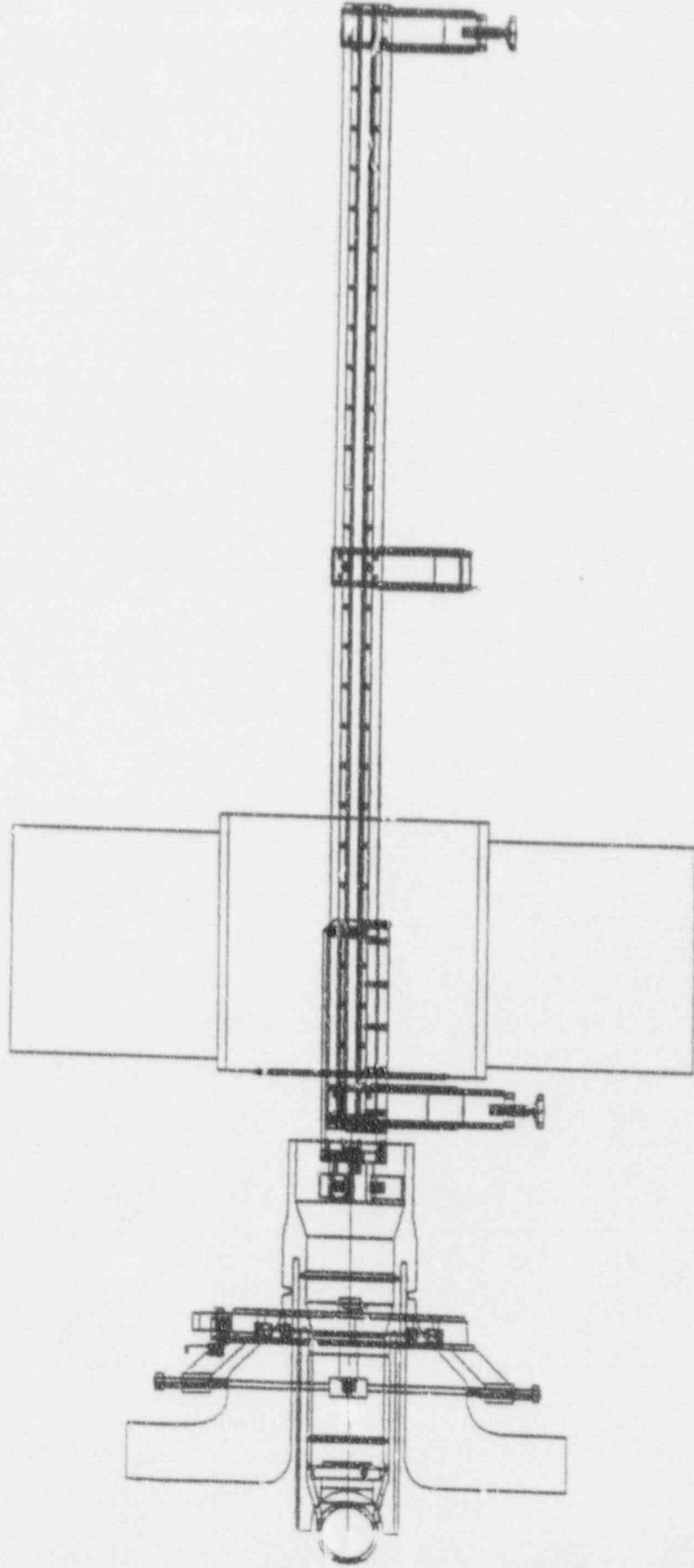


DETAIL D

NOZZLE WELD END PREPARATION

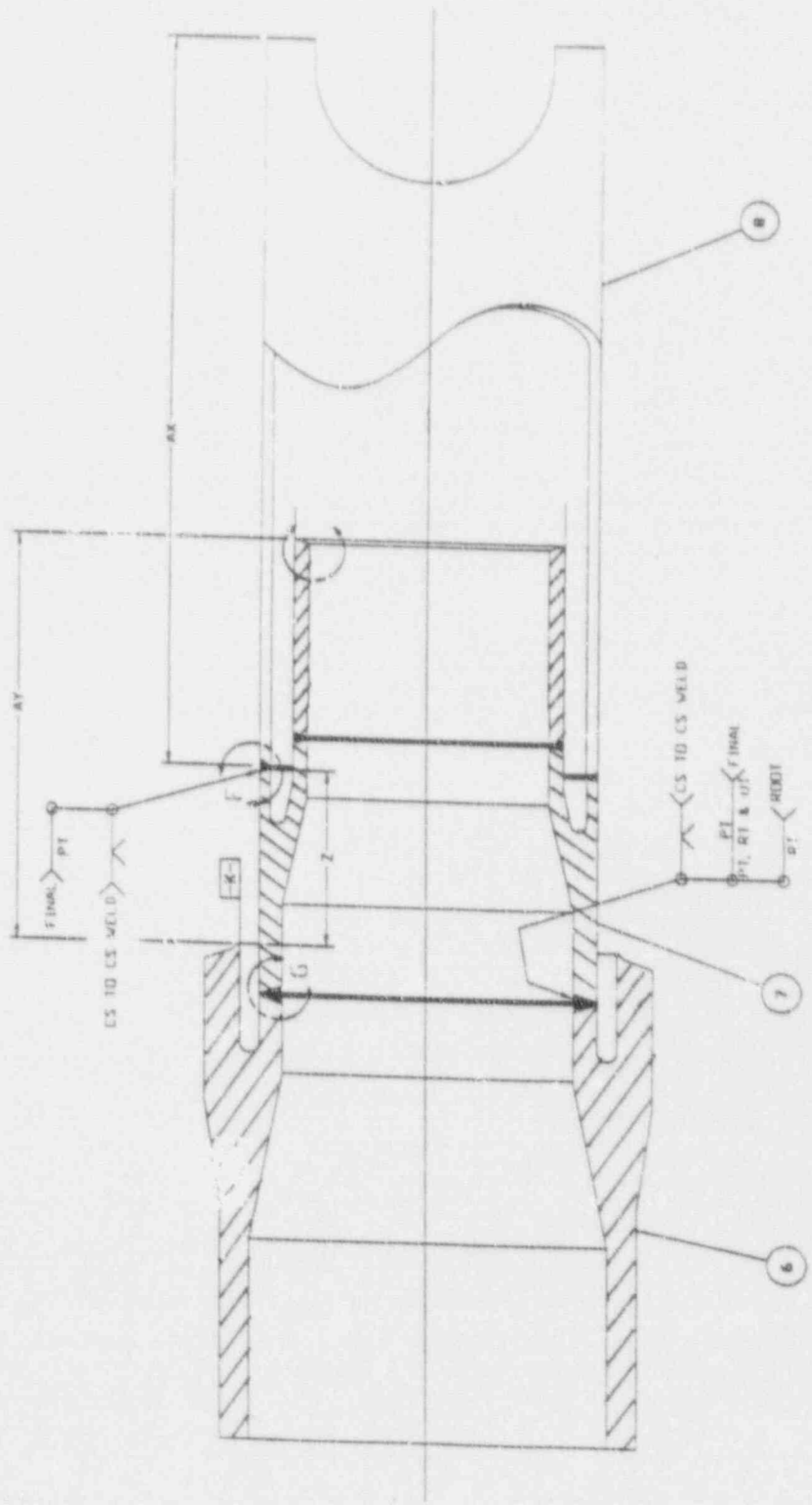
147



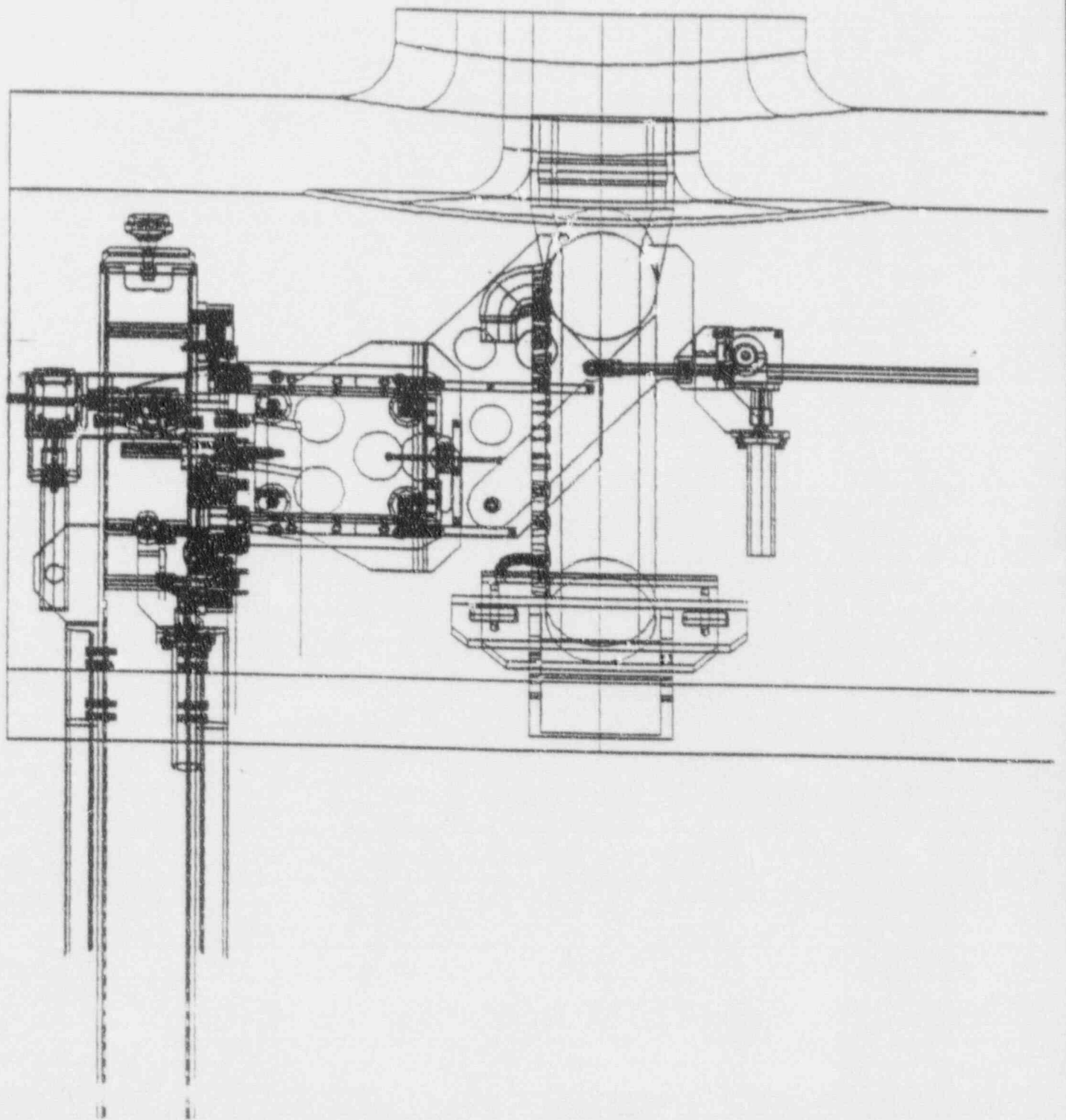


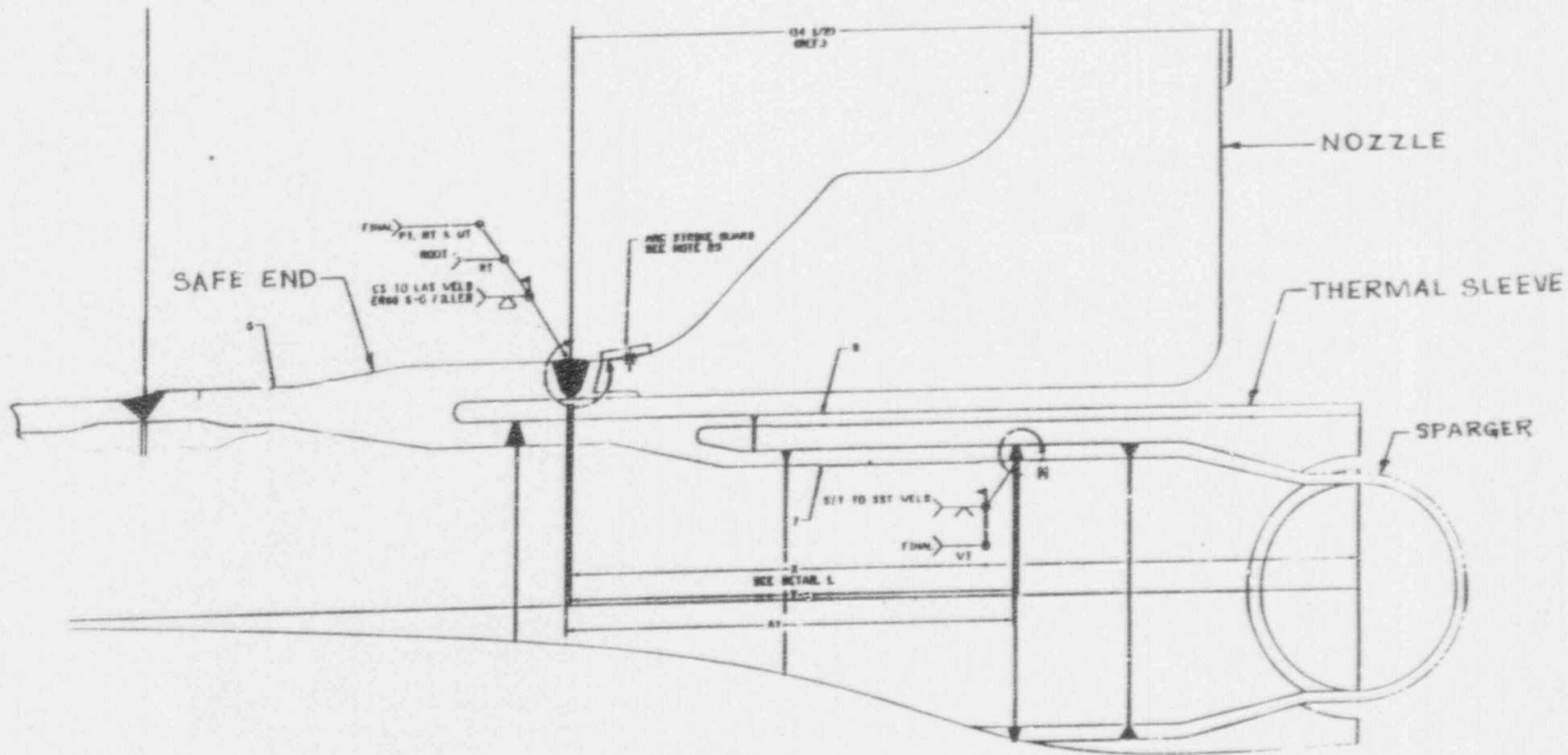
SAFE END INSERTION POSITION





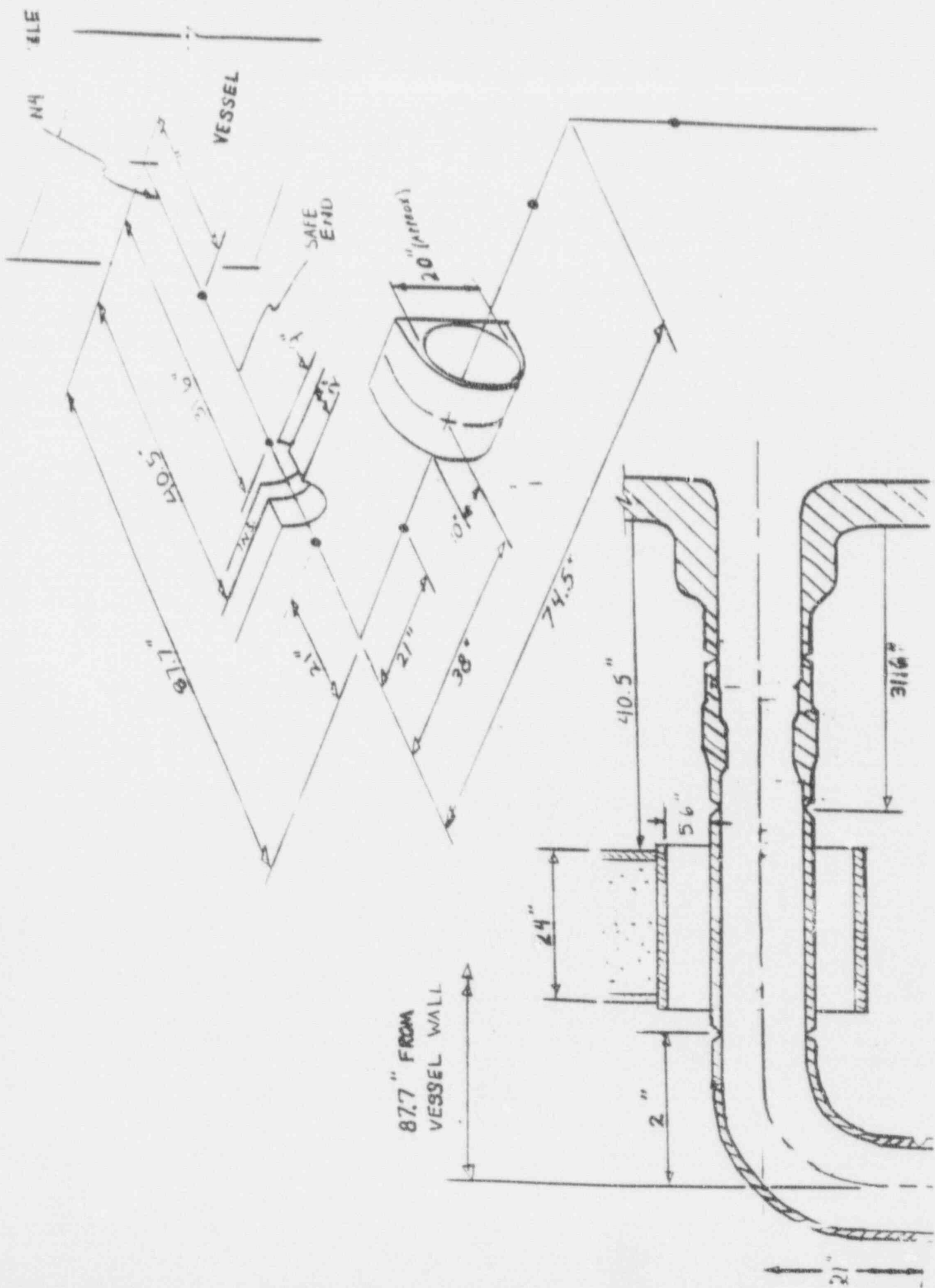
SAFE END ASSEM.





NEW SAFE END CONFIGURATION

671



10/15/91

RF-4 GOALS

GOAL	MEASURE OF SUCCESS
MR scope growth via Emergent Work excluding safety items	≤ 10
MR Scope Growth prior to outage	≤ 10
MWO Scope Growth between freeze at 120 days before outage and outage start	≤ 10%
MWO Scope Growth via Emergent Work in RF-4	≤ 50%
MWO Scope Growth of previously known tasks submitted after the 7.5 month preoutage freeze	≤ 5%
Schedule Performance Negativity	≤ 0 DAYS
Lost Time Accidents	0
Service Water System Closed Loop Installation and Chemical Cleaning	100%
Recirc and Cleanup System Chemical Cleaning	100%
Recordable Injuries (OHSA)	≤ 7
Person-Rem Exposure	≤ 500
Contaminations	≤ 200
Near Miss and Lost Time Accidents	0
Unplanned ESF Actuations	≤ 3
Budget	≤ +5%

File: RF-4 GOALS