

GPU Nuclear Corporation

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February 27, 1992 C311-92-2030

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Dear Sir:

Subject: Three Mile Island Nuclear Station, Unit 1 (TMI-1)

Operating License No. DPR-50,

Docket No. 50-289

1991 Technical Specifications Section 6 Annual Report

Attached is the 1991 Technical Specifications Section 6 Annual Report for Three Mile Island Nuclear Station, Unit 1 (TMI-1). This report is being submitted in accordance with Section 6.9.1.B and 6.17 of the TMI-1 Technical Specifications (T.S.). The attachments to this letter contain the following information:

- Attachment I Tabulation of Personnel Exposure Data for the Calendar Year 1991 (per T.S. Section 6.9.1.B.1).
- Attachment II Aircraft Movement Data from the Harrisburg International Airport for the Calendar Year 1991 (per T.S. Section 6.9.1.B.2).
- Attachment III Leak Reduction Program Test Information for the Calendar Year 1991 (per T.S. 6.9.1.B.3).
- Attachment IV Pressurizer Power Operated Relief Valve and Pressurizer Safety Valve Challenges for the Calendar Year 1991 (per T.S. Section 6.9.1.B.4).
- Attachment V Results of Specific Activity Analysis Primary Coolant System (per T.S. Section 6.9.1.B.5).

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Attachment VI - Major Changes to Radioactive Waste Treatment Systems (per T.S. Section 6.17).

Sincerely,

J&Branchton

T. G. Broughton Vice President and Director, TMI-1

DVH/mkk

Attachments

cc: TMI-1 Senior Project Manager Region I Administrator TMI Senior Resident Inspector

ATTACHMENT I

NUMBER OF PERSONNEL AND MAK REMS BY WORK AND JOB FUNCTION REPORT --- FERIOD DATE 02/06/92 SELF READING POCKET DOSIMETER DATA

OPU NUCLEAR THI UNIT-1 01/01/91 - 12/31/91 PAGE 1

### PREACTOR OPERATIONS/SURV. **MAINTERNANCE PERSONNEL	JOB CATEGORY JOB FUNCTION	STATION NUMBER	PERSONNEL REMS	UTILITY	PERSONNEL. REMS	CONTRACTOR	PERSONNEL REMS
OPERATING PERSONNEL	REACTOR OPERATIONS/SURV						
OPERATING PERSONNEL	ALTERNATIVE DESCRIPT	146	E 778		0.44	268	3.000
HEALTH PHYSICE PERSONNEL							
SUPERVISORY PERSONNEL 104 4.50							
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ATTACHMENT I

BUMBER OF PERSONNEL AND MAR REMS BY WORK AND JOB FUNCTION REPORT----PERIOD

SELF READING POCKET DOSIMETER DATA SELF READING POCKET DOSIMETER DATA

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DATE 02/06/92

OB CATEGORY JOB FUNCTION	STATION NUMBER	PERSONNEL REMS	UTILITY PER	REMA	CONTRACTOR FUMBER	PERSONNEL REMS
* TOTAL BY JOB FUNCTION +						
MAINTENANCE PERSONNEL	166	29.335	11	.216	635	96.645
OPERATING PERSONNEL	121	38.242	4	.138	13	1.447
HEALTH PHYSICS PERSOFNEL	8.2	19.945		.063	5.2	10.838
SUPERVISORY PERSONNEL	132	14.971	12	1.64	3.6	3.344
ENGINEERING PERSONNEL	102	5.711	2.6	379	8.5	7.454
ADMINSTRATIVE PERSONNEL	145	2,220	8.7	.141	30	1.415
** GRAND TOTAL **	748	120.424	155	1.119	853	121.141

ATTACHMENT II

AIRCRAFT MOVEMENTS AT THE HARRISBURG INTERNATIONAL AIRPORT

JANUARY 1 THROUGH DECEMBER 31, 1991

- 1. Total Aircraft Movements = 99,220
- Total number of movements of aircraft larger than 200,000 pounds is estimated to be less than 200.

NOTE: For Item 2, the data is based on estimates provided by the Capital City Airport Tower (Harrisburg Approach)*. This facility is responsible for tracking Harrisburg International Airport flights. The tower is not required to (and does not) record flights by weight or plane category.

The total number of movements (200) is less than .20% of the total movements. This compares to the TMI-1 FSAR assumption of 3%.

*Effective February 23, 1992, Harrisburg Approach is located at the Harrisburg Tower.

ATTACHMENT III

Annual Report Regarding the Periodic Leak Reduction Program Tests (T.S. 6.9.1.B.3).

Table I summarizes the results of the Leak Reduction Program tests and inspections, by procedure number, that were performed between January 1 and December 31, 1991. Component identification of those components found to be leaking and the type of repair (if required) are included in Table I. Leaking components were repaired and retested such that the leakage was reduced from the as found condition to an acceptable level.

The two hour site boundary and 30 day low population zone offsite dose considerations were not limiting for acceptable leakage criteria in 1991. The "AS-FOUND" or "AS LEFT" leakage conditions recorded did not significantly increase the magnitude of either onsite or offsite releases during 1991.

TABLE 1

1991 LEAK REDUCTION PROGRAM TEST RESULTS FOR TMI-1

SURVEILLANCE PROCEDURE	DATE OF PERFORMANCE	LEAKING COMPONENT I.D.	AS FOUND		MAINIENANCE
1303-11.16 "Decay Heat Removal System Leakage"	10/17/91		0	0	None
1303-11.18 "R.B. Local Leak Rate Testing"	See Note 2	See Note 2	17737 SOOM	17377 SOOM	See Note 2
1303-11.27 "Makeup & Purification System Leakage Check"	09/24/91	MU-PlA Seal	6 ml HR	6 <u>ml</u> HR	See Note 3
		MU-PIB Seal	240 <u>ml</u> HR	240 <u>ml</u> HR	See Note 3
		MU-PIC Seal	24 <u>ml</u> HR	24 ml HR	See Note 3
1303-11.28 "Liquid Waste System Leak Check"	05/22/91	WXV1012	60 <u>ml</u> HR	0	Tightened Valve Packing
1303-11.29 "Waste Gas Disposal System Leak Check"	12/11/91		0	0	None
1303-11.30 "Reactor Coolant Sampling Leakage Check"	06/13/91		0	0	Mone
1303-11.31 "Hydrogen Recombiner System Leak Check	11/10/91		0	0	None
1303-11.50 "RB Spray System Leakage Check"	06/18/91		0	0	None

ATTACHMENT III

TABLE 1 (Continued)

The LLRT MAX PATH "AS LEFT" Gaseous Leakage was 17377 SCCM. "As-Found" MAX 113 was 17737 SCCM - See Note 1.

the total "AS-LEFT" Liquid Leakage was 270 ml/hr. As-Found was 330 ml/hr.

The two hour site boundary and 30 day low population zone offsite dose considerations were not limiting conditions for acceptable leakage criteria in 1991. All "AS-FOUND" or "AS-LEFT" leakage recorded did not significantly increase the magnitude of either onsite or offsite releases during 1991.

Per TMI-1 Tech. Spec. 4.4.1, total leakage less than 104,846 SCCM is considered acceptable for the as-left leakage condition.

- NOTE 1: Per the "MIN PATH" method of calculating leakage, the As-Found LLRT Leakage was 8767 SCCM and the As-Left was 8839 SCCM. The "MAX PATH" method assumes for two valves in series hat the one with lower leakage has failed to close post accident. "MIN PATH" assumes the opposite.
- NOTE 2: The valve leak testing occurred on various dates during 1991 9R Refueling Outage. Repairs were performed on the following components due to higher than desirable leakage:
 - 1. CM-V4: Replaced leaking solenoid.
 - West Fuel Transfer Tube Penetration Pressurization System Joint Leakage Repaired.
 - MU-V2B Yoke Rushing Replaced During MOVATS.
 - WDL-V304 Lapped Seats.
- NOTE 3: Considered relatively minor leakage and below the level that repairs would be initiated to repair the leakage.

ATTACHMENT IV

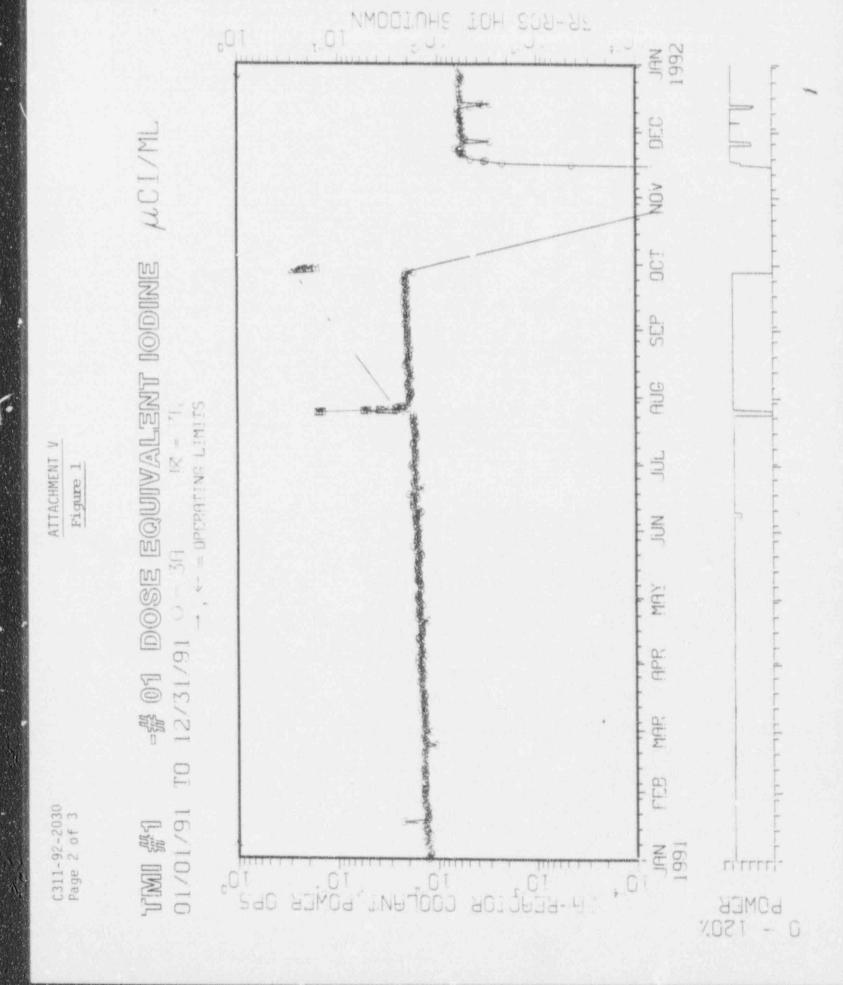
Pressurizer Power Operated Relief Valve and Pressurizer Safety Valve Challenges for Calendar Year 1991.

There were \underline{no} pressurizer power operated relief valve or pressurizer safety valve challenges which occurred during 1991 in response to any plant transients.

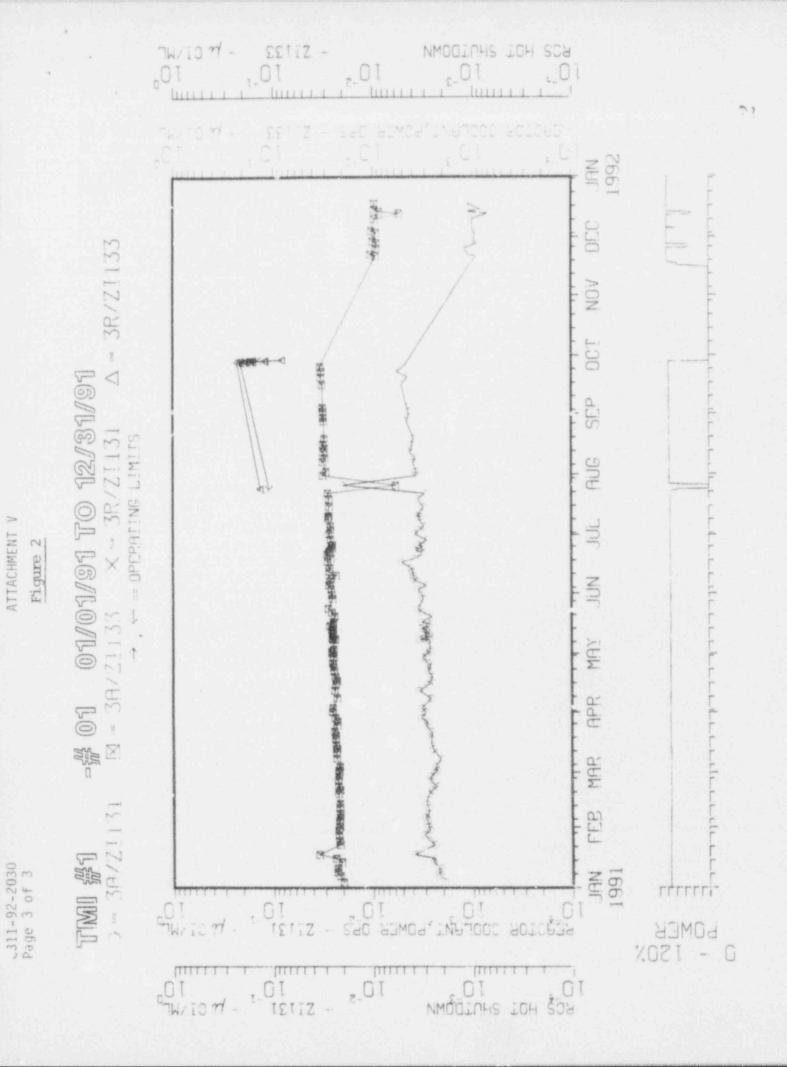
ATTACHMENT V

RESULTS OF SPECIFIC ACTIVITY ANALYSIS PRIMARY COOLANT SYSTEM

Technical Specification 6.9.1.B.5 requires certain information regarding the results of specific activity an lyses in which the primary coolant exceeded limits of Technical Specification 3.1.4.1 be reported. These limits were not exceeded during 1991. Figure 1 contains a graph of the dose equivalent 1-131 concentration and Figure 2 contains a graph of the I-131 and I-133 concentrations. The figures show that all values remained below 1 $\mu\text{Ci/ml}$ during 1991.



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ATTACHMENT VI

Technical Specification Section 6.17 requires reporting of "Major Changes to Radioactive Waste Treatment Systems." Major changes are interpreted to mean changes that would alter how the system functions or the predicted releases that were previously analyzed.

Based on the above, no major changes were made during 1991.