Commonwealth Edison Company LaSalle Generating Station 2601 North 21st Road Marseilles, IL 61341-9757 Tcl 815-357-6761

ComEd

November 10, 1995

U.S. Nuclear Regulatory Commission **Document Control Desk** Washington, D.C. 20555

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for October, 1995.

D. J. Ray Station Manager LaSalle County Station

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cc: H. J. Miller, Regional Administrator - Region III NRC Senior Resident Inspector - LaSalle IL Department of Nuclear Safety - LaSalle IL Department of Nuclear Safety - Springfield, IL NRR Project Manager - Washington, D.C. GE Representative - LaSalle Regulatory Assurance Supervisor - LaSalle Licensing Operations Director - Downers Grove Nuclear Fuel Services Manager - General Office Off-Site Safety Review Senior Participant - Downers Grove **INPO** Records Center Central File

LASALLE NUCLEAR POWER STATION

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UNIT 1

MONTHLY PERFORMANCE REPORT

October 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373 LICENSE NO. NPF-11 TABLE OF CONTENTS (UNIT 1)

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 - AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS B.
 - LICENSEE EVENT REPORTS C.
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 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
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 - 1. Main Steam Safety Relief Valve Operations
 - 2. Major Changes to Radioactive Waste Treatment System

 - Static O-Ring Failures
 Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2056.

II. . MONTHLY REPORT

A.	SUMMARY	OF	OPERATING	EXPERIENCE	(Unit	1)	
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Day	Time	Event
1	0000	Reactor critical, Generator on-line at 1000 Mwe.
	1200	Reduced power level to 740 Mwe for performance of a rod set.
	1630	Increased power level to 1015 Mwe.
4	1900	Reduced power level to 880 Mwe to place the Turbine Driven Reactor Feed Pump on-line.
5	0400	Increased power level to 1125 Mwe.
15	1600	Reduced power level to 850 Mwe for performance of a rod set.
16	0200	Increased power level to 1125 Mwe.
19	0500	Reduced power level to 1060 Mwe to place the "1D" Condensate/Condensate Booster Pump on-line.
	0545	Increased power level to 1110 Mwe.
	0600	Reduced power level to 1060 Mwe to take the "1D" Condensate/Condensate Booster Pump off-line.
	0730	Increased power level to 1110 Mwe.
20	0430	Reduced power level to 1040 Mwe to swap the Heater Drain Pumps.
	0600	Increased power level to 1110 Mwe.
27	2100	Reduced power level to 830 Mwe for performance of Turbine Generator and Main Steam Isolation Valve testing.
28	0515	Reduced power level to 800 Mwe.
	1700	Increased power level to 1080 Mwe.
29	0000	Reduced power level to 785 Mwe for performance of scram time testing.
	2400	Power level at 1100 Mwe.
31	2400	Reactor critical, Generator on-line at 1050 Mwe.

B. AMENDMENTS TO THE FACILITY OF TECHNICAL SPECIFICATION

On October 10, 1995, Amendment 107 was issued to license NPF-11 (Unit 1). This amendment updated the changes in responsibility and reporting relationships to reflect the present station and corporate organizations.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 1)

LER No.	Occurence Date	Description
95-013	08/25/95	Missed calibration of the "1B" Diesel Generator low lube oil pressure shutdown switch.
95-016	10/24/95	A manual Reactor scram due to increasing Reactor water level which was caused by the Emergency Governor Lockout Valve sticking.

- D. DATA TABULATIONS (Unit 1)
 - 1. Operating Data Report (See Table 1)
 - 2. Average Daily Unit Power Level (See Table 2)
 - 3. Unit Shutdowns and Significant Power Reductions (See Table 3)
- E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)
 - Safety Relief Valve Operations (None)
 - Major Changes to Radioactive Waste Treatment Systems (None)
 - Static O-Ring Failures (None)

UNIQUE REPORTING REQUIREMENTS (UNIT 1) (continued)

4. Changes to the Off-Site Dose Calculation Manual

The following is a summary of the changes to the Offsite Dose Calculation Manual, Revision 1.7 (September 1995) for LaSalle Site Specific Sections: Chapters 10, 11, 12 and Appendix F. A determination has been made that these changes will not reduce the accuracy nor reliability of the dose calculations and setpoint determinations.

-Added text regarding potential waste oil incineration and potential exfiltration of Turbine Building air.

-Implement Uniform Radiological Environmental Monitoring Program with appropriate updates.

-State 25% frequency definition clarification to Chapter 12 for ODCM surveillances.

-Inserted Technical Specification requirement for liquid holdup tanks and maine condenser release rate.

-Inserted summary table of effluent and environmental summary of regulations.

-Revised the definition of Member of the Public and Occupationally Exposed according to the 8-14-95 10CFR20 Revision.

-Reduced distance of Restricted Area Boundary in sectors E and ESE sectors Appendix F.

-Revised Restricted Area Boundary Map to indicate Waste Storage locations.

-Editorial comments to ensure consistency in ODCM and Technical Specification text.

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TABLE 1 D.1 OPERATING DATA REPORT DOCKET NO. 050-373

DOCKET NO. 050-373 UNIT LASALLE OHE DATE November 10, 1995 COMPLETED BY M.J. CIALKOWSKI TELEPHONE (815)-357-6761

OPERATING STATUS

1.	REPORTING PERIOD: October 1995		GROSS HOURS IN REPORTING PERIOD	745
٤.	CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net): Design Electrical Rating (MWe-N	1,036

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

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		REPORTING PERIOD DATA				
5. REACTOR CRITICAL TIME (HOURS)		THIS MONTH 745.0	YEAR-TO-DATE 6,838.1	CUMULATIVE 73,478.0		
6. REACTOR RESERVE SHUTDOWN TIME (HO	JURS)	0.0	0.0	1,641.2		
7. GENERATOR ON-LINE TIME (HOURS)		745.0	6,765.0	71,876.1		
8. UNIT RESERVE SHUTDOWN TIME (HOURS	5)	0.0	0.0	1.0		
9. THERMAL ENERGY GENERATED (MWHt)		2,358,959	21,692,383	214,379,208		
10. ELECTRICAL ENERGY GENERATED (MWH	e-Gross)	798,862	7,278,987	71,669,878		
11. ELECTRICAL ENERGY GENERATED (MWH	e-Net)	773,354	7,043,450	68,812,594		
12. REACTOR SERVICE FACTOR (%)		100.0	93.7	70.8		
13. REACTOR AVAILABILITY FACTOR (%)		100.0	93.7	72.4		
14. UNIT SERVICE FACTOR (%)		100.0	92.7	69.3		
15. UNIT AVAILIBILITY FACTOR (%)		100.0	92.7	69.3		
16. UNIT CAPACITY FACTOR (USING MDC)	(%)	100.2	93.2	64.0		
17. UNIT CAPACITY FACTOR (USING DESIG	GN MWe) (%)	96.3	89.6	61.5		
18. UNIT FORCED OUTAGE FACTOR (%)		0.0	5.2	8.0		

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): 01/27/96

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

DOCKET NO. 050-373 UNIT LASALLE ONE DATE November 10, 1995 COMPLETED BY M.J. CIALKOWSKI TELEPHONE (815)-337-6761

TABLE 2 D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

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	REPORT PERIOD:	October 1995		
DAY	POWER	DAY	POWER	
1	935	17	1,079	
2	970	18	1,075	
3	960	19	1,064	
4	948	20	1,064	
5	1,076	21	1,063	
6	1,084	22	1,061	
7	1,088	23	1,058	
8	1,083	24	1,053	
9	1,078	25	1,048	
10	1,072	26	1,043	
11	1,064	27	1,021	
12	1,060	28	942	
13	1,055	29	944	
14	1,052	30	1,047	
15	1,022	31	1,029	
16	1,085			

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20% (UNIT 1)

YEARLY SEQUENTIAL

TYPE DATE (YYMMDD)

F: FORCED S: SCHEDULED DURATION (HOURS) REASON METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)

(None)

NUMBER

SUMMARY OF OPERATION :

The unit remained on-line at high power throughout the month. Several minor power reductions were required due to rod pattern adjustments, maintenance and surveillance activities.

LASALLE NUCLEAR POWER STATION

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UNIT 2

MONTHLY PERFORMANCE REPORT

October 1995

COMMONWEALTE EDISON COMPANY

NRC DOCKET NO. 050-374 LICENSE NO. NPF-18 TABLE OF CONTENTS (UNIT 2)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
- C. LICENSEE EVENT REPORTS
- D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- E. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Safety Relief Valve Operations
 - 2. Major Changes to Radioactive Waste Treatment System
 - 3. Static O-Ring Failures
 - 4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

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The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2056.

II. . MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 1125 Mwe.
13	1200	Reduced power level to 1090 Mwe due to Reactor Recirculation Pump Flow Control Valve drifting.
	2400	Increased power level to 1125 Mwe.
18	0500	Reduced power level to 1085 Mwe to swap the Heater Drain Pumps.
	1400	Increased power level to 1130 Mwe.
26	1000	Reduced power level to 1120 Mwe due to Reactor Recirculation Pump Flow Control Valve testing.
	1100	Increased power level to 1130 Mwe.
	1300	Reduced power level to 1120 Mwe due to Reactor Recirculation Pump Flow Control Valve testing.
	1400	Increased power level to 1130 Mwe.

31 2400 Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On October 10, 1995, Amendment 93 was issued to license NPF-18 (Unit 2). This amendment updated the changes in responsibility and reporting relationships to reflect the present station and corporate organizations.

- C. SUBMITTED LICENSEE EVENT REPORTS (Unit 2) (None)
- D. DATA TABULATIONS (Unit 2)
 - 1. Operating Data Report (See Table 1)
 - 2. Average Daily Unit Power Level (See Table 2)
 - 3. Unit Shutdowns and Significant Power Reductions (See Table 3)
- E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)
 - Safety Relief Valve Operations (None)
 - Major Changes to Radioactive Waste Treatment Systems (None)
 - Static O-Ring Failures (None)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2) (continued)

4. Changes to the Off-Site Dose Calculation Manual

The following is a summary of the changes to the Offsite Pose Calculation Manual, Revision 1.7 (September 1995) for LaSalle Site Specific Sections: Chapters 10, 11, 12 and Appendix F. A determination has been made that these changes will not reduce the accuracy nor reliability of the dose calculations and setpoint determinations.

-Added text regarding potential waste oil incineration and potential exfiltration of Turbine Building air.

- -Implement Uniform Radiological Environmental Monitoring Program with appropriate updates.
- -State 25% frequency definition clarification to Chapter 12 for ODCM surveillances.
- -Inserted Technical Specification requirement for liquid holdup tanks and maine condenser release rate.
- -Inserted summary table of effluent and environmental summary of regulations.
- -Revised the definition of Member of the Public and Occupationally Exposed according to the 8-14-95 10CFR20 Revision.
- -Reduced distance of Restricted Area Boundary in sectors E and ESE sectors Appendix F.
- -Revised Restricted Area Boundary Map to indicate Waste Storage locations.
- -Editorial comments to ensure consistency in ODCM and Technical Specification text.

TABLE 1 D.1 OPERATING DATA REPORT

DOCKET NO. 050-374 UNIT LASALLE TWO DATE November 10, 1995 COMPLETED BY M.J. CIALKOWSKI TELEPHONE (815)-357-6761

1.	REPORTING PERIOD:	October 1995		GROSS HOURS IN REPORTING PERIOD:	745
2.	CURRENTLY AUTHORIZED POWER	LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net): Design Electrical Rating (MWe-Net):	1,036

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

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OPERATING STATUS

. READING FOR RESIRICITOR (IF ANI).	REP	REPORTING PERIOD DATA			
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE		
5. REACTOR CRITICAL TIME (HOURS)	745.0	4,617.6	69,825.6		
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9		
7. GENERATOR ON-LINE TIME (HOURS)	745.0	4,392.0	68,377.5		
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0		
9. THERMAL ENERGY GENERATED (MWHt)	2,470,055	13,409,096	207, 161, 024		
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	842,831	4,557,955	69,246,524		
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	816,638	4,370,974	66,555,736		
12. REACTOR SERVICE FACTOR (%)	100.0	63.3	72.2		
13. REACTOR AVAILABILITY FACTOR (%)	100.0	63.3	74.0		
14. UNIT SERVICE FACTOR (%)	100.0	60.2	70.7		
15. UNIT AVAILIBILITY FACTOR (%)	100.0	60.2	70.7		
16. UNIT CAPACITY FACTOR (USING MDC) (%)	105.8	57.8	66.4		
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	101.7	55.6	63.8		
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	4.1	10.2		

1	9.	SHUTDOWNS	SCHEDULED	OVER THE	NEXT 6	MONTHS (TYPE,	DATE, AND	DURATION OF	EACH):	N/A
2	0.	IF SHUTDON	WN AT END	OF REPORT	PERIOD	. ESTIMATED DA	TE OF STAR	TUP:		N/A

DOCKET NO. 050-374 UNIT LASALLE TWO DATE November 10, 1995 COMPLETED BY M.J. CIALKOWSKI TELEPHONE (815)-357-6761

TABLE 2 D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

	REPORT PERIOD:	October 1995	
DAY	POWER	DAY	POWER
1	1,093	17	1,098
2	1,092	18	1,093
3	1,087	19	1,096
4	1,086	20	1,099
5	1,089	21	1,100
6	1,094	22	1,099
7	1,095	23	1,101
8	1,095	24	1,103
9	1,093	25	1,103
10	1,090	26	1,102
11	1,089	27	1,103
12	1,089	28	1,102
13	1,090	29	1,147
14	1,097	30	1,103
15	1,098	31	1,102
16	1,100		

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TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20% (UNIT 2)

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METHOD OF CORRECTIVE YEARLY TYPE SHUTTING DOWN ACTIONS/COMMENTS SEQUENTIAL DATE F: FORCED DURATION THE REACTOR OR (LER # if NUMBER (YYMMDD) S: SCHEDULED (HOURS) REASON REDUCING POWER applicable)

(None)

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout the month. Several minor power reductions were required during the month due to maintenance and surveillance activities.