

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9	PAGE (3) 1 OF 2
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TITLE (4)  
Reactor Not Vented With Vessel Temperature Less Than Pressurization Temperature

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																				
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																		
07	11	84	84	006	08	03	84				0 5 0 0 0																																		
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) N</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="6">POWER LEVEL (10) 0.010</td> <td><input type="checkbox"/> 20.402(b)</td> <td><input type="checkbox"/> 20.406(c)</td> <td><input type="checkbox"/> 50.73(a)(2)(iv)</td> <td><input type="checkbox"/> 73.71(b)</td> </tr> <tr> <td><input type="checkbox"/> 20.406(a)(1)(i)</td> <td><input type="checkbox"/> 50.36(e)(1)</td> <td><input type="checkbox"/> 50.73(a)(2)(v)</td> <td><input type="checkbox"/> 73.71(c)</td> </tr> <tr> <td><input type="checkbox"/> 20.403(a)(1)(ii)</td> <td><input type="checkbox"/> 50.36(e)(2)</td> <td><input type="checkbox"/> 50.73(a)(2)(vii)</td> <td rowspan="4">OTHER (Specify in Abstract below and in Text, NRC Form 365A)</td> </tr> <tr> <td><input type="checkbox"/> 20.406(a)(1)(iii)</td> <td><input checked="" type="checkbox"/> 50.73(a)(2)(i)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(A)</td> </tr> <tr> <td><input type="checkbox"/> 20.406(a)(1)(iv)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)</td> <td><input type="checkbox"/> 50.73(a)(2)(viii)(B)</td> </tr> <tr> <td><input type="checkbox"/> 20.406(a)(1)(v)</td> <td><input type="checkbox"/> 50.73(a)(2)(iii)</td> <td><input type="checkbox"/> 50.73(a)(2)(x)</td> </tr> </table>												OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)											POWER LEVEL (10) 0.010	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(e)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)	<input type="checkbox"/> 20.403(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)	<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)
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LICENSEE CONTACT FOR THIS LER (12)

NAME Tim Wojtulewicz	TELEPHONE NUMBER (X-523)
	AREA CODE 8 1 5 9 4   2 - 2   9   2   0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
A									

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

With the reactor locked in shutdown since early April, 1984, for turbine work, the shift Control Room Engineer observed that the reactor vessel was not vented at less than 149°F. Reactor pressure was 0 psig, and reactor vessel level was +308 inches at the time of this observation.

This event was the first violation of the Tech Spec (3.5.B.1) venting requirement at Dresden.

Upon discovery of the violation, reactor metal temperature was increased.

This event was of minimal safety significance, since no control rod drive pump was running, and one condensate pump was running. Both recirculation pumps were running to maintain reactor water temperature between 130° to 149°F. If the reactor vessel was inadvertently pressurized, reactor temperature was above the 100°F minimum temperature required for an inservice pressure test. Operating personnel will be reminded of procedure requirement during the Operator training week. Possibility of Tech Spec change is being investigated to prevent recurrence.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Dresden Nuclear Power Station Unit 3	DOCKET NUMBER (2)  0   5   0   0   0   2   4   9	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8   4	-   0   0   6	-   0   1	0   2	OF 0   2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

With the reactor vessel locked in shutdown the shift Control Room Engineer observed that the reactor vessel was not vented at less than 149°F. This event was a violation of the venting requirement in Technical Specification 3.6.B.1. The reactor metal temperature was increased by increasing reactor water temperature.

The cause of this event was an inadvertant procedure violation. During shutdowns of long duration it is desirable to leave the head vent closed to prevent the introduction of additional contamination into the drywell.

All licensed operators will be reminded of the venting requirement when less than 149°F during the Operating training week.

This event was of minimal safety significance since no control rod drive pump was running, and one condensate pump was running. Both recirculation pumps were running to maintain reactor water temperature between 130° to 140°F. If the reactor vessel was inadvertently pressurized, reactor temperature was above the 100°F minimum temperature required for an inservice pressure test. Investigation into the basis for the Tech Spec requirement indicates that at conditions which existed, the 149°F is conservative. CECO Technical Services personnel indicated that no adverse effect occurred as a result of this event. Therefore, Dresden Station is investigating the feasibility of changing the venting requirement in Technical Specification 3.6.B.1 which would prevent recurrence.

This event is the first violation of the Technical Specification (3.6.B.1) venting requirement at Dresden.



**Commonwealth Edison**

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

August 7, 1984

DJS Ltr #84-780

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #84-006-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

D.J. Scott  
Station Superintendent  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III  
File/NRC  
File/Numerical

IE22  
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