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November 2, 1995  
NG-95-2865

Mr. William T. Russell, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Attn.: Document Control Desk  
Mail Station P1-37  
Washington, DC 20555-0001

Subject: Duane Arnold Energy Center  
Docket No: 50-331  
Op. License No: DPR-49  
Clarification of Meeting Summary from July 18, 1995  
Meeting between NRC and BWRVIP  
Reference: NRC Meeting Summary from C. K. Battige (NRC) to J. R. Strosnider  
(NRC), dated July 25, 1995  
File: B-11

Dear Mr. Russell:

On July 18, 1995, a meeting was held between the NRC Staff and representatives of the Boiling Water Reactor Vessel and Internals Project (BWRVIP) to discuss the BWRVIP's generic alternative approach to address 10 CFR 50.55a(g)(6)(ii)(A), "Augmented Examination of Reactor Vessel." The subject meeting summary erroneously stated that "Significant service-induced flaws have been observed" in the Duane Arnold Energy Center (DAEC) reactor vessel.

We wish to clarify that no service-induced flaws have been found in the DAEC reactor vessel. In 1978, significant cracking was found in the recirculation system inlet safe ends. This cracking was due primarily to a mechanical notch which was machined into the safe end during original fabrication. The notch provided a significant stress riser and provided a place for intergranular stress corrosion cracking to initiate and propagate rapidly. The safe ends were replaced at that time with a newer design.

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Mr. William T. Russell

November 2, 1995

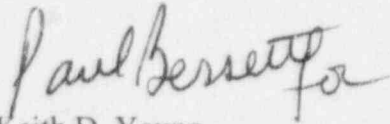
NG-95-2865

Page 2

This clarification has been provided informally to NRC staff members; however, due to the sensitivity of the issue of reactor vessel integrity, we hereby request a correction to the referenced meeting summary.

Should you have any questions regarding this matter, please contact this office.

Sincerely,



Keith D. Young

Manager, Nuclear Licensing

KDY/CJR/so

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J. Franz  
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C. Battige (NRC-NRR)  
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