WESTINGHOUSE CLASS 3

Although information contained in this report is nonproprietary, no distribution shall be made outside Westinghouse or its licensees without the customer's approval.

EVALUATION OF THE ACCEPTABILITY OF THE REACTOR
VESSEL HEAD LIFT RIG, REACTOR VESSEL INTERNALS
LIFT RIG, LOAD CELL, AND LOAD CELL LINKAGE
TO THE REQUIREMENTS OF NUREG 0612
for
SOUTH CAROLINA ELECTRIC AND GAS COMPANY
VIRGIL C. SUMMER PLANT

APRIL, 1983

H. H. SANDNER, P. E.

Approved: J. R. Marshall, P.E. Manager Refueling Equipment Engineering

WESTINGHOUSE ELECTRIC CORPORATION
Nuclear Energy Systems
P.O. Box 355
Pittsburgh, PA 15230

ABSTRACT

An evaluation of the Virgil C. Summer reactor vessel head and internal lift rigs, load cell and load cell linkage was performed to determine the acceptability of these devices to meet the requirements of NUREG 0612. The evaluation consists of: (1) a comparison report of the ANSI N14.6 requirements and the requirements used in the design and manufacture of these devices; (2) a stress report in accordance with the design criteria of ANSI N14.6; and (3) a list of recommendations to enable these devices to demonstrate compliance with the intent of NUREG 0612 and ANSI N14.6.

TABLE OF CONTENTS

Section	Title	Page
-	ABSTRACT	111
1.	INTROD UCT ION	1-1
	1.1 Background	1-1
2.	COMPONENT DESCRIPTION	2-1
	2.1 Reactor Vessel Head Lift Rig	2-1
	2.2 Reactor Vessel Internals Lift Rig	2-1
	2.3 Load Cell and Load Cell Linkage	2-2
3.	SCOPE OF EVALUATION	3-1
	3.1 Study of ANSI N14.6-1978	3-1
	3.2 Stress Report	3-1
	3.3 Recommendations	3-2
4.	DISCUSSION OF EVALUATIONS	4-1
	4.1 Study of ANSI N14.6-1978	4-1
	4.2 Stress Report	4-1
	4.3 Recommendations	4-2
5.	CONCLUSIONS	5-1
6	DECOMMEND ATTONS	6-1

LIST OF ILLUSTRATIONS

Ffjore	Title	Page
2-1	Reactor Vessel Head Lift Rig	2-3
2-2	Reactor Vessel Internals Lift Rig	2-4

ATTACHMENTS

- A. Comparison of ANSI N14.6-1978 Requirements for Special Lifting
 Devices and the Requirements for the Reactor Vessel Head Lift Rig,
 Reactor Vessel Internals Lift Rig, Load Cell and Load Cell Linkage
 for South Carolina Electric and Gas Company, Virgil C. Summer Plant.
- B. Stress Report Reactor Vessel Head Lift Rig, Reactor Vessel Internals Lift Rig, Load Cell and Load Cell Linkage for South Carolina Electric and Gas Company, Virgil C. Summer Plant.

REFERENCES

- 1. George, H., Control of Heavy Loads at Nuclear Power Plants

 Resolution of Generic Technical Activity A-36, NUREG 0612,

 July, 1980.
- ANSI N14.6-1978 Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Material
- Westinghouse Drawing 1098E56 3-Loop Lifting Rig Head, General Assembly
- Westinghouse Drawing 1143E65 South Carolina Electric and Gas Company, Virgil C. Summer Internals Lifting Rig General Assembly
- 5. Manual of Steel Construction, Seventh Edition, American Institute of Steel Construction.

SECTION 1 INTRODUCTION

The Nuclear Regulatory Commission (NRC) issued NUREG 0612 "Control of Heavy Loads at Nuclear Power Plants" [1] in 1980 to address the control of heavy loads to prevent and mitigate the consequences of postulated accidental load drops. NUREG 0612 imposes various training, design, inspection and procedural requirements for assuring safe and reliable operation for the handling of heavy loads. In the containment building, NUREG 0612 Section 5.1.1(4) requires special lifting devices to meet the requirements of ANSI N14.6-1978-"American National Standard for Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Materials". [2] In general, ANSI N14.6 contains detailed requirements for the design, fabrication, testing, maintenance, and quality assurance of special lifting devices. The Virgil C. Summer lifting devices which can be categorized as special lifting devices and which are contained in the scope of this report are:

- 1. Reactor vessel head lift rig
- 2. Reactor vessel internals lift rig
- 3. Load cell and load cell linkage

This report contains the evaluation performed on these lifting devices to determine the acceptability of these devices to meet the above requirements.

1.1 BACKGROUND

The reactor vessel head lift rig, the reactor vessel internals lift rig, load cell and load cell linkage were designed and built for the Virgil C. Summer Plant circa 1974-76. These devices were designed to the requirement that the resulting stress in the load carrying members when

subjected to the total combined lifting weight should not exceed the allowable stresses specified in the ${\rm AISC}^{[5]}$ code. Also, a 125 percent load test was required on both devices followed by appropriate non-destructive testing. These items were not classified as nuclear safety components and requirements for formal documentation of design requirements and stress reports were not applicable. Thus, stress reports and design specifications were not formally documented. Westinghouse defined the design, fabrication and quality assurance requirements on detailed manufacturing drawings and purchase order documents. Westinghouse also issued field assembly and operating instructions, where applicable.

SECTION 2 COMPONENT DESCRIPTION

2.1 REACTOR VESSEL HEAD LIFT RIG

The reactor vessel head lift rig^[3] (Figure 2-1) is a three-legged carbon steel structure, approximately 43 feet high and 12 feet in diameter, weighing approximately 21,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and Control Rod Drive Mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operation, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2 REACTOR VESSEL INTERNALS LIFT RIG

The internals lifting rig^[4] (Figure 2-2) is a three-legged carbon and stainless steel structure, approximately 30 feet high and 13 feet in diameter weighing approximately 20,000 pounds. It is used to handle the upper and lower reactor vessel internals packages. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The reactor vessel internals lift rig attaches to the internals package by means of three rotolock studs which engage three rotolock inserts located in the internals flange. These rotolock studs are manually operated from the internals lift rig platform using a handling tool which is an integral part of the rig. The studs are normally spring retracted upward and are depressed to engage the inserts. Rotating the mechanism locks it in both positions.

2.3 LOAD CELL AND LOAD CELL LINKAGE

The load cell is used to monitor the load during lifting and lowering the reactor vessel head or internals to ensure no excessive loadings are occurring. The unit is a load sensing clevis type, rated at 500,000 pounds.

This load cell is a part of the load cell linkage which is an assembly of pins, plates, and bolts that connect the polar crane main hook to the lifting blocks of both the reactor vessel head and the internal lift rigs.

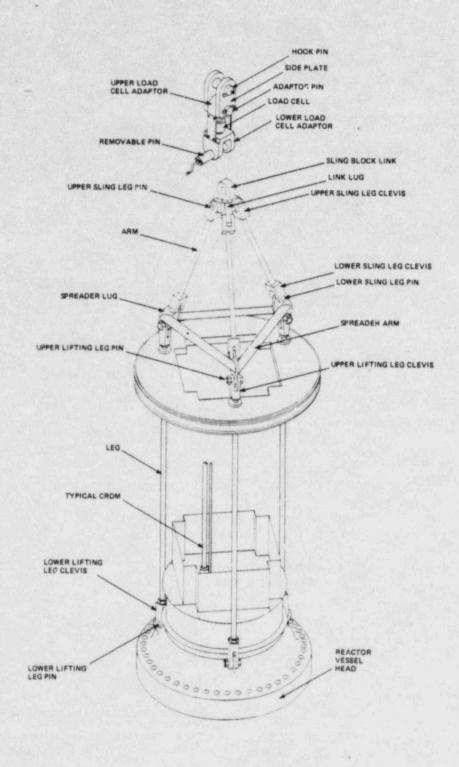


Figure 2-1. Reactor Vessel Head Lift Rig

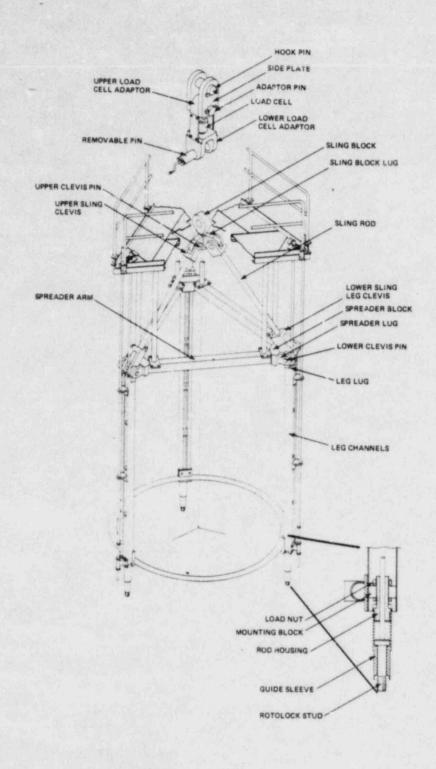


Figure 2-2. Reactor Vessel Internals Lift Rig

SECTION 3 SCOPE OF EVALUATION

The evaluation of these lifting devices consists mainly of three parts:

- 1. A detailed review of the ANSI N14.6 requirements
- 2. Preparation of a stress report
- Recommendations to demonstrate compliance with NUREG 0612, Section 5.1.1(4).

Discussion of these items follows.

3.1 REVIEW OF ANSI N14.6-1978

A detailed comparison was made of the information contained in ANSI N14.6 with the information that was used to design, manufacture, inspect and test these special lifting devices. The detailed comparison is provided in three parts:

- 1. Overall item by item comparison of requirements
- Preparation of a critical item list per ANSI N14.6 Section 3.1.2, and
- 3. Preparation of a list of nonconforming items.

This detailed analysis is contained in Attachment A to this report.

3.2 PREPARATION OF A STRESS REPORT

Section 3.1.3 of ANSI N14.6 and NUREG 0612 Section 5.1.1(4) require a stress report to be prepared. Special loads and allowable stress criteria are specified for this analysis. The stress report is Attachment B to this report.

3.3 RECOMMENDED ACTIONS

An obvious result from the previous evaluations is a list of items that can be performed to demonstrate to the NRC that these special lifting devices are in compliance with the guidelines of ANSI N14.6 and NUREG 0612 Section 5.1.1(4). These recommendations are identified in Section 6.

SECTION 4 DISCUSSION OF EVALUATIONS

4.1 STUDY OF ANSI N14.6-1978

A review of ANSI N14.6 identifies certain analyses to be performed and certain identifications that are required to be made to demonstrate compliance with this document. These are a preparation of a stress report in accordance with Section 3.2 and a preparation of a critical items list in accordance with Section 3.1.2. The stress report is Attachment B to this report. The critical items list has been prepared per Section 3.1.2 and is contained in Appendix A to Attachment A. This list identifies the critical load path parts and welds, the materials of these items, and the applied non-destructive volumetric and surface inspections that were performed.

A detailed item by item comparison of all the requirements of ANSI N14.6 and those used for the design, manufacture and inspection of these lifting devices is contained as Table 2-1 of Attachment A. The comparison shows that these devices meet the intent of the ANSI document for design, fabrication and quality control. However, they do not meet the requirements of ANSI N14.6 for periodic maintenance, proof and functional testing. Thus, a tabulation of those ANSI N14.6 requirements that are incompatible with these lifting devices was prepared and is Appendix B to Attachment A. Included in Appendix B to Attachment A are recommended actions that may be used to demonstrate acceptability to the NRC.

4.2 STRESS REPORT

As part of the invoking of the ANSI N14.6 document, the NRC requested utilities to demonstrate their compliance with the stress criteria with some qualifying conditions. Attachment B is the stress report for these devices performed in accordance with the criteria of ANSI N14.6. A discussion is included which responds to the NRC qualifying conditions of NUREG 0612. All of the tensile and shear stresses, meet the design

criteria of Section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with the accompanying allowable stress limits of yield and ultimate strength, respectively. In addition, all of the tensile and shear stresses meet the requirements of the AISC^[5] code.

4.3 RECOMMENDATIONS

The recommendations identified in Section 6 require a review of plant maintenance and operating instructions to ensure that they contain information relative to the identification, maintenance and periodic testing required by ANSI N14.6. The extent of the periodic testing is also addressed and the recommendations identify procedures which are intended to fully meet the intent of NUREG 0612 and ANSI N14.6 with the least amount of perturbation to the refueling sequence. These recommendations do not involve any equipment changes.

SECTION 5 CONCLUSIONS

The following conclusions are apparent as a result of this evaluation:

- The ANSI N14.6 requirements for design, fabrication and quality assurance are generally in agreement with those used for these special lift devices.
- The ANSI N14.6 criteria for stress limits associated with certain stress design factors for tensile and shear stresses are adequately satisfied.
- 3. These devices are not in strict compliance with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance. Recommendations are included to identify actions that should enable these devices to be considered in compliance with the intent of ANSI N14.6.
- 4. The application of the ANSI N14.6 criteria for stress design factor of 3 and 5 are only for shear and tensile loading conditions. Other loading conditions are to be analyzed to other appropriate criteria.

SECTION 6 RECOMMENDATIONS

The following recommendations address the areas of ANSI N14.6 which are incompatible with the present lifting devices and which are considered most important in demonstrating the continued reliability of these devices. They consist of suggestions and proposed responses to identify compliance to the NRC and future considerations.

- 6.1 Recommend that no changes be made to the reactor vessel internals lift rig should the stresses, discussed in Attachment B, be considered excessive by others because:
 - a. The design weight used in the stress calculations is based on the weight of the lower internals. The lower internals are only removed when a periodic inservice inspection of the vessel is required (once/10 years).
 - b. Prior to removal of the lower internals, all fuel is removed. Thus the concern for handling over fuel is non-existent in this particular case.
 - c. Normal use of the rig is for moving the upper internals which weigh less than one-half of the lower internals. The design weight is based on lifting the lower internals. Thus all the stresses could be reduced by approximately 50 percent and considered well within the ANSI N14.6 criteria for stress design factors.
- 6.2 Review plant operating procedures to include consideration of ANSI N14.6 Sections 5.1.3 through 5.1.8. These sections include requirements for: scheduled periodic testing; special identification and marking; maintenance, repair, testing and use. Westinghouse remarks on addressing these sections are listed in Attachment A, Appendix B, Items 5, 6, and 7.

- 6.3 A proposed response to the requirement of ANSI N14.6, Section 5.2.1, requiring an initial acceptance load test prior to use equal to 150 percent of the maximum load is that the 125 percent of maximum load test that was performed be accepted in lieu of the 150 percent load test.
- 6.4 A proposed response to ANSI N14.6 Section 5.3 which requires, annually, either a 150 percent maximum load test or dimensional, visual and non-destructive testing of major load carrying welds and critical areas follows. (Since the 150 percent load test is very impractical, the approach identified in the following recommendation is to perform a minimum of non-destructive testing.)

a. Reactor Vessel Head Lift Rig

Prior to use and after reassembly of the spreader assembly, lifting lug, and upper lifting legs to the upper portion of the lift rig, visually check all welds. Raise the vessel head slightly above its support and hold for 10 minutes. During this time, visually inspect the sling block lugs to the lifting block welds, and spreader lug to spreader arm weld. If no problems are apparent, continue to lift, monitoring the load cell readout at all times.

Reactor Vessel Internals Lift Rig

Prior to use, visually inspect the rig components and welds while on the storage stand for signs of cracks or deformation. Check all bolted joints to ensure that they are tight and secure. After connection to the upper or lower internals, raise the assembly slightly off its support and hold for 10 minutes. During this time, visually inspect the sling block lugs to the lifting block welds. If no problems are apparent, continue to lift, monitoring the load cell readout at all times.

The above actions do not include a non-destructive test of these welds because: a. Access to the welds for surface examination is difficult. These rigs are in containment and some contamination is present. b. All tensile and shear stresses in the welds are within the allowable stress. c. The items that are welded remain assembled and cannot be misused for any other lift other than their intended function.

- d. To perform non-destructive tests would require:
 - (1) Removal of paint around the area to be examined which is contaminated.
 - (2) Performance of either magnetic particle inspection or liquid penetrant inspection and
 - (3) Repainting after testing is completed.
 - (4) Cleanup of contaminated items.

Performing non-destructive tests on these welds every refueling would increase the critical path refueling time.

Dimensional checking is not included since these structures are large (about 13 feet diameter by 43 feet high) and the results of dimensional checking would always be questionable. Other checks on critical load path parts such as pins, are also not included since an examination of these items would require disassembly of the special lift devices.

6.5 Recommend that a periodic non-destructive surface examination of critical welds and/or parts be performed once every ten years as part of an inservice inspection outage.

WESTINGHOUSE CLASS 3

Although information contained in this report is nonproprietary, no distribution shall be made outside Westinghouse or its licensees without the customer's approval.

ATTACHMENT A to WCAP-10233

Comparison of ANSI N14.6-1978 Requirements for Special Lifting Devices and the Requirements for the Reactor Vessel Head Lift Rig, Reactor Vessel Internals Lift Rig, Load Cell, and the Load Cell Linkage

for

South Carolina Electric and Gas Company Virgil C. Summer Plant

APRIL, 1983

H. H. Sandner, P.E.

Approved

J. R. Marshall, P.E., Manager

Refueling Equipment Engineering

ABSTRACT

The requirements used in the original design, fabrication, testing, maintenance and quality assurance were compared to the ANSI N14.6-1978 requirements for the Virgil C. Summer reactor vessel head and internals lift rig, load cell and load cell linkage. A critical items list per ANSI N14.6 Section 3.1.2 has been prepared and a tabulation of ANSI N14.6 requirements that are, at present, incompatible with the Virgil C. Summer lifting devices has been prepared.

TABLE OF CONTENTS

Section	Title	Page
	ABSTRACT	111
1	PURPOSE	1-1
2	INTRODUCTION	2-1
	2.1 Background	2-1
	2.2 Component Description	2-2
	2.2.1 Reactor Vessel Head Lift Rig	2-2
	2.2.2 Reactor Vessel Internals Lift Rig	2-2
	2.2.3 Load Celi and Load Cell Linkage	2-3
3	DISCUSSION	3-1
4	CONCLUSIONS	4-1
APPENDIX A -	CRITICAL ITEMS LIST PER ANSI N14.6-1978	A-1
APPENDIX B -	TABULATION OF ANSI N14.6-1978 REQUIREMENTS INCOMPATIBLE WITH THE VIRGIL C. SUMMER SPECIAL LIETING DEVICES	B-1

LIST OF ILLUSTRATIONS

Figure	Title	Page
A-1	Reactor Vessel Head Lift Rig	A-6
A-2	Reactor Vessel Internals Lift Rig,	A-10

LIST OF TABLES

Table	Title	Page
2-1	Comparison of the Requirements of ANSI N14.6 and Virgil C. Summer Special Lift Devices	2-4
A-1	Reactor Vessel Head Lift Rig, Critical Items List of Parts per ANSI N14.6-1978	A-3
A-2	Reactor Vessel Head Lift Rig, Critical Items List of Welds per ANSI N14.6-1978	A-5
A-3	Reactor Vessel Internals Lift Rig, Critical Items List of Parts per ANSI N14.6-1978	A-7
A-4	Reactor Vessel Internals Lift Rig Load Cell and Load Cell Linkage, Critical Items List of Welds per ANSI N14.6-1978	A-9

REFERENCES

- Westinghouse Drawing 1098E56 3-Loop Lifting Rig Head, General Assembly.
- Westinghouse Drawing 1143E65 South Carolina Electric and Gas Company, Virgil C. Summer Plant Internals Lifting Rig General Assembly.
- 3. Manual of Steel Construction, Seventh Edition, American Institute of Steel Construction.

SECTION 1

The purpose of this report is to compare the requirements of the special lifting rigs used to lift the reactor vessel head and reactor vessel upper and lower internals with the requirements contained in ANSI N14.6 for special lifting devices.

SECTION 2 INTRODUCTION

ANSI N14.6-1978-"American National Standard for Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Materials" contains detailed requirements for the design, fabrication, testing, maintenance and quality assurance of special lifting devices. NUREG 0612 "Control of Heavy Loads at Nuclear Power Plants", paragraph 5.1.1(4), specifies that special lifting devices should satisfy the guidelines of ANSI N14.6-1978. Subsequently the Nuclear Regulatory Commission (NRC) has requested operating plants to demonstrate compliance with NUREG 0612. To demonstrate compliance with this document, a detailed comparison of the original design, fabrication, testing, maintenance and quality assurance requirements with those of ANSI N14.6 is necessary.

Thus, the ANSI N14.6 document has been reviewed in detail and compared to the requirements used to design and manufacture the reactor vessel head lift rig, the reactor vessel internals lift rig, load cell, and the load cell linkage. This comparison is listed in Table 2-1.

2.1 BACKGROUND

The reactor vessel head and internals lifting rigs were designed and built for the Virgil C. Summer Nuclear Power Plant, circa 1974-76. These devices were designed to the requirement that the resulting stress in the load carrying members, when subjected to the total combined lifting weight, should not exceed the allowable stresses specified in the AISC^[3] code. Also, a 125 percent load test was required on both devices, followed by appropriate non-destructive testing. Westinghouse also required non-destructive tests and inspections on critical load path parts and welds both as raw material and as finished items. These requirements of design, manufacturing and quality-assurance were identified on detailed manufacturing drawing and purchasing documents.

Westinghouse also issued field assembly and operating instructions, where applicable.

2.2 COMPONENT DESCRIPTION

2.2.1 Reactor Vessel Head Lift Rig

The reactor vessel head lift $\operatorname{rig}^{[1]}$ is a three-legged carbon steel structure, approximately 43 feet high and 12 feet in diameter, weighing approximately 21,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and control rod drive mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod sling assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operations, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2.2 Reactor Vessel Internals Lift Rig

The reactor vessel internals lift rig^[2] is a three-legged carbon and stainless steel structure, approximately 30 feet high and 13 feet in diameter weighing approximately 20,000 pounds. It is used to handle the upper and lower reactor vessel internals packages. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The reactor vessel internals lift rig attaches to the internals package by means of three rotolock studs which engage three rotolock inserts located in the internals flange. These rotolock studs are manually operated from the internals rig platform using a handling tool which is

an integral part of the rig. The studs are normally spring retracted upward and are depressed to engage the inserts. Rotating the mechanism locks it in both positions.

2.2.3 Load Cell and Load Cell Linkage

The load cell is used to monitor the load during lifting and lowering the reactor vessel head or internals to ensure no excessive loadings are occurring. The unit is a load sensing device type, rated at 500,000 pounds.

This load cell is a part of the load cell linkage which is an assembly of pins, plates, and bolts that connect the polar crane main hook to the lifting blocks of both the reactor vessel head and internals lift rigs.

TABLE 2-1
COMPARISON OF THE REQUIREMENTS OF ANSI N14.6 AND
VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
1 1.1 to 1.3 2	Scope and Definitions - These sections define the scope of the document and include pertinent definitions of specific items	These sections are definitive, and not requirements.
3 3.1 3.1.1 to 3.1.4	Design Designer's Responsibilities - This section contains requirements for preparing a design specification and its' contents, stress reports; repair procedures; limi- tations on use with respect to environ- mental conditions; marking and nameplate information; and critical items list.	A. No design specification was written concerning these specific requirements. However, assembly and detailed manufacturing drawings and purchasing documents contain the following requirements: (1) Material specification for all the critical load path items to ASTM, ASME specifications or special listed requirements. (2) All welding, weld procedures and welds to be in accordance with ASME Boiler and Pressure Vessel Code - Section IX. (3) Special non-destructive testing for specific critical load path items to be performed to written and approved procedures in accordance with ASTM or specified requirements

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL'C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
		(4) All coatings to be performed to strict compliance with specified requirements.
		(5) Letters of compliance for materials and specifications were required for verification with original specifi- cations.
		B. A stress report was not originally required but has been prepared and is Attachment B.
		C. Repair procedures were not identified.
		D. No limitations were identified as to the use of these devices under adverse environments.
		E. Markings and nameplate information was not addressed.
		F. Critical item lists have been prepared for each device and are identified as Appendix A to this Attachment A.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.2 3.2.1 to 3.2.6	Design Criteria Stress Design Factors - These sections contain requirements for the use of stress design factors of 3 and 5 for allowable stresses of yield and ultimate respec- tively for maximum shear and tensile stresses; high strength material stress design factors; special pins; wire rope and slings to meet ANSI B30.9-1971; and drop-weight tests and Charpy impact test requirements	1. These devices were originally designed to the requirement that the resulting stress in the load carrying members, when subjected to the total combined lifting weight, should not exceed the allowable stresses specified in the AISC code. A stress report (Attachment B) has been generated which addresses the capability of these rigs to meet the ANSI design stress factors. 2. High strength materials are used in some of these devices (mostly for pins, load cell). Although the fracture toughness was not determined, the material was selected based on its fracture toughness characteristics. However, the stress design factors of ANSI N14.6 Section 3.2.1 of 3 and 5 were used in the analysis and the resulting stresses are acceptable. 3. Where necessary, the weight of pins was considered for handling.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.3 3.3.1 to 3.3.8	Design Considerations - These sections contain considerations for; materials of construction, lamellar tearing; decontamination effects; remote engagement provisions; equal load distribution; lock devices; position indication of remote actuators; retrieval of device if disengaged; and nameplates.	Decontamination was not specifically addressed. Locking plates, pins, etc. are used throughout these special lifting devices. Remote actuation is only used when engaging the internals lift rig with the internals and position indication is provided from the operating platform.
3.4 3.4.1 to 3.4.6	Design Considerations to Minimize Decontam- ination Efforts in Special Lifting Device Use - These sections contain fabrication, welding, finishes, joint and machining requirements to permit ease in decontamination.	Decontamination was not specifically addressed. However, the design and manufacture included many of these items, i.e. lock devices, pins, etc.
3.5 3.5.1 to 3.5.10	Coatings - These sections contain provisions for ensuring proper methods are used in coating carbon steel surfaces and for ensuring non-contamination of stainless steel items.	The requirements for coating carbon steel surfaces are contained in a Westinghouse process specification referenced on the assembly and detail drawings when applicable. These specifications require a proven procedure, proper cleaning, preparation, application and final inspection of the coating. These requirements meet the intent of 3.5.1 through 3.5.8. No provisions were included in these designs for consideration of decontamination materials or the use of noncontaminating contact materials for use in stainless steel parts.

62598:1/032283

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
3.6 3.6.1 to 3.6.3	Lubricants - These sections contain requirements for special lubricants to minimize contamination and degradation of the lubricant and contacted surfaces or water pools	On the head lifting rig, threaded con- netions and 63 finishes are coated with Fel/pro N-1000 as indicated on the drawings. On the internals lift device, threaded connections are coated with neolube. On the load cell linkage, silicone grease and neolube are used where applicable as indicated on the drawings.
4 4.1 4.1.1 to 4.1.12	Fabrication Fabricators Responsibilities -These sections contain specific requirements for proper quality assurance, document control, deviation control, procedure control, material identification and certificate of compliance.	All the manufacturers welding procedures and non-destructive testing procedures were reviewed by Westinghouse prior to use. All critical load carrying members require certificates of compliance for material requirements. Westinghouse performed certain checks and inspections during various steps of manufacturing. Final Westinghouse review includes visual, dimensional, procedural, cleanliness, personnel qualification, etc. and issuance of a quality release to ensure conformance with drawing requirements.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
4.2 4.2.1 to 4.2.5	Inspectors Responsibilities -These sections contain requirements for a non-supplier inspector.	Westinghouse Quality Assurance personnel performed some in-process and final inspections similar to those identified in these sections, and issued a Quality Release. (Also see comments to Section 4.1 above)
4.3 4.3.1 to 4.3.3	Fabrication Considerations -These sections contain special requirements for ease in decontamination or control of corrosion.	General good manufacturing processes were followed in the manufacture of these devices. However, the information defined in these sections was not specifically addressed.
5.1 5.1.1 to 5.1.8	Acceptance Testing Maintenance, and Assurance of Continued Compliance Owner's Responsibilities - Sections 5.1.1 and 5.1.2 require the owner to verify that the special lifting devices meet the performance criteria of the design specification by reviewing records and witness of testing.	Both the Reactor Vessel Head and Internal Lift Rigs were proof tested upon completion with a load of approximately 1.25 times the design weight. Upon the completion of the test, all parts, particularly welds, were visually inspected for cracks or obvious deformation. Critical welds were magnetic particle inspected. In addition, the Westinghouse Quality Release verifies that the criteria for letters of compliance for materials and specifications required by the Westinghouse drawings and purchasing documents was satisfied.
	Section 5.1.3 requires periodic functional testing	Maintenance and inspection procedures should be revised to include a visual check of critical welds and parts during lifting to comply with this requirement for functional testing.

TABLE 2-1 (cont) COMPARISON OF REQUIREMENT OF THE ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
	Section 5.1.4 requires operating procedure	Operating instructions for the reactor vessel internals lift rig were furnished to the utility and operating procedures were prepared and are used.
	Sections 5.1.5, 5.1.5.1 and 5.1.5.2 require special identification and marking to prevent misuse.	It is obvious from their designs that these rigs are special lifting devices and can only be used for their intended purpose. Specific identification of the rig can be made by marking, with stencils, the rig name and rated capacity, preferably on the spreader assembly.
	Sections 5.1.6, 5.1.7 and 5.1.8 require the owner to provide written documentation on the maintenance, repair, testing and use of these rigs.	Operating instructions and maintenance instructions should be reviewed to assure that they contain the requirements to address maintenance logs, repair and testing history, damage incidents etc.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6 Section	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
5.2 and 5.3 5.2.1 to 5.2.3 and 5.3.1 to 5.3.8	Acceptance Testing and Testing to Verify Continuing Compliance - These paragraphs require the rigs to be initially tested at 150% maximum load followed by non-destructive testing of critical load bearing parts and welds and also annual 150% load tests or annual non-destructive tests and examinations; qualification of replacement parts.	The head and internals lift rigs were load tested as indicated in Section 5. At each refueling it is suggested that a check of critical welds and parts be included in the maintenance procedures for both lifting devices. Preferably, during the initial lift at each refueling, a visual inspection should be made. Further note that with the use of the load cell for the head and internals, lifting and lowering is monitored at all times. Replacement parts should be in accordance with the original or equivalent requirements.
5.4 5.4.1 to 5.4.2	Maintenance and Repair - This section requires any maintenance and repair to be performed in accordance with original requirements and no repairs are permitted for bolts, studs and nuts.	Maintenance and repair procedures should contain, as much as possible, requirements that were used in the original fabrication. The critical items list of Appendix A contains the original type of non-destructive testing. The procedure should also define bolts, studs and nuts as non-repairable items.

TABLE 2-1 (cont) COMPARISON OF THE REQUIREMENT OF ANSI N14.6 AND VIRGIL C. SUMMER SPECIAL LIFT DEVICES

ANSI N14.6	Description of ANSI N14.6 Requirement	Actual Special Lift Device Requirements
5.5 5.5.1 to 5.5.2	Non-destructive Testing Procedures, Personnel Qualifications, and Acceptance Criteria - This section requires non- destructive testing to be performed in accordance with the requirements of the ASME Boiler and Pressure Vessel Code	Liquid penetrant, magnetic particle, ultrasonic and radiograph inspections were performed on identified items. These were in accordance with ASTM specifications, ASME Code, Westinghouse process specifications or as noted on detailed drawings and provide similar results to the requirement of the ASME Code.
6 6.1 6.2 6.3	Special Lifting Devices for Critical Loads - These sections contain special requirements for items handling critical loads.	It is assumed that compliance with NUREG 0612, Section 5.1 has been demonstrated and therefore this section is not applicable to these devices.

SECTION 3 DISCUSSION

The reactor vessel head and internals lift rigs, load cell and load cell linkage generally meet the intent of the ANSI N14.6 requirements for design and manufacture. However, they are not in strict compliance with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance.

Although no specific design specification was written, the assembly and detailed manufacturing drawings and purchase order documents contain equivalent requirements. A stress report has been prepared for these devices. These devices, for the most part, were manufactured under Westinghouse surveillance with identified hold points, procedure review and personnel qualification which adequately meet these related ANSI requirements. A 125 percent load test was performed on both the head and internals lift rigs followed by the appropriate non-destructive testing.

It is anticipated that 100 percent load test, performed on each device, at each refueling, followed by a visual check of critical welds would be sufficient to demonstrate compliance. This may require modification of Virgil C. Summer operating and maintenance procedures.

Upon completion of the field assembly of the reactor vessel head lifting rig, the assembly procedure calls for a 100 percent load test (lifting of the assembled head), with a visual inspection for any signs of distortion.

SECTION 4 CONCLUSIONS

The review of the ANSI N14.6 requirements and comparison with the original Westinghouse requirements has shown that these items are generally in agreement for the design, fabrication and quality assurance of the lifting devices. However, the lifting devices are not in strict compliance with the ANSI N14.6 requirements for acceptance testing, maintenance and verification of continuing compliance. These specific requirements that are incompatible with the lifting devices are discussed in Appendix B with suggested actions. Westinghouse's objective was to provide a quality product and this product was designed, fabricated, assembled and inspected in accordance with internal Westinghouse requirements. In general, Westinghouse requirements meet the intent of ANSI N14.6 but not all the specific detailed requirements.

APPENDIX A CRITICAL ITEMS LIST PER ANSI N14.6-1978

1. GENERAL

Section 3.1.2 of ANSI N14.6-1978 specifies that the design specification shall include a critical items list, which identifies critical components and defines their critical characteristics for material, fabrication, non-destructive testing and quality assurance.

"Critical items list" is further defined in ANSI N14.6, Section 2 as:

"critical items list. A list that specifies the items of a special lifting device and their essential characteristics for which specified quality requirements shall apply in the design, fabrication, utilization, and maintenance of the device."

Load carrying members and welds of these special lifting devices are considered to be the critical items.

Tables A-1, A-2, A-3 and A-4 are the critical items list of parts and welds for the reactor vessel head lift rig, the reactor vessel internals lift rig and the load cell and load cell linkage. These tables include the material identification, and the applicable volumetric and surface inspections that were performed in the fabrication of these special lifting devices. In some instances, non-destructive testing was not specified since the material selection and strength result in very low tensile stresses and thus, non-destructive testing was not justified.

The material selection for all critical load path items was made to ASTM, ASME or special material requirements. The material requirements were supplemented by Westinghouse imposed non-destructive testing, and/or special heat treating requirements for almost all of the critical items. Westinghouse required all welding, welders, and weld procedures to be in accordance with ASME Boiler and Pressure Vessel Code Section IX

for all welds. Westinghouse required a certificate, or letter of compliance that the materials and processes used by the manufacturer were in accordance with the purchase order and drawing requirements. Westinghouse also performed final inspections on these devices and issued quality releases for the internals and head lifting rigs.

TABLE A-1

REACTOR VESSEL HEAD LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE

CRITICAL ITEMS LIST OF PARTS

PER ANSI N14.6-1978

			Non-destruct	ive Testing
Item ^(a)	Description	Material	Material	Finished
1,3,7	Hook Pin Adaptor Pin Removable Pin	ASTM A564, Type 630	Ultrasonic	Magnetic Particle
2	Side Plate	ASTM A588 Gr. A Q&T	Ultrasonic	Magnetic Particle
4,6,8	Upper Load Cell Adaptor Lower Load Cell Adaptor Sling Block	ASTM A508 Class 2	Ultrasonic	Magnetic Particle
5	Load Cell	17-4 pH, SS Cond. H-1100	Ultrasonic	Liquid Penetrant
9	Link Lug Spreader Lug	ASTM A588, Gr. A, Q&T	Ultrasonic	Magnetic Particle
10,14, 18,21	4" Dia. Upper Sling Leg Pin 4" Dia Lower Sling Leg Pin Upper Lifting Leg Pin 3 1/2" Lower Lifting Leg Pin	ASTM A434 AISI 4340, C1. BD	Ultrasonic	Magnetic Particle
11,13,	Upper Sling Leg Clevis Lower Sling Leg Clevis Upper Lifting Leg Clevis Lower Lifting Leg Clevis		Ultrasonic	Magnetic Particle

⁽a) See figure A-1

TABLE A-1 (cont) REACTOR VESSEL HEAD LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

(-)			Non-destruct	ive Testing
Item(a)	Description	Material	Material	Finished
12,19	Arm Leg	ASTM A306 Gr. 70	Ultrasonic	Magnetic Particle
16	Spreader Arm	ASTM A106, Gr. B		

⁽a) See figure A-1

TABLE A-2

REACTOR VESSEL HEAD LIFT RIG, LOAD CELL, AND LOAD CELL LINKAGE

CRITICAL ITEMS LIST OF WELDS

PER ANSI N14.6-1978

		Non-destructive Testing	
Item	Description	Root Pass	Final
8,9	Lugs to Lifting Block (Full Penetration)	Magnetic Particle	Magnetic Particle Radiograph
15,16	Spreader Arm Lug to Spreader Arm (fillet)	Magnetic Particle	Magnetic Particle

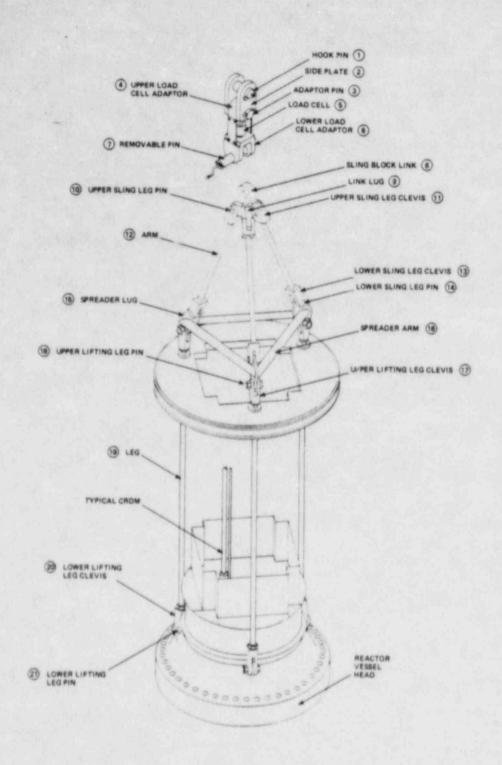


Figure A-1. Reactor Vessel Head Lift Rig

TABLE A-3 REACTOR VESSEL INTERNALS LIFT RIG CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

			Non-destruct	ive Testing
Item(a)	Description	Material	Material	Finished
1	Sling Block	ASTM A508, Class 2	Ultrasonic	Magnetic Particle
2	Sling Block Lug	ASTM A588, Grade A	Ultrasonic	Magnetic Particle
3,7	Upper Clevis Pin Lower Clevis Pin		Ultrasonic	Liquid Penetrant
4,6	Upper Sling Clevis Lower Sling Clevis	ASTM A471 Steel Forging, Class 3	Ultrasonic	Magnetic Particle
5	Sling Rod	ASTM A306 Grade 70 or 80	Ultrasonic	Magnetic Particle
8	Spreader Lug	ASTM A516 normalized or ASTM A637, Grade B Q&T	Ultrasonic	Magnetic Particle
9,13	Spreader and Mounting Block	ASTM A350 LFI Forging	 Ultrasonic	Magnetic Particle
10,12	Spreader Arm Leg Channel	ASTM A36	=	 Visual
11	Leg Lug	ASTM A516, Grade 70	Ultrasonic	Magnetic Particle

⁽a) See figure A-2

TABLE A-3 (cont) REACTOR VESSEL INTERNALS LIFT RIG CRITICAL ITEMS LIST OF PARTS PER ANSI N14.6-1978

(-)			Non-destruct	ive Testing
Item(a)	Description	Material	Material	Finished
14	Load Nut	ASTM A276, Type 304, Hot Rolled, Condition A		
15,16	Rod Housing Guide Sleeve	ASTM A276, Type 304, Hot Rolled, Annealed & Pickled, Condition A	Ultrasonic	Liquid Penetrant
17	Rotolock Stud	ASTM A564, Type 630, 17-4 pH H-1100	Ultrasonic	Liquid Penetrant

⁽a) See figure A-2

TABLE A-4 REACTOR VESSEL INTERNALS LIFT RIG CRITICAL ITEMS LIST OF WELDS PER ANSI N14.6-1978

		Non-destructi	ve Testing
Item	Description	Root Pass	Final
1,2	Lugs to Lifting Block (Full Penetration)	Magnetic Particle	Magnetic Particle Radiograph
8,9	Lug to Spreader Block (Full Penetration)	Magnetic Particle	Magnetic Particle
11,12	Leg Lug to Channel Leg · (fillet)	Magnetic Particle	Magnetic Particle
12,13	Mounting Block to Channel Leg (fillet)	Magnetic Particle	Magnetic Particle

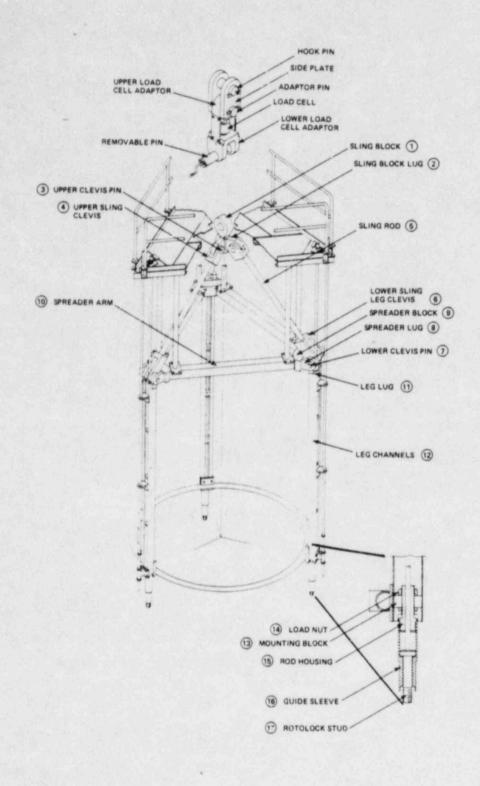


Figure A-2. Reactor Vessel Internals Lift Rig

APPENDIX B

TABULATION OF ANSI N14.6-1978 REQUIREMENTS INCOMPATIBLE WITH THE VIRGIL C. SUMMER LIFTING DEVICES

1. GENERAL

The comparison of the various ANSI N14.6 requirements and those of these lifting devices has shown that these devices are not in strict compliance with all the ANSI N14.6 requirements. Listed below is a tabulation of those sections of ANSI N14.6 considered most important in demonstrating the continued load handling reliability of these special lifting devices. Associated Westinghouse remarks are also listed and could be used as suggested actions and/or responses to demonstrate compliance to the NRC.

la. Requirement:

Para. 3.1.4 - requires the designer to indicate permissible repair procedures and acceptance criteria for the repair.

1b. Remarks:

Any repair to these special lifting devices is considered to be in the form of welding. Should pins, bolts or other fasteners need repair, they should be replaced, in lieu of repair, in accordance with the original or equivalent requirements for material and non-destructive testing. Weld repairs should be performed in accordance with the requirements identified in NF-4000 and NF-5000 (Fabrication and Examination) of the ASME Boiler and Pressure Vessel Code, Section III, Division 1 Subsection NF.

2a. Requirement:

Para. 3.2.1.1 - requires the design, when using materials with yield strengths above 80 percent of their ultimate strengths, to be based on the material's fracture toughness and not the listed design factors.

2b. Remarks:

High strength materials are used in these devices. Although the fracture toughness was not determined, the material was selected based on it's fracture toughness characteristics. However, in lieu of a different stress design factor, the stress design factors listed in 3.2.1 of 3 and 5 were used in the analysis and the resulting stresses are considered acceptable.

3a. Requirement:

Para. 3.2.6 requires material for load-bearing members to be subjected to drop-weight or Charpy impact tests.

3b. Remarks:

Fracture toughness requirements were not identified for all the material used in these special lifting devices. However, the material selection was based on its fracture toughness characteristics.

4a. Requirement:

Para. 5.1 lists <u>Owner Responsibilities</u> and 5.1.2 requires the owner to verify that the special lifting devices meet the performance criteria of the design specification by records and witness of testing.

4b. Remarks:

There wasn't any design specification for these rigs. A 125 percent load test followed by the appropriate non-destructive testing was performed. In addition, the Westinghouse Quality Release, may be considered an acceptable alternate to verify that the criteria for the letters of compliance for materials and specifications required by Westinghouse drawings and purchasing document were satisfied.

5a. Requirement:

Para. 5.1.3 requires periodic functional testing and a system to indicate continued reliable performance.

5b. Remarks:

Maintenance and inspection procedures should include a visual check of critical welds and parts during lifting to comply with this requirement for functional testing.

6a. Requirement:

Para. 5.1.5, 5.1.5.1 and 5.1.5.2 require special identification and marking to prevent misuse.

6b. Remarks:

It is obvious, from their designs, that these rigs are specific lifting devices and can only be used for their intended purpose and parts are not interchangeable. Specific identification of the rig can be made by marking with stencils, the rig name and rated capacity, preferably on the spreader assembly.

7a. Requirement:

Para. 5.1.6, 5.1.7 and 5.1.8 require the owner to provide written documentation on the maintenance, repair, testing and use of these rigs.

7b. Remarks:

Operating instructions and maintenance instructions should be reviewed to assure that they contain the requirements to address maintenance logs, repair and testing history, damage incidents and other items mentioned in these paragraphs.

8a. Requirement:

Para 5.2.1 requires the rigs to be initially tested at 150 percent maximum load followed by non-destructive testing of critical load bearing parts and welds.

8b. Remarks:

Both the reactor vessel head and internals lifting rigs and load cell were proof tested upon completion with a load of approximately 1.25 times the design weight. Upon completion of the test, all parts, particularly welds, were visually inspected for cracks or obvious deformation and critical welds were magnetic particle inspected. In addition the Westinghouse Quality Release verified that the criteria for letters of compliance for materials and specifications required by the Westinghouse drawings and purchasing documents were satisfied.

9a. Requirement:

Para 5.2.2 requires replacement parts to be individually qualified and tested.

9b. Remarks

Replacement parts, should they be required, should be made of identical (or equivalent) material and inspections as originally required. Only pins, bolt and nuts are considered replacement parts for the reactor vessel head and internal liftings.

10a. Requirement:

Para 5.3 requires testing to verify continuing compliance and annual 150 percent load tests or annual non-destructive tests and examinations to be performed.

10b. Remarks

These special lifting devices are used during plant refueling which is approximately once per year. During plant operation these special lifting devices are inaccessable since they are permanently installed and/or remain in the containment. They cannot be removed from the containment unless they are disassembled and no known purposes exist for disassembly. Load testing to 150 percent of the total weight before each use

would require special fixtures and is impractical to perform. Crane capacity could also be limiting. It is suggested that a check (visual) of critical welds and parts be conducted at initial lift prior to moving to full lift and movement for these devices. Further note that with the use of the load cell for the head and internals lift rig, all lifting and lowering is monitored at all times.

2. SUMMARY

The ANSI requirements for periodic checking and functional load testing appear to be most difficult to demonstrate compliance. It is almost impractical to perform the 150 percent load test prior to each use. It is suggested that the proposal to the NRC include a 100 percent load test to be performed with a minimum of non-destructive testing, (visual-only) in the critical parts and welds.

ATTACHMENT B to WCAP-10233

Although information contained in this report is nonproprietary, no distribution shall be made outside Westinghouse or its licensees without the customer's approval.

STRESS REPORT
REACTOR VESSEL HEAD LIFT RIG,
REACTOR VESSEL INTERNALS LIFT RIG
AND THE LOAD CELL LINKAGE
FOR
SOUTH CAROLINA ELECTRIC AND GAS COMPANY
VIRGIL C. SUMMER PLANT

April, 1983

H. H. Sandner, P. E.

Approved: J. R. Marshall, P. E., Manager Refueling Equipment Engineering

ABSTRACT

A stress analysis of the Virgil C. Summer reactor vessel head and internal lift rigs load cell and load cell linkage was performed to determine the acceptability of these devices to meet the design requirements of ANSI N14.6.

ACK NOWLEDGMENT

Acknowledgment is hereby made to the following individuals who contributed to the structural analysis presented in this report.

- J. S. Urban
- F. Peduzzi
- J. Richard

TABLE OF CONTENTS

Section	Title	Page
	ABSTRACT	111
1	INTRODUCTION	1-1
	1.1 Background	1-1
2	COMPONENT DESCRIPTION	2-1
	2.1 Reactor Vessel Head Lift Rig	2-1
	2.2 Reactor Vessel Internals Lift Rig	2-1
	2.3 Load Cell and Load Cell Linkage	2-2
3	DESIGN BASIS	3-1
	3.1 Design Criter:a	3-1
	3.2 Design Weights	3-2
4	MATERIALS	4-1
	4.1 Material Description	4-1
5	SUMMARY OF RESULTS	5-1
	5.1 Discussion or Results	5-1
	5.2 Conclusion	5-3
APPENDIX A	DETAILED STRESS ANALYSIS - REACTOR	
	VESSEL HEAD LIFT RIG	A-1
APPENDIX B	DETAILED STRESS ANALYSIS - REACTOR	
	VESSEL INTERNALS LIFT RIG, LOAD CELL	
	AND LINKAGE	B-1

LIST OF ILLUSTRATIONS

Figure -	Title	Page
5-1	Reactor Vessel Head Lift Rig	5-14
5-2	Reactor Vessel Internals Lift Rig,	5-25
	Load Cell and Linkage	5-

LIST OF TABLES

Table	Title	Page
4-1	Reactor Vessel Head Lift Rig	
	Material and Material Properties	4-2
4-2	Reactor Vessel Internals Lift Rig,	
	Load Cell and Load Cell Linkage	
	Material and Material Properties	4-4
5-1	Summary of Results - Reactor Vessel	
	Head Lift Rig	5-4
5-2	Summary of Results - Reactor Vessel	
	Internals Lift Rig, Load Cell and	
	Load Cell Linkage	5-15

REFERENCES

- George, H., Control of Heavy Loads at Nuclear Power Plants Resolution of Generic Technical Activity A-36, NUREG 0612, July, 1980.
- ANSI N14.6-1978, "Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds (4500kg) or More for Nuclear Material," American National Standards Institute, New York, 1978.
- Westinghouse Drawing 1098E56 3-Loop Lifting Rig-Head, General Assembly
- Westinghouse Drawing 1143E65 South Carolina Electric and Gas Company, Virgil C. Summer Internals Lifting Rig Assembly
- Lin, C. W., "Approximate Evaluation of Dynamic Load Factors for Certain Types of Load Factors for Certain Types of Loading," ASME Paper 70-WA/NE-2.
- Biggs, J. M., <u>Introduction to Structural Dynamics</u>, McGraw-Hill, New York, 1964.
- Gwinn, Jr., J. T., "Stop Over-Designing for Impact Loads," Machine Design, 33, pp. 105-113 (1961).
- Manual of Steel Construction, Seventh Edition, American Institute of Steel Construction.

SECTION 1 INTRODUCTION

The Nuclear Regulatory Commission (NRC) issued NUREG 0612 "Control of Heavy Load at 'uclear Power Plants" in 1980 to address the control of heavy loads to prevent and mitigate the consequences of postulated accidental load drops. NUREG 0612 imposes various training, design, inspection and procedural requirements for assuring safe and reliable operation for the handling of heavy loads. In the containment building, NUREG 0612 requires special lifting devices to meet the requirements of ANSI N14.6-1978 "American National Standard for Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds or More for Nuclear Materials". [2] In general, ANSI N14.6 contains detailed requirements for the design, fabrication, testing, maintenance and quality assurance of special lifting devices.

This report contains the stress analysis performed on the Virgil C. Summer reactor vessel head lift rig, reactor vessel internals lift rig and the load cell and load cell linkage to determine the acceptability of these devices to meet these requirements.

1.1 BACKGROUND

The reactor vessel head lift rig, the reactor vessel internals lifting rig and load cell and load cell linkage, were designed and built for the Virgil C. Summer Nuclear Power Plant, circa 1974-1976. These devices were designed to the requirements that the resulting stress in the load carrying members when subjected to the total combined lifting weight should not exceed the allowable stresses specified in the AISC^[8] code. Also a 125 percent load test was required on both devices, followed by appropriate non-destructive testing. These items were not classified as nuclear safety components and thus requirements for formal documentation of design requirements and stress reports were not applicable. Thus, stress reports and design specifications were not

formally documented. Westinghouse defined the design, fabrication and quality assurance requirements on detailed manufacturing drawings and purchase order documents. Westinghouse also issued field assembly and operating instructions, where applicable.

SECTION 2 COMPONENT DESCRIPTION

2.1 REACTOR VESSEL HEAD LIFT RIG

The reactor vessel head lift rig^[3] is a three-legged carbon steel structure, approximately 43 feet high and 12 feet in diameter, weighing approximately 21,000 pounds. It is used to handle the assembled reactor vessel head.

The three vertical legs and control rod drive mechanism (CRDM) platform assembly are permanently attached to the reactor vessel head lifting lugs. The tripod sling assembly is attached to the three vertical legs and is used when installing and removing the reactor vessel head. During plant operations, the sling assembly is removed and the three vertical legs and platform assembly remain attached to the reactor vessel head.

2.2 REACTOR VESSEL INTERNALS LIFT RIG

The reactor vessel internals lift rig^[4] is a three-legged carbon and stainless steel structure, approximately 30 feet high and 13 feet in diameter weighing approximately 20,000 pounds. It is used to handle the upper and lower reactor vessel internals packages. It is attached to the main crane hook for all lifting, lowering and traversing operations. A load cell linkage is connected between the main crane hook and the rig to monitor loads during all operations. When not in use, the rig is stored on the upper internals storage stand.

The reactor vessel internals lift rig attaches to the internals package by means of three rotolock studs which engage three rotolock inserts located in the internals flange. These rotolock studs are manually operated from the internals lift rig platform using a handling tool which is an integral part of the rig. The studs are normally spring retracted upward and are depressed to engage the inserts. Rotating the mechanism locks it in both positions.

2.3 LOAD CELL AND LOAD CELL LINKAGE

The load cell is used to monitor the load during lifting and lowering the reactor vessel head or internals to ensure no excessive loadings are occurring. The unit shall be a load sensing clevis type rated at 500,000 pounds. This load cell is a part of the load cell linkage which is an assembly of pins, plates, and bolts that connect the polar crane main hook to the lifting blocks of both the reactor vessel head and internals lift rig.

SECTION 3 DESIGN BASIS

3.1 DESIGN CRITERIA

NUREG 0612, para raph 5.1.1(4) states that special lifting devices should satisfy the guidelines of ANSI N14.6. Further, NUREG 0612, 5.1.1(4) states: "In addition, the stress design factor stated in Section 3.2.1.1 of ANSI N14.6 should be based on the combined maximum static and dynamic loads that could be imparted on the handling device based on characteristics of the crane which will be used. This is in lieu of the guideline in Section 3.2.1.1 of ANSI N14.6 which bases the stress design factor on only the weight (static load) of the load and of the intervening components of the special handling device".

It can be inferred from this paragraph that the stress design factors specified in Section 3.2.1.1 of ANSI N14.6 (3 and 5) are not all inclusive. Also, it can be inferred that the specified ANSI N14.6 stress design factors should be increased by an amount based on the crane dynamic characteristics.

The dynamic characteristics of the crane would be based on the main hook and associated wire ropes holding the hook. Most main containment cranes use sixteen (16) or more wire ropes to handle the load. Should the crane hook suddenly stop during the lifting or lowering of a load, a shock load could be transmitted to the connected device. Because of the elasticity of the sixteen or more wire ropes, we consider the dynamic factor for a typical containment crane to be not much larger than 1.0.

Even if the worst conditions existed, the maximum design factor that is recommended by most design texts [5, 6, 7] is a factor of two for

loads that are suddenly applied. The stress design factors required in Section 3.2.1.1 of ANSI N14.6 are:

3 x (weight) < Yield Strength 5 x (weight) < Ultimate Strength

The factor of 3 specified, based on yield strength, is certainly large enough to compensate for suddenly applied loads, where the dynamic impact factor would be as high as 2.0.

To provide flexibility on stress design factor, the analysis of the devices was performed with stress design factors of 1, 3 and 5. Thus, any stress design factor may be easily applied to satisfy any concerns.

3.2 DESIGN WEIGHTS

The following design weights were used in the analysis of the lifting devices:

3.2.1 Reactor Vessel Head Lift Rig

The design weight is 270,000 pounds which is the total weight of the assembled head and the lifting device.

3.2.2 Reactor Vessel Internals Lift Rig

The design weight for:

- a. The Lower Assembly, Items 13 through 17 of calculations; is 202,000 pounds.
- b. The design weight for the rest of the rig is 230,000 pounds.

SECTION 4
MATERIALS

4.1 MATERIAL DESCRIPTION

The materials and material properties for the reactor vessel head lift rig, the reactor vessel internals lift rig and load cell linkage are listed in Tables 4-1 and 4-2.

TABLE 4-1
REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE MATERIAL
AND MATERIAL PROPERTIES

Item ^(a)	Description	Material	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
1,3,7	Hook Pin Adaptor Pin Removable Pin	ASTM A564, Type 630	105	135
2	Side Plate	ASTM A588, Grade A, Q&T	60	80
4,6,8	Upper Load Cell Adaptor Lower Load Cell Adaptor Sling Block	ASTM A508, Class 2	50	80
5	Load Cell	17-4 pH, SS Condition H-1100	115	140
9,15	Link Lug	ASTM A588, Grade A, Q&T	60	80
10,14, 18,21	4" Dia. Upper Sling Leg Pin 4" Dia. Lower Sling Leg Pin Upper Lifting Leg Pin 3 1/2" Lower Lifting Leg Pin	ASTM A434 AISI 4340 Class BD	110	140
11,13, 17,20	Upper Sling Leg Clevis Lower Sling Leg Clevis Upper Lifting Leg Clevis Lower Lifting Leg Clevis	ASTM A237, AISI 4340 Steel, Class G	110	135

⁽a) See figure 5-1.

TABLE 4-1 (cont)

REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE MATERIAL

AND MATERIAL PROPERTIES

Item ^(a)	Description	Material Material	Yield Strength ^S y (ksi)	Ultimate Strength Sult (ksi)
12,19	Arm Leg	ASTM A306 Grade 70	35	70
16	Spreader Arm	ASTM A106 Grade B	35	60

(a) See figure 5-1.

TABLE 4-2
REACTOR VESSEL INTERNALS LIFT RIG MATERIAL
AND MATERIAL PROPERTIES

Item ^(a)	Description	Material	Yield Strength Sy (ksi)	Ultimate Strength Sult (ksi)
1	Sling Block	ASTM A508, Class 2	50	80
2	Sling Block Lug	ASTM A588, Grade A	60	80
3,7	Upper Clevis Pin Lower Clevis Pin	ASTM A564, Type 630	105	135
4,6	Upper Sling Clevis Lower Sling Clevis	ASTM A471, Steel Forging, Class 3	95	110
5	Sling Rod	ASTM A306, Grade 70 or 80	35	70
8	Spreader Lug	ASTM A516 Normalized or ASTM A637, Grade B, Q&T	38	70
9,13	Spreader and Mounting Block	ASTM A350 LFI Forging	30	60
10,12	Spreader Arm Leg Channels	ASTM A36	36	58
11	Leg Lug	ASTM A516, Grade 70	38	70
14	Load Nut	ASTM A276, Type 304 Hot Rolled, Cond. A	30	75
15,16	Rod Housing Guide Sleeve	ASTM A276, Type 304 Hot Rolled, Annealed and Pickled, Cond. A	30	75
17	Rotolock Stud	ASTM A564, Type 630, 17-4 pH, H-1100	115	140

⁽a) See figure 5-2.

SECTION 5 SUMMARY OF RESULTS

Tables 5-1 and 5-2 summarize the stresses on each of the parts which make up the reactor vessel head, load cell and load cell linkage and the internals lift rig. All of the tensile and shear stresses, meet the design criteria of Section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with accompanying allowable stress limits of yield and ultimate strength, respectively. In addition, all of the tensile and shear stresses meet the requirement of not exceeding the allowables of the AISC^[8] code.

5.1 DISCUSSION OF RESULTS

5.1.1 Application of ANSI N14.6 Criteria

Both the reactor vessel head and internals lift rig were originally designed to the requirement that all resulting stresses in the load carrying members, when subjected to the total combined lifting weight, should not exceed the allowable stresses specified in the AISC^[8] code.

The design criteria of Section 3.2.1.1 of ANSI N14.6, requiring application of stress design factors of three and five with the accompanying allowable stresses, are to be used for evaluating load bearing members of a special lifting device when subjected to loading conditions resulting in shear or tensile stresses. Application of these design load factors to other loading conditions is not addressed in ANSI N14.6. However, these two stress design factors have been used to determine the stresses of the load carrying members when subject to other loading conditions, viz. bending, bearing. This is an extremely conservative approach and in several instances the resulting stresses exceed the accompanying allowable stress limit.

5.1.2 Structures Loaded in Bearing

The parts of the internals lift rig that do not meet the ANSI N14.6 criteria (3 and 5) when analyzed for bearing stresses are the lower sling leg clevis (item 6), the lower clevis pin (item 7), the spreader lug (item 8), the leg lug (item 11), and the rotolock stud (item 17).

However, since bearing stresses are localized stresses, they can, if necessary, be considered under Section 3.2.1.2, which states that the stress design factors of 3.2.1.1 are not intended to apply to situations where high local stresses are relieved by slight yielding. None of the bearing stresses reach the yield stress, and in fact, all of the bearing stresses meet the design criteria of the AISC^[8] code.

5.1.3 Combined Stresses

The combined tensile stress from bending and tension, in the spreader lug (item 8), of the internals lift rig exceeds the Section 3.2.1.1 criteria.

Bending, however, is not a uniform stress, but a local fiber stress, and is at maximum at the outermost fiber. Even if the fiber stress reached anywhere near the yield stress, the rest of the cross-section could assume the additional load. As indicated above, bending too can be considered under Section 3.2.1.2. Bending contributes to the major portion of the stress shown in the table, and, as a result, as shown, the tensile stress without bending is extremely low and well within the Section 3.2.1.1 criteria. The combined stress also meet the AISC code criteria.

5.1.4 Fillet Weld Stresses

The fillet weld connecting the leg lug (item 11) to the leg channel (item 12) on the internals lift rig meets the ASME criteria for weld

stresses based on base material properties. However, when applying the ANSI N14.6 3W and 5W criteria to the nominal stress value, the ASME allowable stress value is exceeded. But, since the ANSI N14.6 criteria is satisfied for this weld, it is considered acceptable.

5.2 CONCLUSION

Application of the ANSI N14.6 criteria of (3 and 5) to these special lifting devices results in acceptable stress limits for tensile and shear stresses. Application of this criteria to all structural members subject to other types of loadings tend to result in oversimplified conservatism and with some stresses exceeding the accompanying allowable limits. However, when using the more appropriate criteria for those cases not addressed by the ANSI N14.6 criteria the stresses are within the appropriate allowable limits. In conclusion, these special lift devices meet the ANSI N14.6 criteria for tensile and shear stresses and meet other appropriate criteria for loading conditions that result in combined and bearing stresses.

TABLE 5-1 SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE

Item(a)	Part Name	Calculated St	Material Allowabl (ksi)				
No.	And Material	Designation	M(p)	Value 3W	5W	S _y (c)	S _{ult} (d)
1	Hook Pin	Shear	3.1	9.3	15.5	105	135
	ASTM A564,	Bearing on Pin	4.1	12.3	20.5		
	Type 630	Bearing on Side Plate	4.5	13.5	22.5		
		Bending	11.8	35.4	59.0		
2	Side Plate	Tension at 7.515" hole	3.8	11.4	19.0	60	80
	ASTM A588	Bearing at 6.515" hole	5.2	15.6	26.0		
	Gr. A, Q&T	Shear Tear-out					
		at 7.515" hole	3.8	11.4	19.0		
		at 6.515" hole	2.2	6.6	11.0		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)		Calculated S	Material Allowable				
No.	Part Name And Material	Designation	M(P)	Value 3W	5W	S _y (c)	Sult (d)
3	Adaptor Pin	Shear	4.1	12.3	20.5	105	135
	ASTM A564	Bearing on Pin	5.2	15.6	26.0		
	Type 630	Bearing on Upper	5.4	16.2	27.0		
		Load Cell Adaptor					
		Bending	19.5	58.5	97.5		
4	Upper Load	Tension at 6.515 hole	3.3	9.9	16.5	50	80
	Cell Adaptor	Bearing	5.4	16.2	27.0		
	ASTM A508,	Thread Shear	7.0	21.0	35.0		
	Class 2	Shear Tear-out at 6.515" hole	3.3	9.9	16.5		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE

Item(a)	Part Name And Material	Calculated Stresses (ksi) Value					Material Allowable (ksi)	
No.		Designation	W(p)	3W	5W	s _y (c)	S _{ult} (d)	
5	Load Cell	Tension	14.4	43.2	72.0	115	140	
	17-4 pH S/S	Thread Shear	7.0	21.0	35.0			
6	Lower Load	Tension at Pin Hole	3.3	9.9	16.5	50	80	
	Cell Adaptor	Bearing	6.9	20.7	34.5			
	ASTM A508	Thread Shear	6.7	20.1	33.5			
	Class 2	Shear Tear-out at Pin Hole	3.3	9.9	16.5			
7	Removable Pin	Shear	4.2	12.6	21.0	105	135	
	ASTM A564	Bearing on Pin	5.1	15.3	25.5			
	Type 630	Bearing on Link	5.7	17.1	28.5			
		Bending	16.9	50.7	84.5		Final P	

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)	Part Name	Calculated Str	Material Allowable (ksi)				
No.	And Material	Designation	M(p)	Value 3W	5W	S _y (c)	S _{ult} (d)
8	Sling Block	Tension at 6.515" hole	3.3	9.9	16.5	50	80
	Link	Shear at 6.515" hole	3.3	9.9	16.5		
	ASTM A508,	Bearing on Pin	5.1	.15.3	25.5		
	Class 2	Tension at Cylindrical Section	5.4	16.2	27.0		
9	Link Lug	Tension at 4.015" hole	3.5	10.5	17.5	60	80
	ASTM A588	Shear Tear at hole	3.5	10.5	17.5		
	Gd.A, Q&T	Bearing at 4.015" hole	6.1	18.3	30.5		
		Combined Stress from Tension and Bending	5.7	17.1	28.5		
4.1		Vertical Shear	1.8	5.4	9.0		Trail to

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE

Item ^(a)	Court Name	Calculated S	tresses (ksi) Value		Material Allowable (ksi)	
No.	Part Name And Material	Designation	м(р)	3W	5W	S _y (c)	Sult
10	4" Dia. Upper	Shear	3.8	11.4	19.0	110	140
10	Sling Leg	Bearing on Pin	6.1	18.3	30.5		
	Pin	Bearing on Upper	6.4	19.2	32.0		
	ASTM A434	Sling Leg Clevis					
		Bending	14.6	.43.8	73.0		
11 0	Upper Sling	Bearing	6.4	19.2	32.0	110	135
	Leg Clevis	Thread Shear	2.7	8.1	13.5		
	ASTM A237	Tension at Thd Relief	1.7	5.1	8.5		
	AISI E4340	Tension at Pin Hole	5.4	16.2	27.0		
	Steel, Cl. G	Shear Tear-out	5.4	16.2	27.0		
12	Arm	Thread Shear	2.7	8.1	13.5	35	70
	ASTM A306	Thread Tension	8.9	26.7	44.5		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

6265B:1/032483

Item(a)	Part Name And Material	Calculated St	Material Allowable (ksi)				
No.		Designation	M(P)	Value 3W	5W	Sy ^(c)	S _{ult} (d)
13	Lower Sling	Bearing	6.4	19.2	32.0	110	135
	Leg Clevis	Thread Shear	2.7	8.1	13.5		
	ASTM A237	Tension at Thd Relief	1.8	5.4	9.0		
	AISI E4340	Tension at Pin Hole	5.4	16.2	27.0		
	Steel, Class G	Shear Tear-out	5.4	16.2	27.0		
14	4" Dia. Lower	Shear	3.9	11.7	19.5	110	140
	Sling Leg	Bearing on Pin	6.9	20.7	34.5		
	Pin	Bearing on Spreader Lug	6.4	19.2	32.0		
	ASTM A434	Bending	13.6	40.8	68.0		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

(a)		Calculated St	Material Allowable				
Item ^(a)	Part Name			Value			ksi)
No.	And Material	Designation	M(p)	3W	5W	Sy (c)	S _{ult} (d)
15	Spreader Lug	Bearing on 4.005" Hole	7.0	21.0	35.0	60	80
	ASTM A588	Tension at Upper Hole	4.7	14.1	23.5		
	Gd.A Steel Plate	Shear Tear-out	4.7	14.1	23.5		
16	Spreader Arm ASTM A106	Nominal Compression Stress	1.7	5.1	8.5	35 F _a = 195 18 ^(f)	60 645 ^(e)
	Gd.B Seamless	Weld Shear	1.0	3.0	5.0	18(f)	

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi)
- (d) Sult is the ultimate strength of the material (ksi)
- (e) F_a is the compressive buckling strength of the material (ksi)
- (f) Stress limit for fillet weld from ASME Boiler and Pressure Vessel Code, Section III Division 1 -Subsection NF 1980 Edition, Table NF-3292 1-1, page 43.

Item(a)	Part Name	Calculated S	Material Allowable (ksi)				
No.	And Material	Designation	M(p)	Value 3W	5W	Sy ^(c)	S _{ult} (d)
17	Upper Lifting	Bearing	5.9	17.7	29.5	110	135
	Leg Clevis	Thread Shear	2.5	7.5	12.5		
	ASTM A237	Tension at Thd Relief	1.7	5.1	8.5		
	AISI E4340	Tension at Pin Hole	5.0	15.0	25.0		
	Steel, Class G	Shear Tear-out	5.0	15.0	25.0		
18	Upper Lifting	Shear	3.6	10.8	18.0	110	140
	Leg Pin	Bearing on Pin	6.4	19.2	32.0		
	ASTM A434	Bearing on Clevis	5.9	17.7	29.5		
		Bending	12.5	37.5	62.5		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

TABLE 5-1 (cont) SUMMARY OF RESULTS REACTOR VESSEL HEAD LIFT RIG AND LOAD CELL LINKAGE

Item(a)	Part Name	Calculated S	Material Allowable (ksi)				
No.	And Material	Designation	М(Р)	Value 3W	5W	Sy(c)	Sult
19	Leg	Thread Shear	2.5	7.5	12.5	35	70
	ASTM A306 Gd.70	Thread Tension	8.1	24.3	40.5		
20	Lower Lifting	Bearing	6.7	20.1	33.5	110	135
	Leg Clevis	Thread Shear	2.5	7.5	12.5		
	ASTM A237	Tension at Thd Relief	1.8	5.4	9.0		
	AISI E4340	Tension at Pin Hole	4.5	13.5	22.5		
	Steel, Class G	Shear Tear-out	4.5	13.5	22.5		

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

(2)		Laiculated	Stresses (ksi				Material Allowable	
(a)	Part Name	OF PERSON AND DESCRIPTION OF THE PERSON AND	/6\	Value		(6)	ksi)	
10.	And Material	Designation	M(p)	3W	5W	Sy (c)	S _{ult} (d	
21	3 1/2" Lower	Shear	4.7	14.1	23.5	110	140	
	Lifting Leg Pin	Bearing on Lug	8.6	25.8	43.0			
	ASTM A434	Bearing on Clevis	6.7	20.1	33.5			
	AISI 4340	Bending	18.0	54.0	90.0			
	Class BD							

- (a) See figure 5-1 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

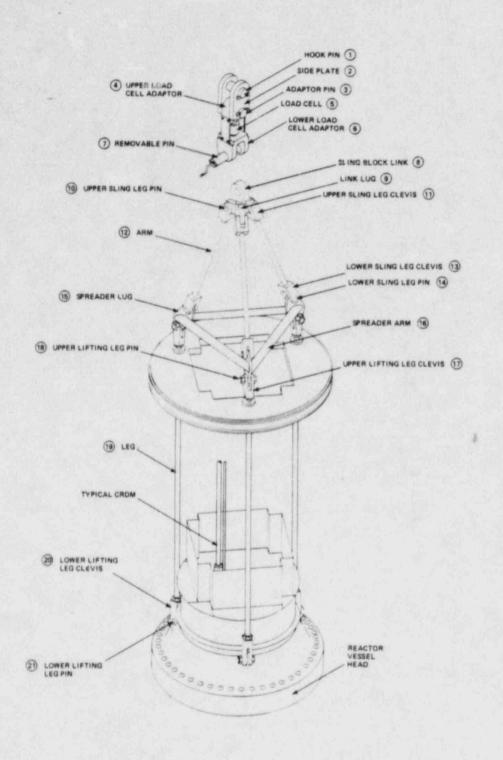


Figure 5-1. Reactor Vessel Head Lift Rig

TABLE 5-2 SUMMARY OF RESULTS REACTOR VESSEL INTERNALS LIFT RIG

		Calculated Stre	sses (ksi))		Material	Allowable
Item ^(a)	Part Name		Value			(ksi)	
No.	And Material	Designation	М(р)	3W	5W	S _y (c)	Sult
1	Sling Block	Tension at 6.515" Dia. Hole	2.8	8.4	14.0	50	80
	ASTM A508 Class 2	Bearing at 6.515" Dia. Hole Shear Tear-out at 6.515"	4.4	13.2	22.0		
		Dia. Hole Tension at 8.00"	2.8	8.4	14.0		
		Cylindrical Section	4.6	13.8	23.0		
2	Sling Block Lug	Tension at 4.015" Dia. Hole	3.6	10.8	18.0	60	80
	ASTM A588 Grade A Steel	Bearing at 4.015" Dia. Hole Shear Tear-out at 4.015"	6.3	18.9	31.5		
		Dia. Hole Vertical Shear at Lug	3.6	10.8	18.0		
		Root Weld Combined Tension from Bend-	1.5	4.5	7.5		
		ing and Tension at Lug Weld	5.2	15.6	26.0		

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item ^(a)		Calculated Stres	Material Allowable				
Item'	Part Name		7.	Value			ksi)
No.	And Material	Designation	M(p)	3W	5W	S _y (c)	S _{ult} (d)
3	Upper Clevis	Shear Stress	4.0	12.0	20.0	105	135
	Pin	Bearing Stress					
	ASTM A564	on Sling Block Lug	6.3	18.9	31.5		
	Type 630	Bearing on Upper Sling Clevis	5.2	15.6	26.0		
		Bending Stress	15.7	47.1	78.5		
4	Upper Sling	Tension at 4.015" Dia. Hole	4.0	12.0	20.0	95	110
	Clevis	Bearing at 4.015" Dia. Hole	5.2	15.6	26.0		
	ASTM A471	Shear Tear-out at 4.015"	17. 71				
	Steel Forging	Dia. Hole	4.0	12.0	20.0		
YE 30	Class 3	Thread Shear	4.2	12.6	21.0		

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)	Part Name And Material	Calculated Stre	Material Allowable (ksi)				
No.		Designation	M(p)	Value 3W	5W	Sy ^(c)	S _{ult} (d)
5	Sling Rod	Thread Shear	4.2	12.6	21.0	35	70
	ASTM A306	Thread Tension	9.0	27.0	45.0		
	Grade 70 or 80	Tension at Thread Relief	9.7	29.1	48.5		
6	Lower Sling	Tension at 4.015" Dia. Hole	4.0	12.0	20.0	95	110
	Leg Clevis	Bearing	30.0	90.0	150.0		
	ASTM A471	Bending at 4.015" Dia. Hole	17.8	153.4	89.0		
	Steel Forging	Combined Max. Bending and					
1996	Class 3	Tension at 4.015" Dia. Hole	21.9	65.7	109.5		
	Residence of the second	Thread Shear	4.2	12.6	21.0		

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)	Part Name	Calculated Str	esses (ks1	Value		Material Allowable (ksi)	
No.	And Material	Designation	M(p)	3W	5W	S _y (c)	Sult
7	Lower Clevis	Maximum Shear at Clevis	8.0	24.0	40.0	105	135
	Pin	Maximum Bearing at Clevis	30.0	90.0	150.0		
	ASTM A564	Bending	15.3	.45.9	76.5		
	Type 630						
	Precipitation						
	Hardening Stair	i-					
	less Steel Age						
	Treated at 1150°F						
	for 4 hours, ai						
	cooled 135,000						
	Min. Tensile						
	Strength RC 28-						

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item(a)	Part Name	Calculated Stre	sses (ksi	Calculated Stresses (ksi) Value		Material Allowab (ksi)	
No.	And Material	Designation	M(p)	3W	5W	S _y (c)	S _{ult} (d)
8	Spreader Lug	Tension at 4.015" Dia. Hole	2.2	6.6	11.0	38	70
1.0	ASTM A516	Bending at 4.015" Dia. Hole	13.3	39.3	66.5		
	Steel Plate,	Combined Tension and Bending	15.4	46.2	77.0		
	Normalized, or A637, Gr. B, Q&T	Bearing	23.1	69.3	119.5		
9	Spreader Block ASTM A350 LFI Forging	Bearing on Spreader Block	3.0	9.0	15.0	30	60
	Steel						

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

1.1	1.	Calculated Str	Material	Material Allowable			
Item(a)	Part Name			Value			ksi)
No.	And Material	Designation	М(р)	3W	5W	S _y (c)	Sult (d)
10	Spreader Arm ASTM A36 Steel	Nominal Compression Stress	3.0	9.0	15.0	36 F _a = 18,	58 929(e)
11	Leg Lug	Tension 4.015" Dia. Hole	2.6	7.8	13.0	38	70
	ASTM A516 Grade 70 Steel	Bending at 4.5" Dia. Hole Comb. Max. Bending Tension	7.9	23.7	39.5		
		at 4.015" Dia. Hole	10.4	31.2	52.0		
		Bearing	18.8	56.4	94.0		
574 3		Weld Stresses	9.4	28.2	47.0	21 ^(f)	

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_v is the yield strength of the material (ksi)
- (d) S_{ult} is the ultimate strength of the material (ksi)
- (e) F_a = allowable compression stress to prevent buckling in absence of bending moment
- (f) Stress limit for fillet welds from ASME Boiler and Pressure Vessel Code, Section III, Division 1 -Subsection NF 1980 Edition, Table NF-3292.1-1, page 43.

Item(a)	Part Name	Calculated Stres	ses (ksi)) Value		Material Allowable (ksi)	
No.	And Material	Designation	M(p)	3W	5W	S _y (c)	Sult (d)
12	Leg Channels ASTM A36 Steel	Tension in Legs	5.6	16.8	28.0	36	58
13	Mounting Block ASTM A350	Bearing of Load Nut to Mounting Block	1.8	5.4	9.0	30	60
	LFI Forging Steel (No CVN Test Req'd.) Welds: E70-18 Electrodes	Shear in Mounting Block Welds	3.1	9.3	15.5	18 ^(f)	

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_v is the yield strength of the material (ksi)
- (d) S_{ult} is the ultimate strength of the material (ksi)
- (f) Stress limit for fillet welds from ASME Boiler and Pressure Vessel Code, Section III, Division 1 Subsection NF 1980 Edition, Table NF-3292.1-1, page 43.

Item ^(a) No.	David Harry	Calculated St	Material Allowable				
	Part Name And Material	Designation	M(p)	Value 3W	5W	s _y (c)	Sult
14	Load Nut ASTM A276	Thread Shear Bearing of Load Nut	4.1	12.3	20.5	30	75
	Type 304, Hot Rolled, Cond. A	to Mounting Block	1.8	5.4	9.0		
15	Rod Housing ASTM A276,	Thread Shear at 4.00-4UNC-2A Thd.	4.1	12.3	20.5	30	75
	Type 304 SST, Hot Rolled,	Thread Shear at 5.0-6UN-2A Thd.	3.5	10.5	17.5		
	Annealed and Pickled, Cond. A	Tension at Thread Relief	8.4	25.2	42.0		

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

Item ^(a) No.	Part Name	Calculated St	Material Allowable (ksi)				
	And Material	Designation	M(p)	Value 3W	5W	S _y (c)	S _{ult} (d)
16	Guide Sleeve ASTM A276,	Thread Shear Tension at Thd Relief	3.5	10.5	17.5 41.0	30	75
	Type 304 SST, Hot Rolled, Annealed, and Pickled, Cond. A	Bearing of Guide Sleeve to Rotolock Stud	9.9	29.7	49.5		

- (a) See figure 5-2 for location of item number and section
- (b) W is the total static weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi)
- (d) S_{ult} is the ultimate strength of the material (ksi)

(-)		Calculated Str	Material Allowable				
Item ^(a)	Part Name			Value		(ksi)	
No.	And Material	Designation	M(p)	3W	5W	Sy (c)	S _{ult} (d)
17	Rotolock Stud	Tension at Section A-A	19.5	58.5	97.5	115	140
	ASTM A564,	Shear of Stud Lands	10.5	31.5	52.5		
	Type 630,	Bearing on Stud Head	9.9	29.7	49.5		
7	17-4 Precipita-	Bending in Lands	14.4	43.2	72.0		
	tion Hardening	Compressive Bearing Stress	29.8	89.4	149.0		
	Stainless Steel	On Land Surfaces					
	Age Treated at						
F 1.8	1100°F for 4						
	hours and air						
	cooled T min =						
	140,000 psi						

- (a) See figure 5-2 for location of item number and section
- (b) W is the total stacic weight of the component and the lifting device
- (c) S_y is the yield strength of the material (ksi) (d) S_{ult} is the ultimate strength of the material (ksi)

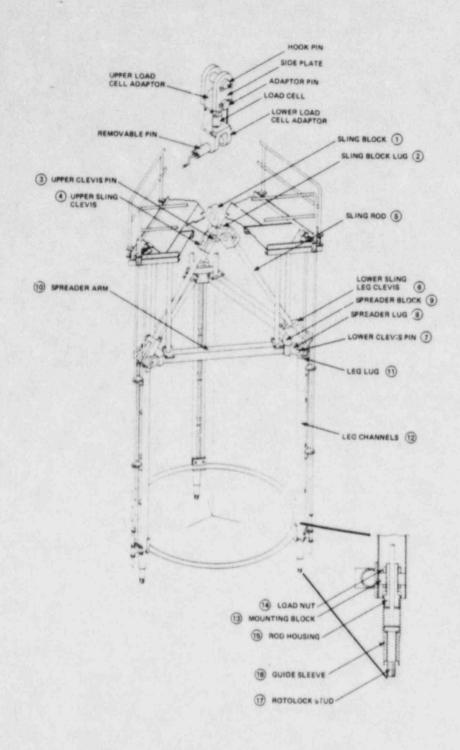


Figure 5-2. Reactor Vessel Internals Lift Rig, Load Cell, and Linkage

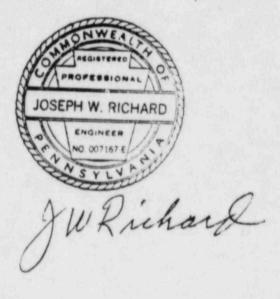
APPENDIX A

DETAILED STRESS ANALYSIS - REACTOR VESSEL HEAD LIFT RIG, LOAD CELL AND LOAD CELL LINKAGE

This appendix provides the detailed stress analysis for the Virgil C. Summer reactor vessel head lift rig and the load cell and load cell linkage, in accordance with the requirements of ANSI N14.6. Acceptance criteria used in evaluating the calculated stresses are based on the material properties given in Section 4.

UCG-27694	Virgil C.	Summer	PAGE 1 OF 42
R. V. Head Lift Rig Assembly		PDC	
F. Peduzzi F. C. Peduzzi	2/83 J. Ric	hard & Rich	ma 2/83

- The purpose of this analysis is to determine the acceptability of this rig to the requirements of ANSI N14.6.
- 2. The results show that all stresses are within the allowable stresses.



		Original Issue	F. Peduzzi
, REVISION NO.	DATE	DESCRIPTION	BY

RESULTING REPORTS, LETTERS OR MEMORANDA:

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

E Coactor Ves	sel Head Lift Rig	PAGE 2 OF 42 OATE CHE'D BY OATE
JECT	AUTHOR	Pichard GROUP
CGE	F.C. Poducy 2/83	GROUP REE
UCG-27594		REE
	19 SPREADER LUG 19 LEG TYPICAL CROM 10 LOWER LIFTING LEG CLEVIS	HOOK PIN ① SIDE PLATE ② ADAPTOR PIN ③ LOWER LOAD CELL ADAPTOR ⑥ SLING BLOCK LINK ③ LINK LUG ⑥ LUPPER SLING LEG CLEVIS ① LOWER SLING LEG CLEVIS ② LOWER SLING LEG PIN ④ PREADER ARM ⑥ UPPER LIPTING LEG CLEVIS ①
3	LOWER LIFTING	REACTOR VESSEL HEAD
	LOWER LIFTING LEG PIN	
REV. REV.	AUTHOR DATE	CHK'D. BY DATE CHK D. BY

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

Reactor Ve	ssel Head Lift Rig		PAGE 3 0	42
CGE	F.C. Polusi 21	73 Priha	S3 DATE CHK'D. BY	DATE
UCG-27694	CALC NO. UD	Gre NO.	GROUP	

DESIGN WEIGHT

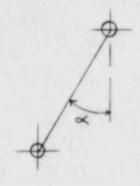
240,000 lbs from Assy DWG (1098 ESb)

21,000 lbs Rig and Platform Assy weight

9,000 lbs Contingencies

270,000 lbs

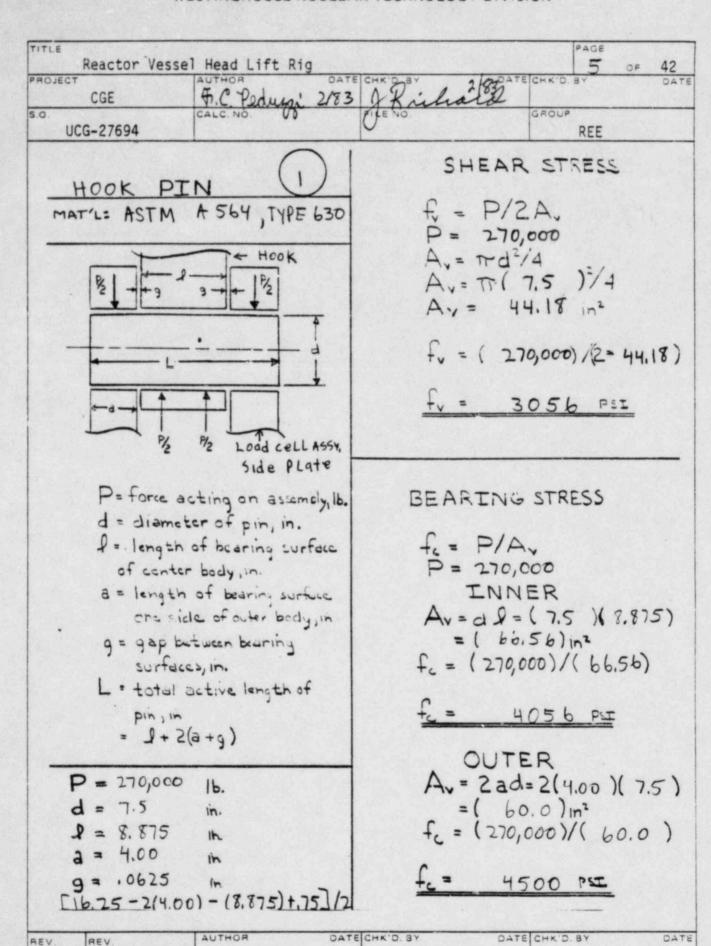
SLING LEG ANGLE



d = 22.8°

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

	essel Head Lift Rig		4 OF
CGE	F. C. Pedus 2/8	of hichards 83	HK'O. BY
UCG-27694	CALC NO. 00	FIGE NO.	REE
LOAD C	ELL LINKAGE AS		
		HOOK	
		HOOK	^
			_
		- ADAPTO	R PIN (3)
		LOAD C	~
			_
	9 9-	LOWER CELL AD	LCAD 6
		Remove	ible 7
	AUTHOR DATE		



DATE

NO

Reactor Vess	Head Lift Rig	ECHKO AT NOTAT	E CHK'D	BY	OF	42
CGE	F.C. Codugi 2/83	S Richard	Caoue			٠
UCG-27694	CALC. NO.	100	GACOF	REE		
PIN						
BENDI	NG STRESS (4)	1				
M= P(3	a+g++ D)					
f _b = Mc/ I = πd' C = d/2						
to = 16P(3	+9+ + 1)(2) (md =)/(md =)					
= 16 P (3	°+ .0625+8875)/11 7.5	3				
= 270,00	0 (.04363)					
= 11,79	TO PSI					

(1) ADAPTED FROM

FACTENING AND JOINING,

HE Ed, A REFERENCE ISSUE OF

MACHINE DESIGN, PENTON PUBLISHERS

PAGE 27

-	The second secon				
REV.	REV.	AUTHOR	DATE CHK'D BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Vessel Head Lift Rig

PROJECT

CGE

AUTHOR

CALC NO.

CALC NO.

PAGE

7 OF 42

PAGE

7 OF 42

PROJECT

CALC NO.

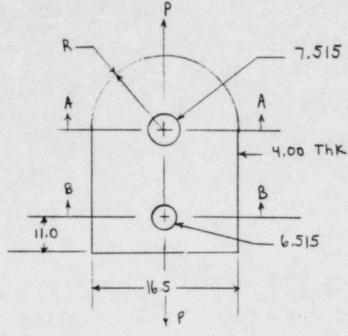
PAGE

7 OF 42

PAGE

REE

(2) SIDE PLATE - ASTM - AS88 GR. A Q \$T



Tension @ 7.515 \$ Hole

$$ft = \frac{9/2}{At}$$
 $A_{\pm} = (16.5 - 7.515)(4.0) = 35.941n^2$

$$f + 135,000 = 3756 PSI$$
 35.94

Bearing @ 6515 \$ Hole

$$f_c = \frac{P/2}{4c} = \frac{135,000}{(6.515)(4)} = 5180 PSI$$

-					
REV.	REV	AUTHOR	DATE CHK'D BY	DATE CHK'D. BY	DATE
NO.	DATE				

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

Reactor Ves	ssel Head Lift Rig		PAGE	OF	42
CGE	Fr. C. Peduni 21	83 Richa	rel2/83		DATE
UCG-27694	CALS NO. W	AUE NO.	REE		

Shear Tear - out

$$f_V = \frac{\beta_2}{2AV}$$

@ 7.515\$ Hole

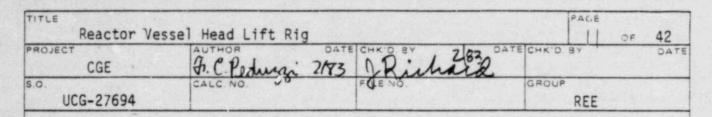
$$f_V = \frac{135,000}{2(8.25-7.515)4} = 3756 PSI$$

@ 6.515 \$ Hole

$$fv = \frac{135,000}{2(11.0 - 6.515)(4)} = 2180 PSI$$

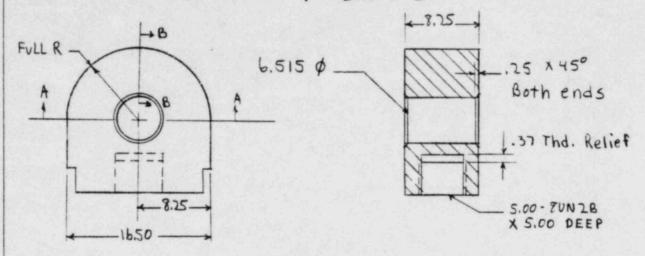
DATE CHK'D BY DATE CHK'D BY DATE
REE
SHEAR STRESS
$f_{v} = P/2A,$ $P = 270,000 \text{ lb.}$ $A_{v} = \text{Tr}(\frac{1}{2}/4)$ $A_{v} = \text{Tr}(\frac{1}{2}/4)$ $A_{v} = 33.18 \text{ in}^{2}$ $f_{v} = (270,000)/(2*33.18)$ $f_{v} = 4069 \text{ PSI}$
BEARTING STRESS $f_{c} = P/A_{v}$ $P = 270,000 \text{ lb.}$ $INNER$ $A_{v} = d J = (6.50)(7.75)$ $= (50.38) \text{ in}^{2}$ $f_{c} = (270,000)/(50.38)$ $f_{c} = 5359 \text{ pst}$ OUTER
$A_{v} = 2 \text{ ad} = 2(4.00)(6.50)$ $= (52.0)_{\text{im}^{2}}$ $f_{c} = (270,000)/(52.0)$ $f_{c} = 5192 \text{ PST}$ DATE[CHK D. BY DATE]
2) 3 8

TITLE	Reactor Vesse	Head Lift Rig		PAC	0 of 42
PROJECT	CGE	F. C. Peduy 2/8	3 & Richa	DATE CHK'D. BY	DATE
s.o.	G-27694	CALC NO. 00	FILE NO.	GROUP	E
	PIN	3			
	BENDING	STRESS (1)			
1	M= P(3 a-	· g + + D)			
	f = Mc/I I = πd4/6				
	C = d/2	٠4			
ť	= 16P(33+	g + 4 8)(2)(md3)			
		+.625+7.75) / 6.53			
	= (270,000)(.07225)			
	= 19,509	S PSI			
(1)	ADAPTED FR	om			
<u>F</u>	ACTENING A	NO JOINING,			
		NEE ISSUE OF			
	AGE 57	N , PENTON PUBLISHERS			
REV.	REV.	AUTHOR DA	TE CHK'D. BY	DATE CHK'D. BY	DATE



UPPER LOAD CELL ADAPTOR 4

Matil. ASTM A- 508, Class 2



Tension at Section A-A (Pin Hole)

$$ft=P$$
 At= (16.5-6.515)(8.25) -4(1)(.25)2= 82.251n2
At

$$fc = \frac{270,000}{82.25} = 3283 PSI$$

Bearing stress

The bearing stress is the same as the inner bearing stress on the pin 3

FC = 270,000 = 5359 PSI 50.38

REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig		PAGE 12 OF	42
CGE	J. C. Podayji ZI	83 Drihas	22 By	DATE
UCG-27694	CALC NO.	ALE NO.	REE	

Thread Shear

Dpitch = Dnom - 1649521n

= 4.91881 11.

D nom = major diameter of extended thread

n = number of thread lin

l = Length of thread engagement

= 5.00 in.

Therefore Av = 38.63 in2

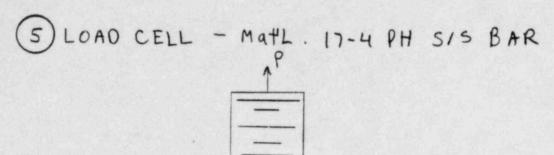
$$f_V = 270,000 = 6989 \text{ PSI}$$
38.63

Shear Tear-out Parallel to Section B. B (Pin Hole)

$$fv = \frac{P}{2AV}$$
, $AV = (8.25 - 6.515)(8.25) - 2(\frac{1}{2})(.25)^2 = 41.12 in$

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig			PAGE 13 OF	42
PROJECT	F. C. Pedusai	2/83 DRich	2 82 DATE CHK	'D. 8Y	DATE
ucg-27694	CALC. NO.	FUE NO.	GRO	REE	



Tension at A-A

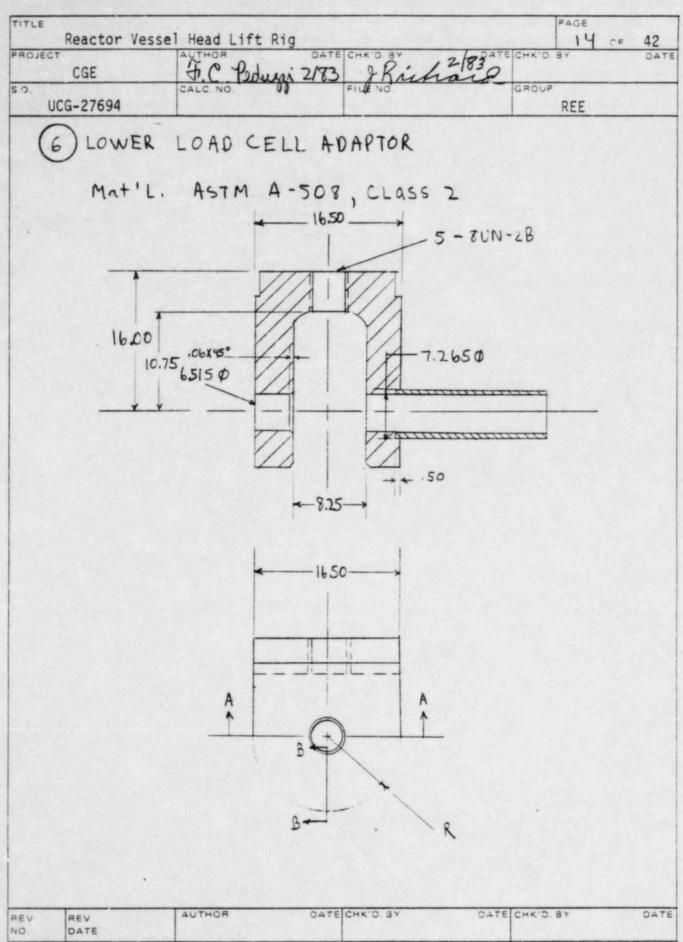
From Mark's Handbook for MES 8+n Ed, P8-13

$$ft = \frac{770,000}{18.7} = 14,439 PSI$$

Thread Shear

The thread shearing stress is the same as for the adaptors. fu = 6989 PSI

REV.	REV.	AUTHOR	OATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				



TITLE			PAGE	
Reactor Ve	ssel Head Lift Rig		15 OF	42
PROJECT	F. C. Podusi 2	73 2 Richo	DATE CHK'D. BY	DATE
ucg-27694	CALG NO.	CYLE NO.	GROUP	

Tension at Section A-A (Pintole)

ft= 1/2 , At=(16.5-6.515)(4.125)-(7.265-6.515)(.5)=40.81 in2

ft= 135,000/40.81 = 3308 PSI

Bearing stress @ Pin Hole

The bearing stress the same as the outer bearing stress on the pin o

fc = 270,000 = 6939 PSI 38.911

Thread Shear

fv = P , Av = 17 Opitch * 2/2.

Opitch = Dnom - . 64952/n

Dpitch = (5.00) - .64952/8

= 4.91881 in.

Drom = majordiameter of external thread

n = number of thread / in

THE REAL PROPERTY.					
AEV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

TITLE			PA	GE	
Reactor Ves	ssel Head Lift Rig			16 OF	42
FROJECT	F. C. Poduni 2	183 Parha	830 CHK'D BY		DATE
UCG-27694	CALC NO.	FYE NO.	GROUP	EE	

L = Length of thread engagement

= 5,25 in

Therefore Av= 40.564 in2

 $fv = \frac{270,000}{40.564} = 6656 PSI$

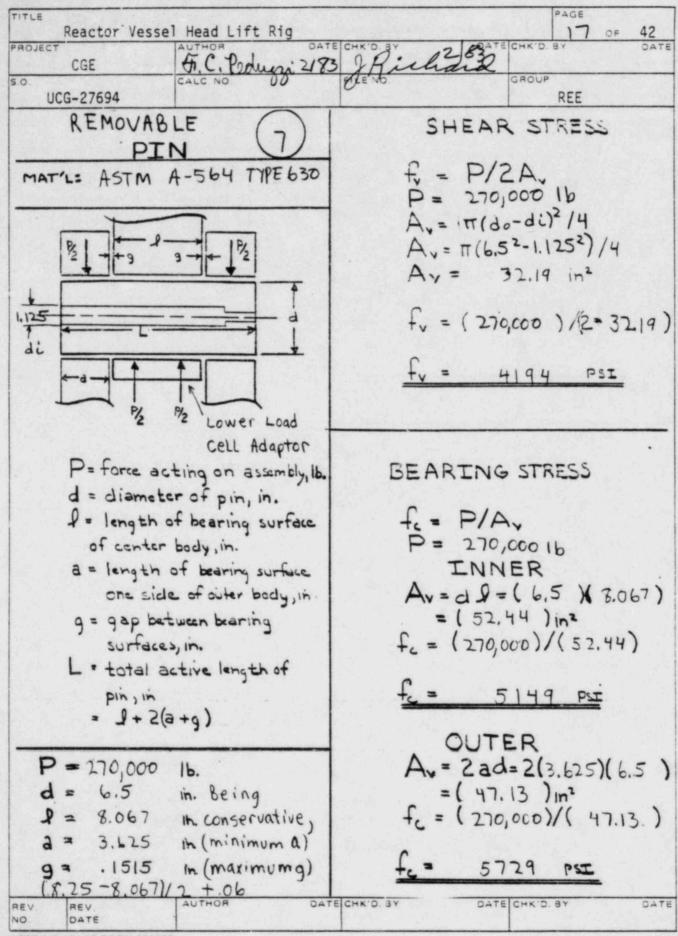
Shear Tear-out Parallel to Sec &B (Pin Hole)

fv=P, Av=(8.25-6.515)(4.125)-(7.265-6.515)(.5)

Av = 20.22 in2

 $f_V = 135,000 = 3338 PSI$ 2(20.22)

		AUTHOR	DATE CHK'D BY	DATE CHK'D. BY	DATE
REV.	REV	40111011	0		
NO.	DATE				



TITLE				PAGE	
Reactor Ves	sel Head Lift Rig			18 OF	42
PROJECT	Fr. C. Pedugi 2	83 Prih	udela CHK'D.	вч	DATE
ucg-27694	CALC. NO.	SILE VO.	GROUP	REE	

REMOVABLE PIN



BENDING STRESS (1)

Mmax= = (3a+g+40)

$$fb = Mc/I$$
 $I = \Pi d^4/64$
 $c = d/2$

ADAPTED FROM

EASTENING AND JOINING

4th Ed., A REFERENCE ISSUE OF

MACHINE DESIGN, PENTON PUBLISHERS

PAGE 27

REV	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO	DATE				

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

Reactor Vessel Head Lift Rig

PROJECT

CGE

AUTHOR

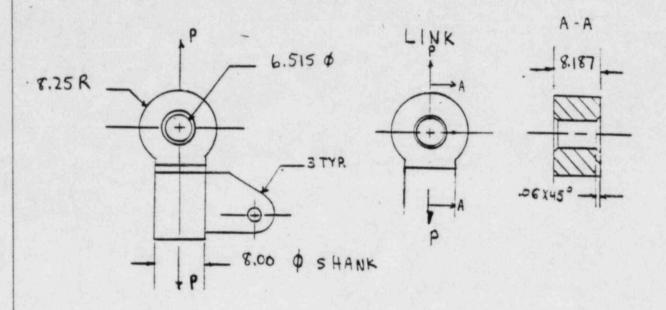
CALC NO.

DATE CHK'S SY

PAGE
19 OF 42

PAGE

(8) SLING BLOCK LINK - ASTM A508 CLASS 2



Tension at 6.515 \$ Hole

$$f = \frac{1}{4}$$
 $A = (8.25 - 3.258)(8.187)(2) - \frac{1}{2}4(.06)^2 = 81.73 \text{ in}^2$

-					
REV.	REV.	AUTHOR	DATE CHK'D, BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig			20 of	42
CGE	F.C. Pedusi 2	183 Rich	Z BOATE CHK'D.	87	DATE
ucg-27694	CALC NO.	FRENO	GROUP	REE	

shear at 6.515 \$ Hole

$$f_{V} = P$$
 $A_{U} = (8.25 - 3.258)(8.187) - \frac{1}{2}2(.06)^{2} = 40.8717$

$$fv = \frac{270,000}{2(40.87)} = 3312 PSI$$

Bearing on 6.500 p Pin.

The bearing stress 1sthe same as the inner bearing stress on the removable pin. The pin is item (7).

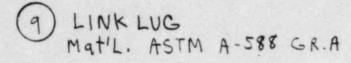
fc = 270,000/52.44 = 5149 PSI

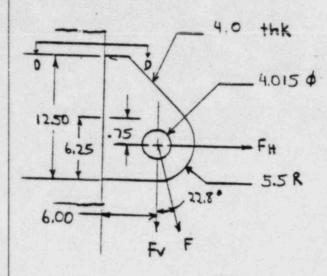
Tension at 8.00 & cylindrical section

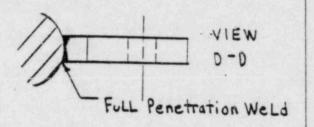
$$ft = P$$
 $A = TT (8.00)^2 = 50.27 \text{ in}^2$

$$fe = 270,000 = 5371 \text{ PSI}$$
 50.27

Reactor Ve	ssel Head Lift Rig		PAGE 2	OF	42
CGE	F. C. Peduni Z	183 & Rich	Z 830 CHK'D. BY		DATE
UCG-27694	CALC. NO. 05	ALE NO.	GROUP		







$$FV = \frac{270,000}{3} = 90,000 \text{ lbs.}$$

Tersion at 4.015 \$\phi\$ Hole

$$ft = E$$
 $At= (5.50-2.007)4 - 2½(.06)² = 13.964 in²$

$$f \in \frac{97,628}{2(13,964)} = 3496 \text{ PSI}$$

shear Tear at Hole

$$fv = E = 97.628$$
 $A = (5.50 - 2.008)4 - 2½(.06)² = 13.9641n²$
 $2Av$ $2(13.964)$

fv = 3496 PSI

-					
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig		PAGE 22 OF 42
PROJECT	F. C. Peduri 25	3 & Richard	Q DATE CHK'D BY DAT
UCG-27694	CALC. NO 00	OLE NO.	REE

Bearing at 4.015 \$ hole

The bearing stress is the same as the inner bearing stress on the upper clevis pin item 10

Combined stress from Tension and Bending

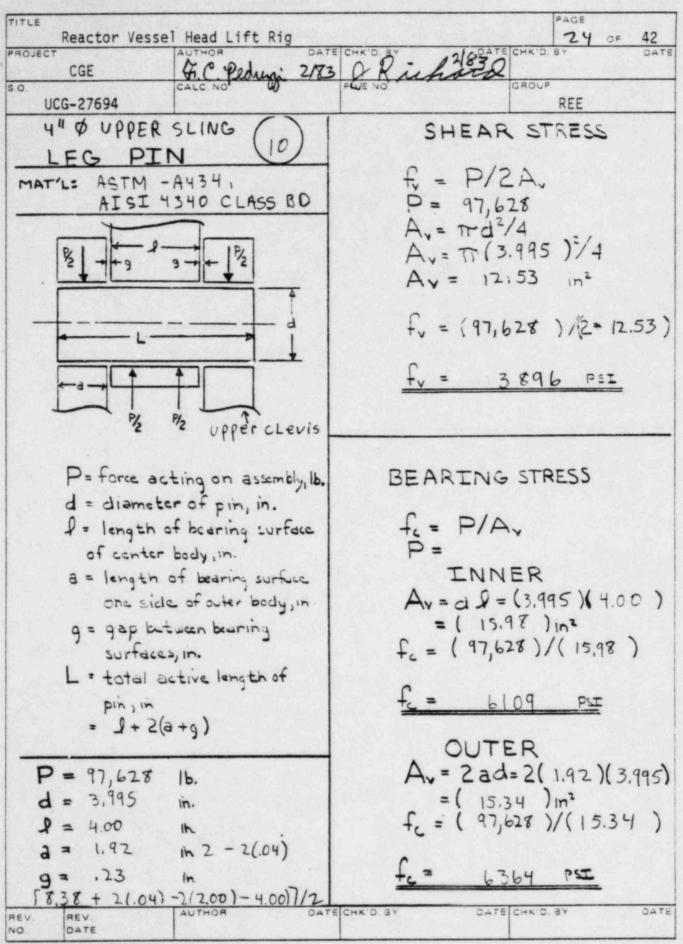
$$2 = \frac{1}{6} (4)(12.5)^2 = 104.17 \text{ in}^3$$

-					
REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

Reactor Ve	ssel Head Lift Rig		23 of	42
CGE	J. C. Pedusi 21	83 Michae	DATE CHK'D. BY	DATE
UCG-27694	cálc. No.	CLÉ NO.	GROUP REE	

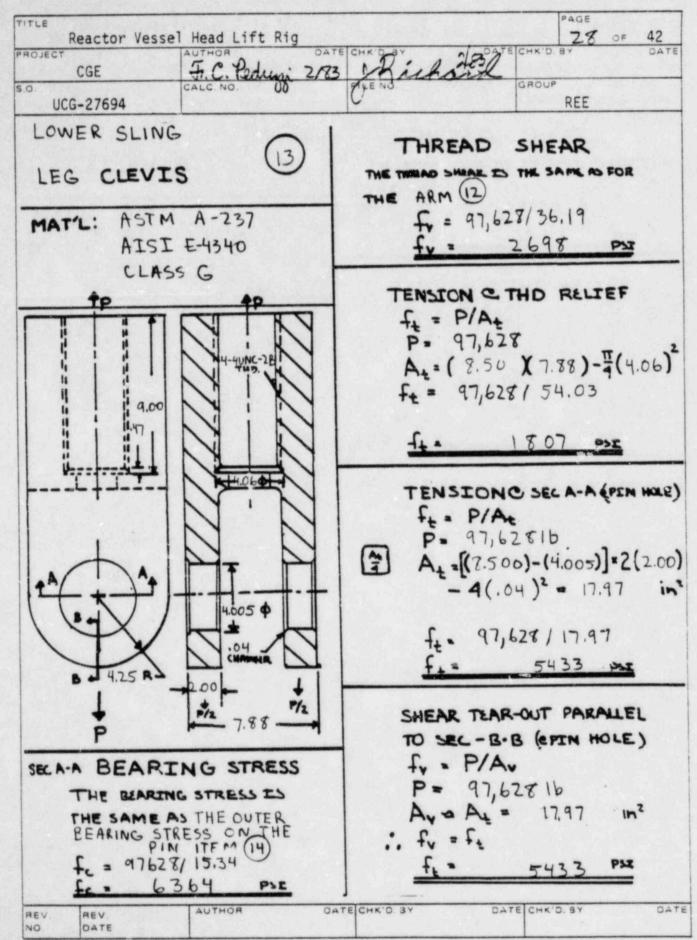
. Vertical Shear at Full Penetration Weld

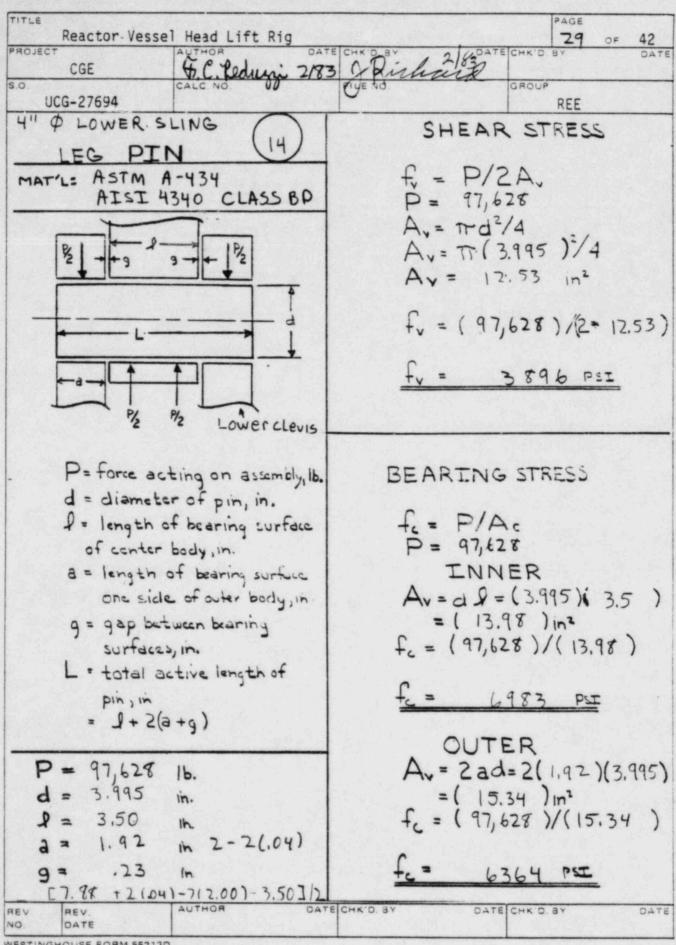


Reactor Ve	ssel Head Lift Rig		25 of		
CGE	AUTHOR	DATE CHK'D BY	GROUP REF	OF 4	
CGE	CALC NO.	1183 DE 13 ME	GROUP		
UCG-27694			REE		
PIN	(71			
	U	9			
BENDI	NG STRESS	1)			
M- P/ 1	(Q + + + p + 6				
1-1-2(3	19 4 4 7)				
fr = Mc	/I				
f = Mc I = πd	4/64				
c= d/	2				
C P/L	1 0 1/4 V	64 \			
tb = 2 (3	a+g++1)(2)(=)(=)	md*)			
=167(3	12+9+41 //(па")			
. 14	12 4) //	.)			
= 168 (3	12 + .23 + 4)/(17)	3,9957			
= 97,62	8 (.14937)				
	E 9 2 0 C T				
= 14,	583 PSI				
(1)					
(1) ADAPTED					
THE RESERVE THE PROPERTY OF THE PARTY OF THE	FERENCE ISSUE OF				
	LEGN , PENTON PUBL	TSH182			
PAGE 27					
REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY		

Reactor Vessel Head Lift Rig CGE CGE CALC NO. CALC NO.	
LEG CLEVIS	THREAD SHEAR THE THE ARM IT. 12 C = 97,628/36.19
MAT'L: ASTM A-237, AISI E-4340 CLASS G	TENSION & THO RELIEF f. = P/A+
9.00 4.00	P= 97,62816 A= (9.50 X 8.38)-4(4.06) f= 97,628158.28 f= 1675 PSE
4.005\$	TENSIONE SEC A-A (PIN HOLE ft = P/At P = 97,628 16 At = [(\$.5)-(4,005)] * 2(2.00 -4(.04)^2 = 17.97 in
4.25 R. 2.00 - 1/2 8.38 - 1/2	ft. 97,628/17.97 ft. 5433 put SHEAR TEAR-OUT PARALLEL
SECA-A BEARING STRESS THE BEARING STRESS ES THE SAME AS OUTER Bearing STRESS ON THE PIN ITEM 10 fc = 97,628/15.34 fc = 63.64 PSE	fy = P/Av P = 97,628 16 Av = Au = 17.97 In² : fv = fu ft = 5433 PM

	ssel Head Lift Rig		PAGE 27 OF 42
CGE	F.C. Peduni	2/83 ARicha	DATE CHK'O. BY DATE
o. UCG-27694	CALC. NO.	WEND.	REE
504-27034			1 REE
ARN	1 12	t. t	EAD SHEAR DIATE
ATTL: ASTA	↑P	Portal = Promo of externa h = 17 l = le	mal THREAD Dom64952/h (4.00)64952/14) =(3.8376) in. = major diameter If thread umber of threads in ingth of thread engagement 6.00 (36.169) in ² 97,628 165
	4.00-4UNK	ft = F FOR EXTERNA At = T4 (D = T4 (D ft = (97,1)	2698 BE AD TENSION P/At LLY THREADED PARTS 19743/4)*-11.083, 628)/(11.083) 7809 PSE
REV. REV.	AUTHOR	OATE CHK'D. BY	DATE CHK'D. BY DAT





Reactor Ves	CGE GALC NO. DATE CHK'O 84 CALC NO. CAL			PAGE 30	42
CGE		2/83 CA	12/830	ECHK'D. BY	DA
UCG-27694	CALC NO.	CIVE NO		REE	
000-27034					
PIN	(4)			
	,				
BENDI	NG STRESS	1)			
M= = (3	a+g+41)				
C = Mc	/ T				
I = 11 d	4/64				
c= d/2	2				
C = (}	ナ・ナ も)(章)	(6H)			
=16P(1)	a+g++1)(2)	(πd^3)			
	,				
- 169(1.92+,23+3.5)	П3.995			
= 97,1	28 (, 13938)				
= 13	,607 PSI				
(1) ADAPTED	FROM				
	S AND JOINING	2,			
4th Ed, A RE	FERENCE ISSUE OF				
	EST ON , PENTON PUZ	CISHR7			
PAGE 27	TAUTHOR	DATE CHK'D. 3	Y D	ATE CHK'D. BY	

Reactor Vessel Head Lift Rig

PROJECT

CGE

CGE

CALC. NO.

CALC. NO.

REE

PAGE

31 OF 42

DATE CHK'D. BY

DATE CHK'D. BY

DATE

CHK'D. BY

DATE

CHK'D. BY

DATE

CHK'D. BY

DATE

CHK'D. BY

DATE

CHK'D. BY

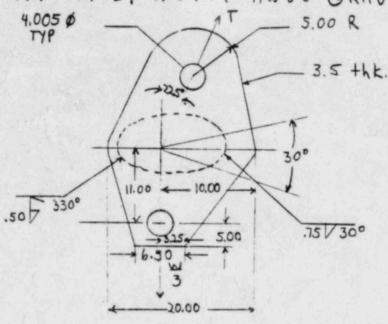
DATE

CHK'D. BY

REE

15 SPREADER LUG

Material: ASTM AS88 GRADE A STEEL PLATE



Bearing Stress

The bearing stress for the upper hole is the same as the bearing stress on the sling leg's Lower pin (4). Using the inner stress:

Fc= 97,628/13.98 = 6983 PSI

The bearing stress for the Lower hole is the same as the bearing stress on the Lifting leg's upper pin 18. Using the inner stress:

FC= 90,000/13.98 = 6438 PSI

_					
REV.	REV	AUTHOR	DATE CHK'D. 3Y	DATE CHK'D. BY	DATE
NO.	DATE				

TITLE Reactor Ves	ssel Head Lift Rig			32 of	42
PROJECT	AUTHOR CALC NO PROMISE 2	183 O Richa	BOATE CHK'D.	3 Y	DATE
UCG-27694	CALC. NO. 00	FOE NO.	GAOOF	REE	

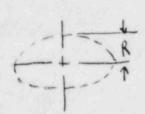
rension at upper Hole

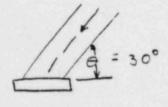
$$f_{t} = 1$$
 Ata (5.00(2) - 4.005) 3.5 = 20.983 in 3

shear Tearout at upper Hole

SPREADER ARM WELD

Be generously consertative & assume that the weld is all 175 V and that the lug and arm to not contact so that all the force treated as acting through the weld. All stress in fillets are traditionally treated as shear across the throat area.





R = outside Radius of pipe = (IC+2t)/2 where

t = wall thickness

				DATE CHK'D. BY	DATE
		AUTHOR	DATE CHK'D. 3Y	UMICICAN D. S.	
REV.	AEV.				
NO.	DATE				

WESTINGHOUSE NUCLEAR TECHNOLOGY DIVISION

Reactor Ve	33 of 42		
CGE	F.C. Peduni 2/8	3 Richa	
ucg-27694	CALC. NO. OF	FIJE NO.	REE
*8 = 17 6	25 + 2(.500))/2=	4 312 = 12	
		1.3125 In.	
l = per	imeter length		
≃ Rπ	r(1/sing+1) =	40.644 in	
fv=P	, Av = .707	wl where	w= weld

P = 21,843 lbs (from spreader arm calculation)

Leg width = ,707 (.75)(40.644) = 21.55 in2

fv= 21,843 = 1014 PSI

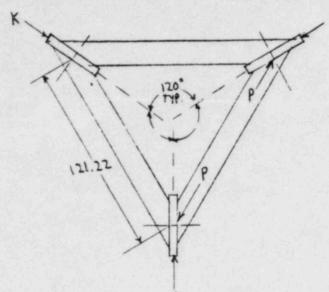
AV

* From Mark's Handbook 8th ed, P8-160 For 8.00 SCH 80 pipe t = .50, ID=7.625

-					
REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig		PAGE 34 OF	42
PROJECT	F. C. Peduzi 21	83 Price	GROUP	DATE
UCG-27694	CALC. NO. 08	FUE NO	GROUP	

16) S PREADER ARM - ASTM A 106 GRADE B SEAMLESS



K = Fv tan 22.8°

= 90,000 tan 22.8°

= 37,833 lbs

2 P cos 30° = K = 37,733 165.

P = 37,733 = 21,843 lbs

NOMINAL COMPRESSIVE STRESS IN SPREADER ARM FOR 8.00 SCH. 80 PIPE (Mark's Handbook, 7th Ed., P7-158)

At = 12.76 in = cross sectional area I = 105.7 in 4 = moment of Inertia S = 24.52 in 3 = Section Modulus $\Gamma = 2.878 \text{ in } 3 = \text{radius of gyration}$

ft = P = 21,843 = 1712 PSIAt 12.76

_					0.70
REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Head Lift Rig		PAGE 35	of 42
PROJECT	F.C. Eduni:	283 Varha	C3 OATE CHK'D. BY	DAT
ucg-27694	CALC. NO.	BUE NO.	REE	

ALLowable Compressive stress (AISC Manual) 7th Ed. Ps-138

K= 165 == Recommended for Fixed - Fixed Ends

$$C_c = \sqrt{\frac{2\pi^2 E}{F_y}} = \sqrt{\frac{2\pi^2 29.000,000}{35,000}}$$

= 127,888

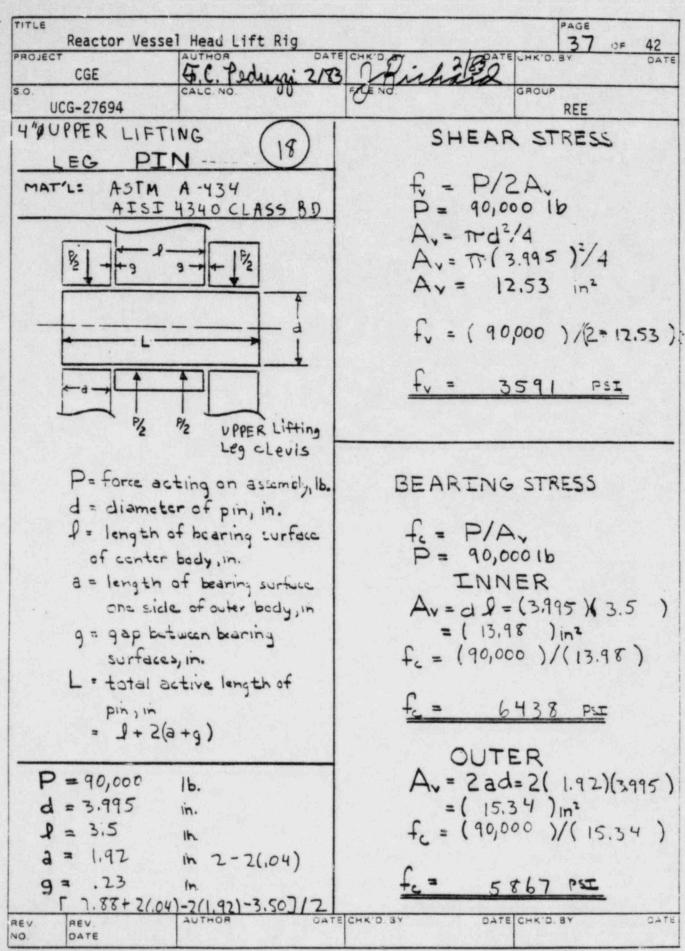
case: kl LCc ..

Fa = allowable compressive stress in the abscènce of bending moments

then
$$F_q = \left(1 - \frac{A^2}{2}\right) F_y / \left(\frac{5}{3} + \frac{3}{8}A - \frac{1}{8}A^3\right)$$

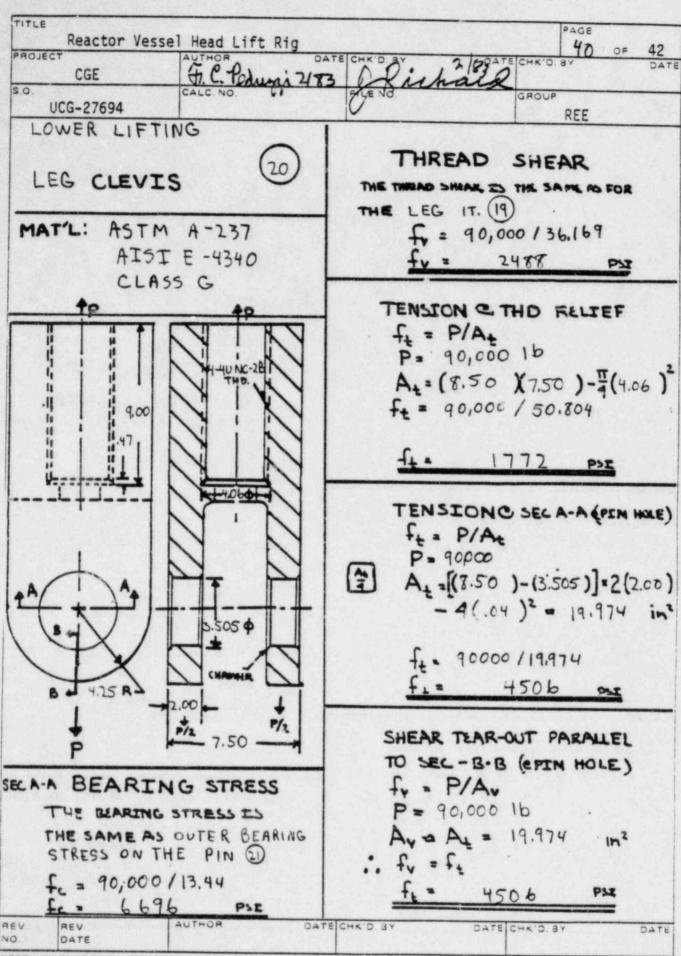
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'C. BY	DATE
NO.	DATE				

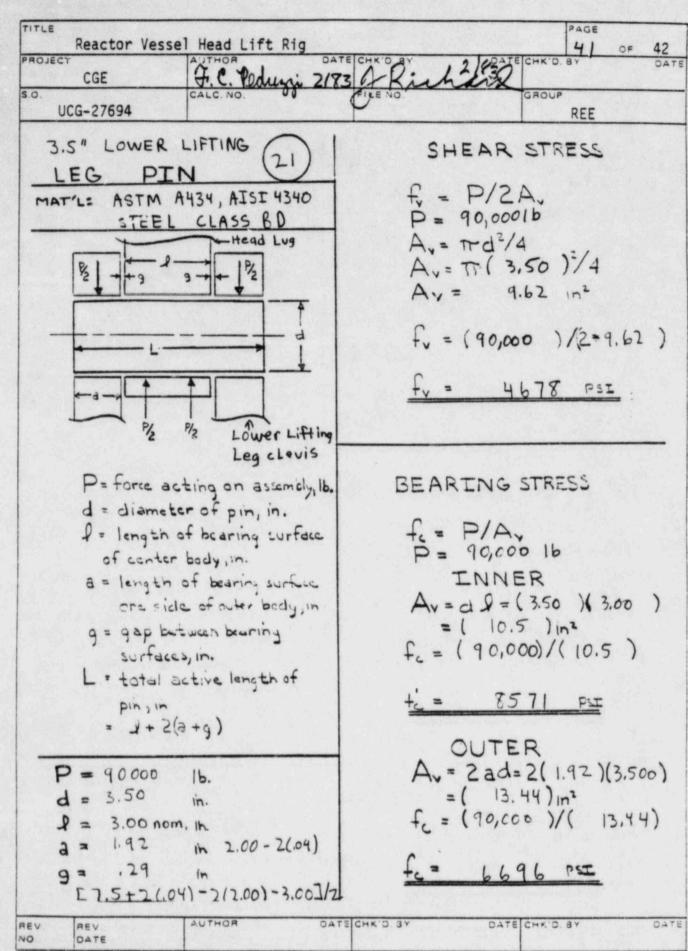
Reactor Vessel	Head Lift Rig		36 of 42
CGE	GC. Poduni 2/	83 Prehai	TEICHK'D. BY OA
ucg-27694	CALC. NO.	PLE NO.	GROUP
UPPER LIFTING		I	REE
LEG CLEVIS	(17)	THREAD THE THEAD SHEAR E	
MAT'L: ASTM AISI E CLASS	-4340	f = 9000	2488 PM
4.00	14-410-100-20 14-410-20 14-410-20	f _t = P/A _t P= 90,00 A _t = (8.50 f _t = 90,00	0 16 (4.06)
4.25 8	4.005\$	F. 90,00 A. 1(7.5 -4(.0)	
	G STRESS TRESS ES THE OUTER SS ON THE	fv = P/Av P = 90,00 Av = Au = : fv = fu	0 16



Reactor Vess	sel Head Lift Rig		PAGE 20 A
CGE	AUTHOR DATE	ARichard	38 OF 4
JCG-27694	CALC. NO.	FUE NO.	REE
UPPER LIF	TING		
BENDI	NG STRESS (1)		
M= P(3	a+g++ 1)		
fb = = (3 a	2+9+4 1)(2)(md2)		
=16 P(15	13 + .13 + 34)/n 3.9953		
= 90,0	00 (.13938)		
= 12,	544 PSI		
4th Ed, A Res	erence Issue of		
	CGE ICG-27694 UPPER LIF LEG PIN BENDI BENDI ADAPTED FATENING 1) ADAPTED 2) ADAPTED 1) ADAPTED 2) ADAPTED 2) ADAPTED 2) ADAPTED 2) ADAPTED 3) ADAPTED 4) AD	CGE F. C. Pedupi 2/73 UCG-27694 UPPER LIFTING EC. Pedupi 2/73	CGE AUTHOR ALC. NO. AL

Reactor V	essel Head Lift Rig		39 OF 42
CGE CGE		3 Filase	OUP OATS
UCG-27694		10	REE
LEG	19	THREAD SHI f. = P/A. A. = Dienter	
MAT'L: ASTN	1. A-306 GRADE 70	FOR AN EXTERNAL THRE Pritch = Dnom Pritch = (4)- = (3,837) Dnom = major of external threa	64952/h :64952/14) b)in.
	↑P	$h = number of length of the = 6.00$ THEREFORE Ay = (36.16) $P = 90,000$ $f_{V} = 248$	and engagement in 1 b
	38	THREAD TER ft = D/At for externally threat At = T/4 (Drom 97 = T/4 (4 974 ft = (90,000)/1 ft = 8121	43/h)2 13/4)°=11.083
REV REV	TYP Y,00-4UNC-ZA	TE CHK'D. BY DATE CH	K'O. BY





Reactor Ve	ssel Head Lift Rig		PAGE 47 OF	12
CGE	F. C. Peduni 2	ATE CHK'D. BY	2 DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	FILENO!	GROUP	

PIN



BENDING STRESS (2)

(1) ADAPTED FROM

FASTENING AND JOINING,

45 Ed, A REFERENCE ISSUE OF

MACHINE DESIGN, PENTON PUBLISHERS

PAGE 27

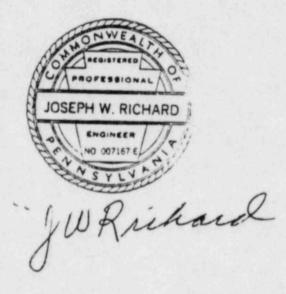
REV.	REV	AUTHOR	DATE CHK'D. 3Y	DATE CHK'D. BY	DATE
	DATE				

APPENDIX B DETAILED STRESS ANALYSIS REACTOR VESSEL INTERNALS LIFT RIG

This appendix provides the detailed stress analysis for the Virgil C. Summer reactor vessel internals lift rig in accordance with the requirements of ANSI N14.6. Acceptance criteria used in evaluating the calculated stresses are based on the material properties given in Section 4.

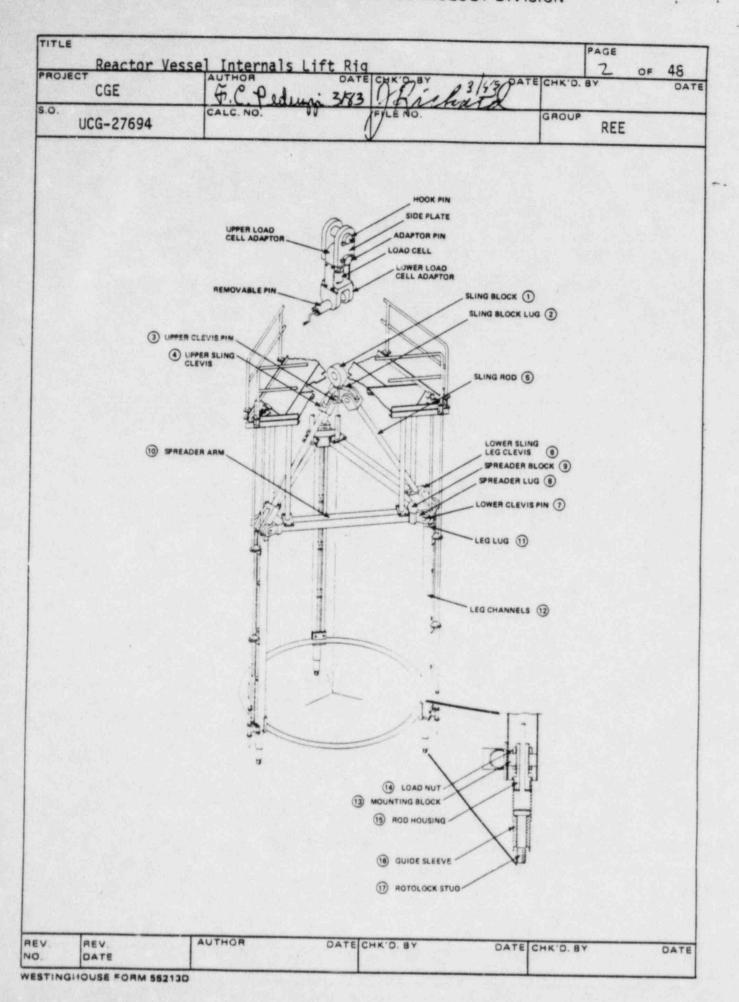
\$.0.	UCG-27694	Virgil C.	Summer	T OF 48
TITLE	R.V. Internals Lift Rig Asse	mbly	ly PDC -	
AUTHOR	F. Peduzzi G.C. Pedusais	3183 J. Rich		43/83
	E AND RESULTS:		1	

- 1. The purpose of this analysis is to determine the acceptability of this rig to the requirements of ANSI N14.6.
- 2. The results show that all stresses are within the allowable stresses.



Original Issue F. Peduzzi
REVISION NO. DATE DESCRIPTION BY

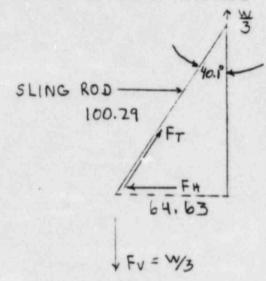
RESULTING REPORTS, LETTERS OR MEMORANDA:



Reactor Ve	ssel Internals Lift	Rig	PAGE 3	OF	48
CGE	AUTHOR	183 A Richard	DATE CHK'D. BY		DATE
ucg-27694	CALC. NO.	FULE NO.	GROUP	Ε	

DESIGN Weight

SLING LEG ANGLE



$$FV = 230,000 = 76,667$$
 lbs.

sin 0 = .6444 , 0 = 40.1°

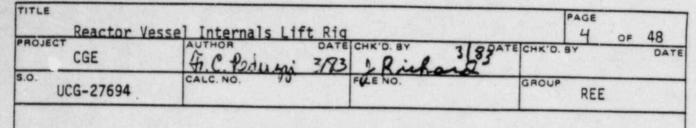
FT = TENSION IN SLING ROD

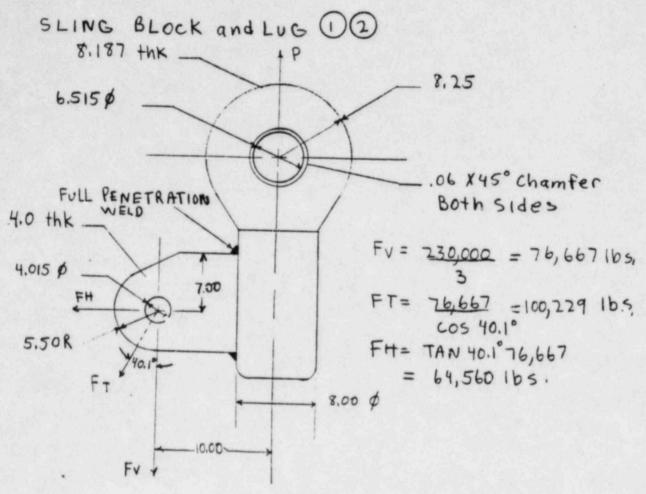
FH = FORCE EXERTED OUTWARD SPREADER BLOCK

FV = FORCE EXERTED ON EACH LEG ASSEMBLY

* ITEMS 13 THROUGH 17 use w = 202,000 lbs

-					
REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				





SLING BLOCK - Matil. ASTM A 508 CLASS 2

Tension @ 6.515 \$ Hole

$$[(8.25)(2) - 6.515] 8.187 - 4(2)(.06)^2 = At$$

At = 81.740 in2

$$ft = \frac{P}{At} = \frac{230,000}{81.740} = 2814 PSI$$

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				UATE
-					Let

TITLE			PAGE	
Reactor Ve	ssel Internals Lift		5 of	48
CGE	F.C. Podusi 3	183 1 Richard	DATE CHK'D. SY	DATE
ucg-27694	CALC. NO.	FKE NO.	REE	

Bearing @ 6.515 p Hole

$$fc = \frac{230,000}{52.56} = 4376 PSI$$

Shear Tear-out @ 6.515 Hole

$$fv = P$$
, $Av = (8.25 - 6.515)(8.187) - 26)(.06)^2 = 40.8717$

$$FV = \frac{230,000}{2(40.87)} = 2814 PSI$$

Tension @ 8.00 & CYLINDRICAL SECTION

$$ft = \frac{\rho}{At}$$
, $At = \frac{\pi d^2}{4} = \frac{\pi (8^2)}{4} = 50.27 \text{ in}^2$

Reactor Ve	ssel Internals Lift	Rig	PAGE	48
CGE	F. C. Poduni	583 & Richard	DATE CHK'D BY	DATE
UCG-27694	CALC. NO.	FILE NO.	GROUP REE	

SLING BLOCK LUG @ MAT'L. ASTM A 588 GRADE A STEEL

Tension @ 4.015 \$ Hole , P = FT

$$ft = P$$
 $At = 2(5.5 - 4.015) 4 = 27.94 in 2$

$$ft = 100, 229 = 3587 PSI$$

27.94

Bearing @ 4.015 \$ Hole , P=Fr

The bearing stress is equal to the inner bearing stress on the upper clevis pin 3

$$fc = \frac{100,229}{16.00} = 6264 PSI$$

Shear Tear-out @ 4.015 \$ Hole, P=FT

$$fv = \frac{p}{2AV}$$
 $Av = \left(5.50 - 4.015\right)(4.00) = 13.97$

$$fv = 100,229 = 3587 PSI$$

2(13.97)

Reactor Ve	ssel Internals Lift	Rig	PAGE 7 OF	48
CGE	F. C. Peduri 3	183 FRICAS	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO. 00	EVE NO.	GROUP	

Vertical Shear @ LUG ROOT WELD , P = FV

$$fv = 76,667 = 1533 PSI$$

Combined Tension from bending and tension @ Luc weld

$$I = bh^3 = (4)(12.5)^3 = 651.04 in^4$$

$$C = (5.50 \pm 7.00) = 6.25 \text{ in}.$$

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift I	Rig		PAGE OF	48
CGE	F. C. Pedung: 3/	83 00:03/8	DATE CHK'D.		DATE
UCG-27694	CALC. NO.	FLE NO.	GROUP	REE	

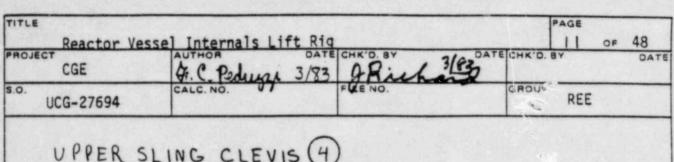
$$ft = \frac{64,560}{50} + \frac{(411,582)(6.25)}{651.04}$$

ft = 1293 + 3951 = 5244 PS	Ft=	1293	+	3951	=	5244	PS'
----------------------------	-----	------	---	------	---	------	-----

REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. EY	DATE
NO.	PATE				

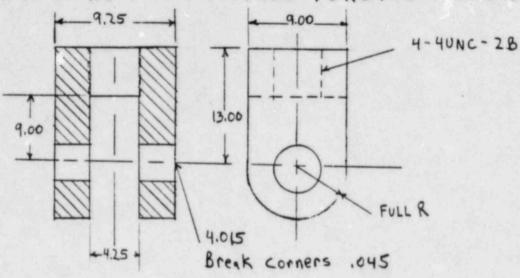
Reactor Vesse	Internals Lift Rig		9 of 48
CGE	G.C. Poden; 3/93	2 R : 2 3 83 0 ATE	CHK'D. BY DAT
UCG-27694	CALC. NO.	FUE NO.	REE
UPPER CLEV	1311	SHEAR	STRESS
MAT'L: ASTM	The same of the sa	0	4 4:00) ² /4
d = diameter l = length of of center b a = length or one side g = gap beto surfaces; L = tatal act pin, in = 1+2(a. P = 100,229 d = 4.00 a = 2.41 g = .170	bearing surface lody, in. f bearing surface of outer body, in learning line. tive length of	$f_c = (100, 27)$ $f_c = b2$ OUTE $A_v = 2ad = (11)$	29 .5 =R (4.00 × 4.00) co)in ² eq)/(16.00) 64 PST .R .2(2.41)(4.00) 28)in ² 29)/(19,28)

Reactor V	essel Internals Lift	Ria	PAGE
CGE	essel Internals Lift		3/830 DATE CHK'D. BY DATE
UCG-27694	CALC. NO. 18	QUENO.	GROUP REE
PIN	(3	01	
BEND	ING STRESS	,	
	古 a+g++1)		
f = Mc Ι = π			
C = d			
+b = \(\frac{1}{2}\)	る+g+ もり)(全)(元)(元)(元)(元)	(da)	
= 16 P	(2.4) +. 170 + 4)/T	13	
= 100,7	129(.157031		
= 15	,739 PSI .		
(1) ADAPTED	S FROM		
4th Ed, A Re	eference Issue of		
PAGE 27	EST ON , PANTON PUBLISH	HERS	
REV.	AUTHOR	ATE CHK'D. BY	DATE CHK'D. BY DATE



UPPER SLING CLEVIS (4)

MATIL: ASTM A471 STEEL FORGING CLASS 3



Tension @ 4.015 \$ Hole

$$ft = P$$
 $Av = (9.25 - 4.25)(9.00 - 4.015) - 4(.045)^2$
 $Av = 24.92 \text{ in}^2$

$$ft = 100,229 = 4022 PSI$$
 24.92

Bearing stress @ 4.015 \$ Hole

The Bearing stress @ the 4.015 & Hole is the same as the outer bearing stress on the PIN (3)

fc=100,229/19.28 = 5199 PSI

THE RESERVE THE PERSON NAMED IN	THE RESERVE OF THE PARTY OF THE	THE RESERVE OF THE PARTY OF THE			And the second second
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rig		PAGE	05 48
CGE	F. C. Peduni	OATE CHK'D. BY	3/83 DATE CHK'D.	84	DATE
UCG-27694	CALC. NO. UU	PLE NO.	GROUP	REE	

Shear Tear-out @ 4.015 \$ Hole

$$f_{V} = P$$
, $A_{V} = \left(\frac{9.00 - 4.015}{2}\right)(9.25 - 4.25) - 2(.045)^{2}$
 $A_{V} = 12.46 \text{ In}^{2}$

$$fV = \frac{100,229}{2(12.46)} = 4022 PSI$$

Thread Shear

The thread shear is the same as the sling rod's thread shear 5

$$FV = 100,229 = 4157 PSI$$
24.112

CGE	Ge Podumi 3	183 & Rich	OATE CHK'D. BY	DA
UCG-27694	CALC. NO.	FILE NO.	GROUP RE	Ε
SLING ROD	5	t.	PAY DINT 1/2	
ATL: ASTM GRADE	A-306 70 or 80	Porto	TRIAL THREAD = Dhom 64952 = (4.0) 64952 = (3.8376) in. m = major diam nal thread	14)
	↑P	I = I	rumber of the length of thread eng 4.00 = (24.112) in ² 100,229165	agement
	SL3 Thid. Relie (Both ENDS)	ft = FOR EXTER At = 174 (DATE DATE PARENTE PARE)2)- 11.083
	TP 4 -40NC.	ft = P $At = T$ $At = C$ $ft = C$	on & thread)2/4

WESTINGHOUSE FORM 55213D

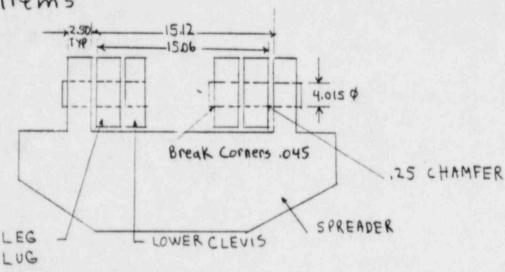
Reactor Ve	ssel Internals Lift	Ria	P	AGE	40
CGE	AUTHOR	183 Richar	OATE CHK'D. BY	17 01	DATE
UCG-27694	CALC. NO.	FILE NO.	GROUP	REE	

SPREADER JOINT

The Spreader Joint Consists of the

- 1) Lower clevis
- 2) Lower clevis pin
- s) Spreader Block Lug
- 4) The Leg Lug

The bearing stresses acting between these Items are calculated on the following pages. The resulting moments, forces and stress distributions are then used as inputs to determine the listed Items stresses in the following calculations on these items

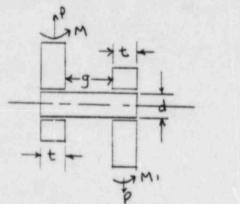


REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				0416

Reactor Ve	ssel Internals Lift	Rig	PAGE 15	of 48
CGE	AUTHOR	13 Richs	DATE CHK'O. BY	DATE
ucg-27694	CALC. NO.	FOLE NO.	GROUP REE	LE LE

The distribution of the bearing stress between Lug and pin is assumed to be similiar to the stress distribution that would be obtained in a rectangular cross section of width d and depth t subjected to a load P and Moment M, as assumed in section 9.5.1 "Lug bearing strength for single shear joints under uniform axial Load", from Technology Inc, Report T1-219-69-24, Dayton, Ohio; wright-Patterson AFB sponsered.

ALL Lugs are of similiar thickness, so the model becomes



t = thickness of Lug

carries only vertical forces and the spreader carries only horizontal forces. The clevis has both horizontal tvertical crimponents acting on it, so that in any one plane only two lugs are interacting.

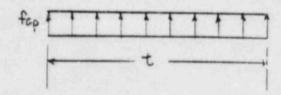
-					
REV.	DATE	AUTHOR	DATE CHK'D. BY	DATE CHK'D, BY	DATE

Reactor Ve	ssel Internals Lift	Rig	PAGE	42
CGE		173 1 Rich	3/830 CHK'D. BY	DATE
UCG-27694	CALC. NO.	FIGE NO.	GROUP REE	

SPREADER JOINT

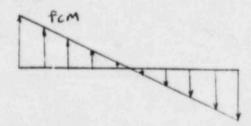
The clevis has both horizontal and vertical components Acting on it, so the stresses will be superimposed for the clevis.

The contact stress due to the force Pwould be fcp = Pdt

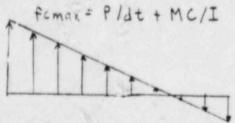


The contact stress due to the moment M would be

fcm = MC/I



when these stresses are combined, the result is



No. of Street, or other					
HEV.	REV.	NUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				
-					

Reactor V	essel Internals	Lift Rig	PAGE 17 OF 48
CGE		1 3/83 Pich	2 DATE CHK'D. BY
UCG-27694	CALC. NO.	II TLE NO.	GROUP REE
		- ^	
There M tota Dividir M = (1	entire mo	t moment between	the soint will be sen the two ends.

In the horizontal plane

M spreader = (16, 1401(5.12)

Mspreader = 82,637 in -16.

REV.	REV.	AUTHOR	OATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rig	PAGE	- 48
CGE	F.C. Peduni 3	83 1 Killy	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO. UO	OLE'NO.	GROUP REE	

SPREADER JOINT .

The combined effects of the horizontal (spreader) and vertical (leg) moments acting on the sLing leg lug are obtained using the method of Sec 10-11, Page 336, of E.P. Popovis MECHANICS OF MATERIALS, 2 ND Edition.

The combined bearing stresses are firmax = firmtfcp

$$P = 100,229/2 = 50,115.65 d = d pin = 4.00 in.$$

 $E = 2.50 - 2(.045) = 2.41 in.$

-					
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Riq	PAGE 19 OF	48
CGE	AUTHOR	183 Rich	BATE CHK'O. BY	DATE
UCG-27694	CALC. NO.	FUE NO.	GROUP REE	

FOR the spreader 3

P = 64,560/2 = 32,28016. A = dpin = 4.00in.t = 2.50in.

fcmax = $\frac{32,280}{4 \times 2.5} + \frac{6(82,637)}{(4.00)(2.5)^2}$

fcmax = 3,228 + 19833 = 23,061 PSI

For the Leg 3

 $P = W = \frac{230,000}{6} = 38,333165$

d = 4.00 in

t = 2.50 -. 25 = 2.25 in.

fcmax = 38,333 + 6 (49,066) (4)(2,25) 4 (2,25)2

fimax = 4259 + 14,538

fcmqx = 18,797 PSI

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Ria	PAGE 20 00	10
CGE	F.C. Pedusai 3	173 & Rich 3	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	MLE NO.	GROUP REE	

Check of extreme fiber stresses to AISC CRITERIA Bearing

the bearing stress calculations for the lower clevis, item b, the Lower clevis pin item b, the spreader lug, item B, and the leg lug item D, are done to determine the stress field, the only criteria being that they be below yield.

Traditionally bearing of pin joints is done ignoring such local concentrations and so the appropriate calculations for which reference allowables are obtainable is fc = P/At; The AISC allowable is fc = 0.90 Fy where Fy = They ield strength of material

Lower clevis and lower clevis pin Bearing

$$fc = P/Ac = FT/2/AC$$
 $Ac = dt = 4.00(2.50 - 2(.045))$
 $= 9.64 \text{ in}^2$
 $ft = 100,229 \text{ lbs/2} = 50,115 \text{ lbs}$
 $fc = 50,115 = 5199 \text{ PSI}$
 9.64

SPREADER LUG Bearing
 $fc = P/Ac$, $Ac = dt = 4(2.5) = 10 \text{ in}^2$
 $fc = (64,560/2) = 3228 \text{ PSI}$

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				0016
-					

Reactor Ve	ssel Internals Lift	Rig	PAGE 71 or	48
CGE	F. C. Pedusci 3	173 1 Rich	SOATE CHK'O. BY	DATE
UCG-27694	CALC. NO. UD	FIJE NO.	GROUP REE	

LEG LUG Bearing

$$fc = W/L = 38,333 = 4259 PSI$$
AC 9

CLevis .9(105,000) = 94,500 2 5199

Lower clevis Pin .9(95,000) = \$5,500 2 5199

Spreader Lug .9(38,000)= 34,200 2 3228

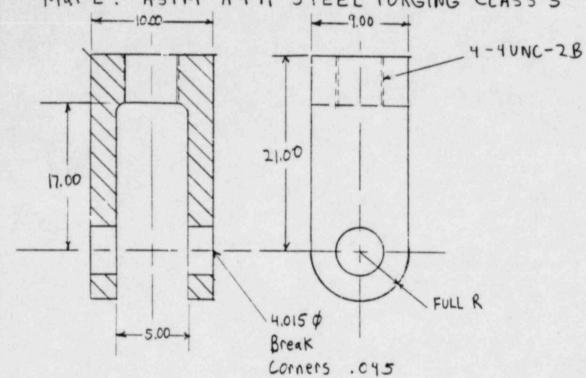
Leg Lug .9(38,000) = 34,200 2 4259

". All of the above items satisfy the AISC Bearing Criteria and are acceptable.

However, in order to be conservative, the higher numbers generated in the first set of calculations will be used to determine the stresses for items 6,7,8 and 11.

REV.	REV	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

CGE AUTHOR DATE CHK'D. BY REE REE	Reacto	r Vessel Inte	ernals Lift	Rig		PAGE 22	OF	48
UCG-27694 CALC. NO. UT FILENO. GROUP REE	PROJECT	AUTHO	A	DATE CHK'D. BY	A 3 83 Q CHK'D.	BY		DATI
100000	UCG-2769	CALC.	vo. 00	LENO.	GROUP	REE		
LOWER SLING LEG CLEVIS 6	LOWE	R SLING	LEG CL	EVIS 6				



Tension @ 4.015 \$ Hole

$$ft=P$$
, $Av = (10.00 - 5.00)(9.00 - 4.015) - 4(.045)^2$
 $AV = 24.92 \text{ In}^2$

$$ft = 100,229 = 4022 PSI$$
 24.92

From Spreader Joint calculation M= 95072 in-16.

STATE OF THE PARTY NAMED IN					
REV.	DATE	AUTHOR	DATE CHK'O. BY	DATE CHK'D. BY	DATE
			Company of the Compan		

Posetor Ve	ssel Internals Lift	Ria	PAGE	19
PROJECT	F. C. Poduni 3	83 Prich	CONTECHKO. BY	DATE
UCG-27694	CALC. NO. UU	FIVE NO.	GROUP REE	

Bending stress @ Section A-A (PIN Hole)

$$I = \frac{6h^3}{12} = \frac{(9-4.015)(2.5)^3}{12} = 6.4909 \text{ in } 4$$

$$fb = (96,072 \times 1.205) = 17,835 \text{ PSI}$$
6.4909

Combined max. bending & tension @ 4.015 & Hole

fmax = fb+ft = 17,835 + 4,022 = 21,857 PSI

Bearing stress From the Spreader Joint calculation:

fc max = 5,199 + 24,812 = 30,011 PSI

Thread shear

The thread shear is the same as the sting rod's thread shear 5

FU- 100,229 24.112

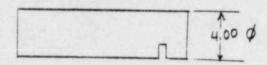
fv = 4157 PSI

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
REV.	DATE				

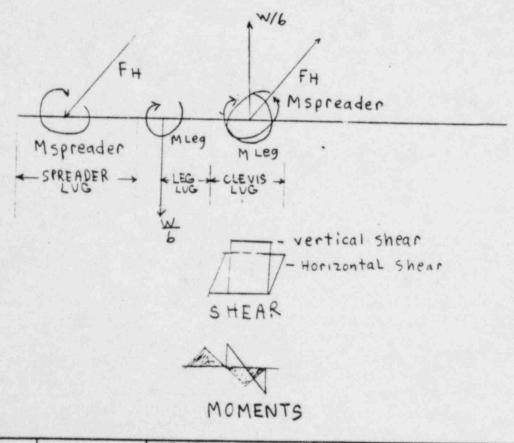
Reactor Ve	ssel Internals Lift	Ria	PAGE 7 4 OF	48
CGE	F.C. Poduzi 3	183 Olich	O TE CHK'O. BY	DATE
ucg-27694	CALC. NO.	FIGE NO!	GROUP REE	

LOWER CLEVIS PIN 7

MatlL: ASTM A 564 TYPE 630 Precipitation Hardening stainless steel AGE TREATED AT 1150°F For 4 hours, aircooled 135,000 PSI MINTENSILE Strength RC 28-31



Forces and moments acting on Pin



Reactor Ve	ssel Internals Lift	Rig	PAGE 25 OF	48
CGE	F.C. Peduni 3	183 a. Richa	1930ATE CHK'O. BY	DATE
ucg-27694	CALC. NO. U	FILE NO.	GROUP REE	

The maximum shear occurs @ the clevis

The maximum Bearing stress occurs at the clevis and is calculated in the spreader Joint analysis

fcmax = 5199 + 24,812

fcmax = 30,011 PSI

Bending stress From the spreader-joint analysis, the maximum Moment is Mclevis = 96,072 in-16.

$$fb = Mc$$
, $c = d = 4.00 = 2.00 \text{ in}$

$$I = \frac{\Pi dY}{64} = \frac{\Pi (4.00)^4}{64} = 12.57 \text{ in}^4$$

$$fb = 96,072(2.00)$$

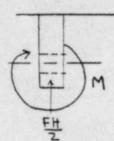
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactur Vessel Internals Lift Rig OATE CHK 0.87 DIECT CGE JUTHOR 3/73 OATE CHK 0.87 UCG-27694 CALC. NO. 973 OATE CHK 0.87 REE SPREADER ASSEMBLY ROO X 8.00 X .375 SQUARE WALL TUBE FULL R Points of Intersect 14.63 TYP of Leg 4	Vessel Inter	Reactur Ve
SPREADER ASSEMBLY 8.00 x 8.00 X .375 SQUARE WALL TUBE FULL R Points of Inversect 19.63 Typ of Leg 4	J.C. C	CGE
8.00 x 8.00 TYP. 7.5 10.0 FULL R Points of leg 9	CALC NO	UCG-27694
Points of leg 9	IDER ASS	SPREADE
Points of leg 9	1//	\
Points of leg 9	74	
Points of Iniersect		
FULL R Points of Intersection of Leg 4	Dr.	WALL TOOL
FULL R Points of Inversect		
FULL R Points of Inversect TH.63 TYP OF Leg 9	7	
FULL R Points of Inversect 10.0 TH.63 TYP OF Leg 9	7.5	
FULL R Points of Initersect 10.0 TH.63 TYP OF Leg 9		
10.0 THIEFSECT	150	
10.0 THIEFSECT		
10.0 + OF 1.eq 9		T
	$-\Theta$	10.0
		¥
REV. AUTHOR DATE CHK'D. BY DATE CHK'D. BY	AUTHOR	esv

TITLE				PAGE	
Reactor Ve	ssel Internals Lift	Rig		127	of 48
CGE	F. C. Bolumi 3	183 NRichal	83 DATE CHK	'D. 8Y	DATE
ucg-27694	CALC. NO.	FILE NO.	GRO	REE	

SPREADER ARM - MATL .: ASTM A - 36 STEEL BLOCK - ASTM A 350 LFI FORGING STEEL (NO CUN TEST REQ'D)

SPREADER LUG & MATIL: ASTM A-516 GRADE 70 STEEL PLATE, NORMALIZED, OR ALB7, GR. B, Q & T FH= 64,560165(PAGES), M= 96,072 in-165 (PAGE 16)



BEARING'STRESS
The bearing stress is calculated
in the spreader Joint calculation

fcmax= 3228 + 19,833 = 23,0617 >I

Tension @ 4.015 \$ Hole (PIN HOLE)

$$ft=P=EH/2$$
, $At=(10.00-4.015)(2.50)=14.9631n^2$
 At At

$$f = 64,560/2 = 2,157PSI$$
 14.963

BENDING @ 4.015 \$ Hole

$$fb max = MC = 32,637(2.5/2)$$

I

$$I = bh^3 = (10 00 - 4.015) (7.5)^3 = 7.79 \text{ in}^4$$
12

Reactor Ve	ssel Internals Lift	Rig	PAGE	OF 48
CGE	4.C. Peduzi	DATE CHKO BY	33 DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	EVLE NO.	GROUP REE	

$$f_{b_{mqq}} = (82,637)(2.5/2) = 13,260 PSI$$

Combined Tension and Bending

MAT'L. ASTM A 350 LFI FORGING STEEL (NO CVN TEST REQ'D)

BEARING ON SPREADER BLOCK

The bearing on the spreader block is equal to the compressive stress in the spreader arm

$$P = \frac{FH}{2} = \frac{64,560/2}{.8660} = 37,275 \text{ lbs}$$

TITLE			PAGE	
Reactor Ve	ssel Internals Lift	Riq	129 06	48
CGE	5. C. Pedus 3	8 Rich	DAYE CHK'O. BY	DATE
UCG-27694	CALC. NO.	PLE NO.	GROUP	

SPREADER ARM TO - MAT'L. ASTM A-36 STEEL

[FROM AISC MANUAL OF STEEL CONSTRUCTION 7 TH ED, P 3-41]

Let L = 74.63 (2 cos 30°) = 129.26 in.

= 11.0 ft.

Let KE = Effective Length factor

= 1 for a pin-pin Beam

. . EFFECTIVE LENGTH = LKE= 10.8 ft.

From the AISC Table

Pallowable = 205,000 lb. For Fyield = 36,000 PSI

A = 10.83 in2

fallowable = Pallowable / A

Fallowable = 205,000/10.83

Fallowable = 18,929 PSI

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

TITLE			PAGE	
Reactor Ve	ssel Internals Lift	Rig	130	of 48
CGE	Fr. C. Edura 3/	183 & Richal	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	FILE NO.	GROUP	

COMPRESSIVE STRESS IN SPREADER ARM

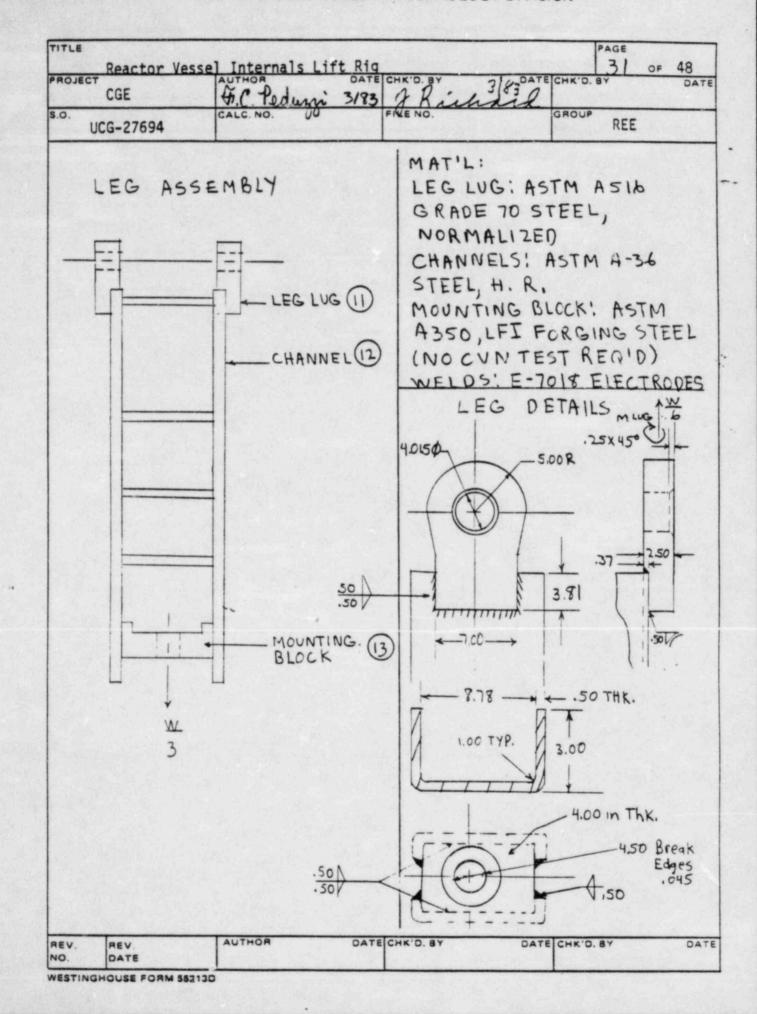
FH P FH
$$2 P \cos 30^\circ = FH$$

$$P = \frac{EH}{2}$$

$$\cos 30$$

$$P = 64,560/2 = 37,275 165$$

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				
			the state of the s		



Reactor Ve	ssel Internals Lift	Rig	PAGE 32 OF	48
CGE	F.C. Peduni 3	183 2 Hist	3/83 OATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	FILE NO.	GROUP	

LEG LUG (I)

MIEG MIEG

TENSION 8 4.015 \$ Hole

MLEG = 49,066 in-16

$$ft = P$$
, $P = W = 230,000 = 38,333 lbs.$
At 6

At= 2.5(10.00-4.015) -. 252 = 14,90 in2

$$f = \frac{W16}{14.90} = \frac{38,333}{14.90} = 2573 \text{ PSI}$$

BENDING @ 4.015 \$ Hole

$$fb = MC$$
, $I = bh3 = (10.00-4.015)(2.5)3 = 7.7931n4$

$$fb = 49,06612.512) = 7870 PSI$$
7.793

-					
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				
		A STATE OF THE PARTY OF THE PAR			

Reactor Ve	ssel Internals Lift	Ria	PAGE 23 OF	48
CGE	F.C. Podusa 3	DATE CHK'D. BY	SO DATE CHK'D. BY	DATE
UCG-27694	CALC. NO. U)	PLE NO.	GROUP	

COMBINED MAX. BENDING AND TENSION & 4.015 \$ HOIE

ftcomb = fb+ft

ftcomb = 1870 + 2573 = 10,443 PSI

BEARING

The bearing stress is calculated in the spreader soint calculations and is

Fcmax = 4,259 + 14,538 = 18,797 PSI

		المتراجين المتراج والمتراوي			
REV.	REV.	AUTHOR	DATE CHY'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

TITLE				PAGE	
Reactor Ve	ssel Internals Lift	Rig		34 OF	48
CGE	AUTHOR	183 Of Rich	3 23 DATE CHK	D. 8Y	DATE
ucg-27694	CALC. NO.	FILE NO.	GROU	REE	

CHECK OF EXTREME FIBER STRESS TO AISC CRITERIA COMBINED TENSION AND BENDING

Combined stresses - Section 5-1,6 For Lower clevis 6

fa = 4,022 PSI fb = 17,835 PSI Fy = 105,000 PSI

For spreader Lug (8)

fq = 2,157 PSI fb = 13,260 PSIFy = 38,000 PSI

For Leg Lug 1

fa = 2573 PSI fb = 7870 PSIFy = 38,000

fa = computed axial stress

fb = computed compressive bending

Fb = compressive bending stress that would be permitted if bending moment alone existed.

Fa = axial stress that would be permitted if axial force alone existed

Section 1.6.2 Axial Tension & Bending governs, ... Formula (1.6-16) governs

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rfq	PAGE 35 or	F 48
CGE	F.C. Peduni 3	18 C Pich	173 DATE CHK'D. BY	DATE
ucg-27694	CALC. NO. UD	EILE'NO.	GROUP REE	

For ALL Three members with combined loads the cross-section is solid so Fb = .75 Fy per Sec 1.5.1.4.3

Lower clevis 6

$$\frac{fq}{.6F_{Y}} + \frac{fb}{.75Fb} + 0 = \frac{4,027}{.6(105,000)} + \frac{17,835}{.75(105,000)} = .290$$

. 290 4 1.0

spreader Lug 3

$$\frac{fg}{.bFy} + \frac{fb}{.75Fb} + 0 = \frac{2,157}{.b(37,000)} + \frac{13,260}{.75(38,000)} = .560$$

.560 4 1.0

LEG LUG 1

$$\frac{fo}{.6Fy}$$
 + $\frac{fb}{.75Fb}$ + $0 = \frac{2573}{.6(38,000)}$ + $\frac{7570}{.75(37,000)}$ = .389

.389 ≤ 1.0

REF: MANUAL OF STEEL CONSTRUCTION

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE L IK'D. BY	DATE
NO.	DATE				

TITLE			PAGE	English mile
Reactor Ve	ssel Internals Lift	Riq	36	of 48
CGE	F.C. Pedurai 3	183 O Richar	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO. UU	FORE NO.	GROUP REE	

LEG LUG WELD

REFERENCE: LINCOLN ELECTRIC COMPANY, SOLUTION TO DESIGN OF WELDMENTS D 810-17 PAGE 3 PMLES

weld, is assumed equivalent to the generically weaker fillet weld for simplicity, and is width 7.00" it is conservatively assumed that the 3.81 has a throat width is .37, use the smaller of .50 (.707), so the weld may be modeled by a simple case50(.707)=.3535 < .37

$$5w = \frac{d}{3}(3b+d)$$

= $\frac{3.81}{3}(3(7)+3.81) = 31.509$

5 = 31,509(.3535) = 11.138 in3

Fubending = Mleg = 49,066 = 4405 PSI S 11.138

fushear force = P/At , P=W/6, A=[217)+23.71)].3535=7.641n² fushear Force = 230,000/6 = 5017 PSI 7.64

For fillet welds, both stresses are treated as shear stresses, so fucomb max = fubending + fushear force

f v combmax = 4405 +5017 = 9422 PSI

REV.	REV.	AUTHOR	DATE CHK'O. BY	DATE CHK'D. BY	DATE
NO.	DATE				
_					

Reactor Ve	ssel Internals Lift	Rig	PAGE	F 48
CGE	G.C. Podus: 3	73 1 Richa	DATE CHK'D. BY	DATE
ucg-27694	CALC. NO.	FILE NO.	GROUP REE	

LEG CHANNELS 12

MATIL. ASTM A-36 STEEL H.R.

calculation of cross-sectional area of Leg Channels

For one channel

$$A = [8.78 - 2(1.00)](.50) + 2[3.00 - 1.00 - .5](.50) + 4[2)(3.0^{2} - 2.0^{2})$$

A = 6.583 in2

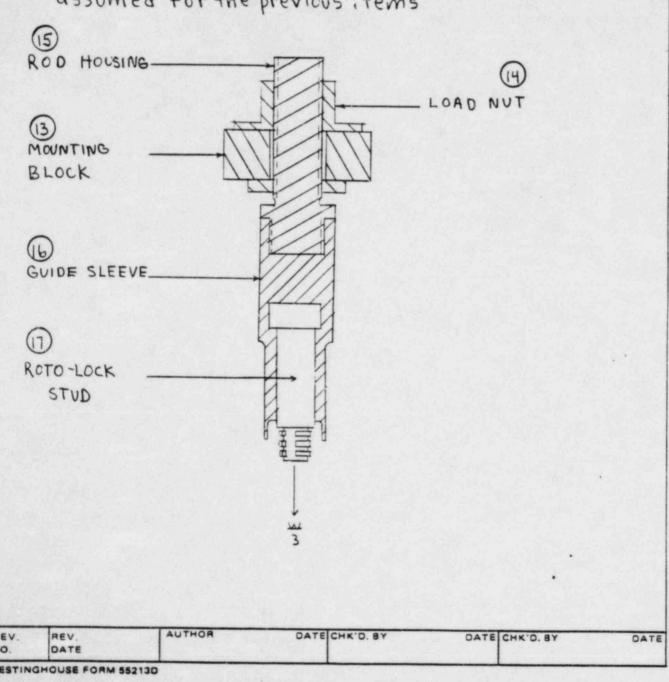
TENSION IN LEGS

At = ACHANNEL = 6.853 in2

Reactor Ve	ssel Internals Lift	Rig	PAGE 39	OF 48
CGE	F.C. Pedusi	3/73 & Rich 3/	DATE CHK'D. BY	DATE
ucg-27694	CALC. NO.	CILENO.	GROUP REE	

ROTO-LOCK ASSEMBLY

The ROTO - LOCK ASSEMBLY sees only the internals weight w = 202,000 lb. so the remaining items will use this design weight instead of the weight assumed for the previous items



Reactor Ve	ssel Internals Lift	Rig	PAGE 39 OF	48
CGE	A.C. Podusa 3/	83 & Richa	83 Q DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	(LE'NO.	GROUP REE	

MOUNTING BLOCK 3

MATIL. ASTM A-350 LFI FORGING STEEL (NO CUNTEST REQID) WELDS: E 70-18 ELECTRODES

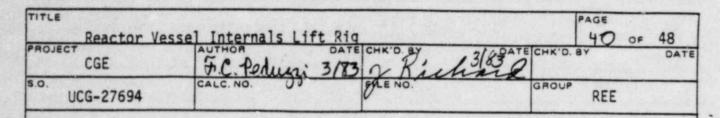
BEARING OF LOAD NUT TO MOUNTING BLOCK

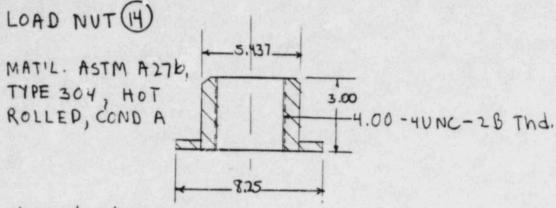
SHEAR IN MOUNTING BLOCK WEIDS

$$Fv = 202,000/3 = 3087 PSI$$

21.81

REV.	REV.	AUTHUR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				





Thread Shear

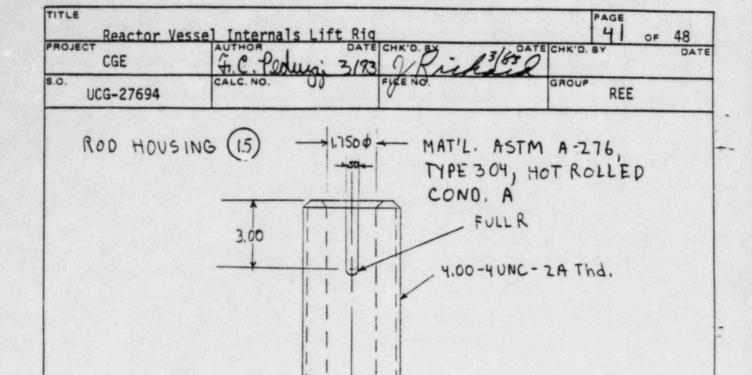
For a 4. 4 unc-2B THO, FROM MARKS HANDBOOK, Dpitch = 3.8376 in, L = 3.00 inAv = $\pi (3.8376)(3) - 3.25(.5) = 16.459 \text{ in}^2$

Bearing of Load Nut to Mounting Block

The bearing stress is calculated for the mounting Block and is

* Subtracted for slot cut for locking tab in red housing

REV.	REV.	AUTHOR	DATE CHK'D. BY	CATE CHK'D, BY	DATE
NO.	DATE				



3.65 \$.50

5.00-6UN-2A Thd.

4.76 \$

Thread shear on upper 4,00-4UNC-2A-Thd
The upper threads shear stress is the same as
the Icad Nut's

$$fv = \frac{202,000/3}{16.459} = 4091 PSI$$

3.00

Thread shear on Lower 5.00 - 6UN - 2 A THOS.

$$fv = P = \frac{W}{Av}$$
, $Av = \pi D pitch 2/2$

REV.	REV.	AUTHOR	DATE CHK'D. UY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rig	PAGE 47 or	F 48
CGE		SAB Parla	CATE CHK'D. BY	DATE
ucg-27694	CALC. NO.	BOLE NO.	GROUP REE	

FOR 5.00-6UN-ZA THD
(MARK'S 8-10) D pitch =
$$0 - 3\sqrt{3} = 5 - 3\sqrt{3}$$

8N 8(6)
Dpitch = 4.8917 in
 $l = 3.00 - .50 = 2.50$ in
 $Av = \pi (4.8917)(2.50) = 19.210$ in²

$$fv = \frac{p}{Av} = \frac{202,000/3}{Av} = 3505 PSI$$

TENSION @ THREAD RELIEF

$$ft = P = \frac{V/3}{At}$$
, $At = \frac{\pi}{4}(40^2 - di^2) = \frac{\pi}{4}(3.65^2 - 1.75^2) = 7.058 \text{ in}^2$

$$F = \frac{207,000/3}{8.058} = 8356 PSI$$

PERSONAL PROPERTY AND ADDRESS OF THE PERSONAL PR	THE RESERVE OF THE PERSON NAMED IN COLUMN TWO IS NOT THE PERSON NAMED IN COLUMN TWO IS NOT THE PERSON NAMED IN				
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				
	and the second s				

Reactor Vessel Internals Lift Riq

PROJECT
CGE

AUTHOR

CALC. NO.

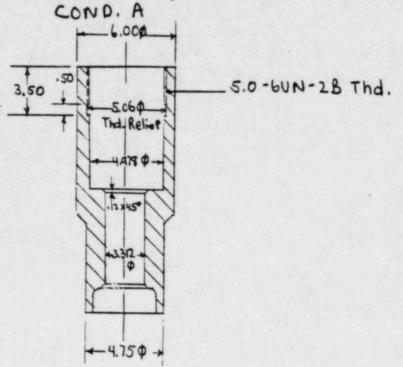
CALC. NO.

PAGE
43 OF 48

PAGE
45 OF 48

PAG

GUIDE SLEEVE 16 MATIL. ASTM A-276, TYPE 304 SST, HOT ROLLED, ANNEALED, AND PICKLED,



Thread Shear

The thread shear is the same as the lower thd. shear on the rod housing

$$fv = P = \frac{1}{4} = \frac{202,00013}{4} = 3505 PSI$$

TENSION @ THO RELIEF

$$ft = \frac{P}{At}$$
, $P = \frac{W}{3}$, $At = \frac{T}{4}(6.00^2 - 5.06^2) = 8.165 \text{ In}^2$

-	-	THE PARTY OF THE P			
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rig	PAGE 44 or	48
CGE	F.C. Peduni 3	183 Richis	DATE CHK'O. GY	DATE
ucg-27694	CALC. NO.	ME'NO.	GAOUP REE	

Bearing of Guide Sleeve to Roto-lock Stud

$$dc = 3.312 + 2(.12) = 3.552 \text{ in.}$$

 $do = 4.790 + -2(.09) = 4.61 \text{ in.}$

-					
REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Peactor Ves	sel Internals Lif	t Ria	PAGE 4.5 of 4
CGE	G. C. Poducai	2/83 Priha	3 BOATE CHK'D. BY
UCG-27694	CALC. NO.	FILE NO.	GROUP REE
ROTO - LO	OCK STUD (7))	
MAT'L.	ASTM AS64, 1	TYPE 630, 17-4	PRECIPITATION
			REATED AT 1100°F
	- 4	790 ¢	
			-0.
		HILL OG X	45° chamfer
		T	
	(-)	3.125	
	A	1 1 7	
	一一	2.095	
	. म्यूनिय		
	' \(\daggregar)	38 378	
	_	27190	
	\		
	EU O FUE	1.18	
		1 14	
		3/1	
	1		
REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY

Reactor Ve	ssel Internals Lift	Rig	PAGE 46 OF	48
CGE	AUTHOR	183 & Rich	A PATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	FILE NO!	GROUP REE	

N = NUMBER OF LANDS = 9

TENSION @ SECTION A-A

ft = P, P = W/3, $At = \Pi d^2/4 = \Pi (2.095)^2/4 = 3.447 In^2$

 $ft = \frac{W/3}{At} = \frac{202,000/3}{3.447} = 19,534 PSI$

SHEAR OF STUD LANDS

fv=P, A = Lcd

Lc = Length of Lands d = Depth of Lands

 $LC = 54 \pi (2.095) \times N = 8.885 d = 719 in$

A = (8.885)(.719) = 6.388 in2

fv 202,000/3 = 10,541 PSI

BEARING STRESS ON STUD HEAD

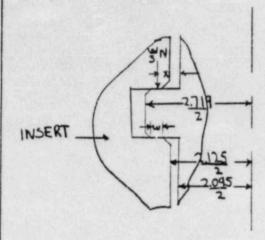
The head's bearing stress is calculated & the guide sLeeve.

fc = 202,000/3 9,928 PSI
6.782

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				

Reactor Ve	ssel Internals Lift	Rig	PAGE 47 OF	48
CGE	F. C. Pedugi 3	0ATE CHK'D. BY	DATE CHK'D. BY	DATE
UCG-27694	CALC. NO.	FILE NO.	GROUP REE	

BENDING IN LANDS



Chamfers , 03 x 45°

w = width of bearing surface w = 2.719 - .03 - (2.125 + .03) = .237 in.2

 $X = 2 \frac{119}{2} = .03 - 2 \frac{095}{2} = W = moment arm of force$

$$x = \frac{2.719}{2} = .03 = \frac{2.795}{2} = .1635$$
 in

fb = M/2 $2 \text{ Land} = Lcd^2/6$ $= 54 \pi (2.095)(.719)^2/6 = .085 \text{ in}^3$ 360

M = Bending Moment

REV.	REV.	AUTHOR	DATE CHK'D. BY	DATE CHK'D. BY	DATE
NO.	DATE				
-					

TITLE				PAGE	
Beactur Ve	ssel Internals Lift	Riq		148 OF	48
CGE	F.C Pedusa: 3	183 Priha	STE CHK'D	. 8Y	DATE
ucg-27694	CALC. NO. UU	OLEND.	GROUP	REE	

COMPRESSIVE BEARING STRESS ON LAND SURFACES

$$Fc = P = \frac{W/3}{A'c} = \frac{202,000/3}{2.263}$$