NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (7-77). LICENSEE EVENT REPORT EXHIBIT A CONTROL BLOCK: 1\_1\_1\_1\_1\_1\_1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0111 REPORT I L 16 101510101013111317 1014115181018 10181013181419 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 0111 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10 [Surveillance testing of fire and smoke detectors per Technical Specification (T.S.) 4.19 was not completed 10121 within the surveillance interval including the ±25% tolerance. All testing outside of the reactor building 10131 10141 Iwas completed during the interval, however, testing within the reactor building was not completed. 10151 Iwas shutdown because of unrelated events and the sesting was completed per T.S. requirements. All detector 10161 Ichecked operable. This occurrence is reportable per T.S. 6.12.3.2.c. 0181 SYSTEM CAUSE CAUSE COMP VALVE CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE 0 9 1 A B 111 1 D |12 Z |13 | Z |15 | Z |16 18 SEQUENTIAL OCCURRENCE REPORT LER/RO | EVENT YEAR REVISION REPORT NO. CODE 17 REPORT | 18 | 0 | NUMBER | 21 22 TYPE NO  $\frac{10}{28}$   $\frac{31}{29}$ 111 0 1 1 1 4 | X | 31 ACTION FUTURE EFFECT SHUTDOWN NPRD-4 ATTACHMENT PRIME COMP. COMPONENT TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB SUPPLIER 1<u>Z</u>120 35 MANUFACTURER 1<u>X</u>|18 1 2 119  $\frac{1010101010122}{37}$ I\_Z |21 1 <u>Z</u> 125 1<u>Z 91919</u>126 1 N 124 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27 [The testing within the reactor building could not be completed within the T.S. surveillance time interval 11101 11111 Ibecause radiation levels were too high to perform testing and remain consistent within ALARA radiation lexposure considerations. The detectors tested satisfactorily at the next cold shutdown. A T.S. change lwas implemented in Amendment 54 of the licensee/T.S. which became effective on 3/23/81. 111 3 1 This change provides I for testing of the detectors inside the reactor building during each cold shutdown exceeding (cont'd) 1 | 4 | 80 FACILITY METHOD OF STATUS % POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION 9 10 12 129 1 NA 9 10 12 13 1151 130 | <u>B</u> |31 | Surveillance Test 4 45 46 132 44 80 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE  $\frac{1}{9} \frac{1}{10} \frac{1}{10} \frac{1}{11} \frac{1}{11}$ 1 6 1 NA 135 1 NA 44 45 136 PERSONNEL EXPOSURES 80 NUMBER TYPE DESCRIPTION 139 PERSONNEL INJURIES 80 NUMBER DESCRIPTION 9 10 10 140 1 NA 9 11 12 141 LOSS OF OR DAMAGE TO FACILITY 80 TYPE DESCRIPTION 1 9 1 1\_Z |42 | NA 143 PUBLICITY ISSUED DESCRIPTION NRC USE ONLY 1 N 144 1 9 10 2101 NA NAME OF PREPARER: Patrick Rogers PHONE: (501) 964-3100

8408130326 840803 PDR ADDCK 05000313 5 PDR .

LICENSEE EVENT REPORT

EXHIBIT A

| LER No. 50-313/80-014/03X-1

Occurrence Date: 04/15/80

Cause Description and Corrective Actions (Continued)

24 hours unless performed during the previous 6 months. It also requires testing during each refueling outage.



ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

August 3, 1984

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U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

> Subject: Arkansas Nuclear One - Unit 1 Docket No. 50-313 License No. DPR-51 Licensee Event Report No. 80-014/03X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.c, attached is the subject report concerning failure to complete fire and smoke detector testing in the reactor building within the time period required by Technical Specifications. This is a revision to a previous submittal dated May 12, 1980.

Very truly yours,

John R. Marshall Manager, Licensing

JRM: RJS: ac

Attachment

cc: Mr. Richard C. DeYoung Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission Washington, DC 20555

> Mr. Norman M. Haller, Director Office of Management & Program Analysis U. S. Nuclear Regulatory Commission Washington, DC 20555

IE-22 1/1