

NRC FORM 366  
(12-81)  
10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION  
LICENSEE EVENT REPORT

APPROVED BY OMB  
3150-0011

CONTROL BLOCK: [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] [ ] ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 [ M A P P S 1 ] [ 2 ] [ 0 ] [ 0 ] [ - ] [ 0 ] [ 0 ] [ 0 ] [ 0 ] [ 0 ] [ - ] [ 0 ] [ 0 ] [ 3 ] [ 4 ] [ 1 ] [ 1 ] [ 1 ] [ 1 ] [ 4 ] [ 5 ]

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01 REPORT SOURCE [ L ] [ 6 ] [ 0 ] [ 5 ] [ 0 ] [ - ] [ 0 ] [ 2 ] [ 9 ] [ 3 ] [ 7 ] [ 0 ] [ 5 ] [ 0 ] [ 2 ] [ 7 ] [ 7 ] [ 8 ] [ 0 ] [ 7 ] [ 2 ] [ 0 ] [ 8 ] [ 4 ] [ 9 ]

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

02 On May 2, 1977, during plant heatup to operating temperature and pressure,  
03 Control Rod Drive 46-39 was withdrawn beyond the full-out position. "Rod  
04 Over-travel" and "Rod Drift" alarms were received indicating that the CRD was  
05 uncoupled from the control rod. (Refer to attachment for additional details.)  
06 LER 77-018/03L ("CRD 46-39")  
07  
08

09 SYSTEM CODE [ R ] [ B ] ⑪ CAUSE CODE [ X ] ⑫ CAUSE SUBCODE [ ] ⑬ COMPONENT CODE [ C ] [ R ] [ D ] [ R ] [ V ] [ E ] ⑭ COMP. SUBCODE [ Z ] ⑮ VALVE SUBCODE [ Z ] ⑯  
17 LER/RO REPORT NUMBER [ 7 ] [ 7 ] ⑰ EVENT YEAR [ ] [ ] ⑱ SEQUENTIAL REPORT NO. [ 0 ] [ 1 ] [ 8 ] ⑲ OCCURRENCE CODE [ / ] [ 0 ] [ 3 ] ⑳ REPORT TYPE [ X ] ㉑ REVISION NO. [ 1 ] ㉒  
ACTION TAKEN [ X ] ⑳ FUTURE ACTION [ C ] ㉓ EFFECT ON PLANT [ Z ] ㉔ SHUTDOWN METHOD [ Z ] ㉕ HOURS [ 0 ] [ 0 ] [ 0 ] [ 0 ] ㉖ ATTACHMENT SUBMITTED [ Y ] ㉗ NRC-4 FORM SUB. [ N ] ㉘ PRIME COMP. SUPPLIER [ N ] ㉙ COMPONENT MANUFACTURER [ G ] [ O ] [ 8 ] [ 0 ] ㉚

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳

10 CRD 46-39 was removed and replaced 9/1/77, during Refueling Outage Number 3.  
11 Disassembly revealed a badly bent uncoupling rod.  
12  
13  
14

15 FACILITY STATUS [ C ] ㉛ % POWER [ 0 ] [ 0 ] [ 3 ] ㉜ OTHER STATUS [ NA ] ㉝ METHOD OF DISCOVERY [ ] [ ] [ ] ㉞ DISCOVERY DESCRIPTION [ Operational Event ] ㉟

16 ACTIVITY CONTENT RELEASED [ Z ] ㊱ AMOUNT OF ACTIVITY [ NA ] ㊲ LOCATION OF RELEASE [ NA ] ㊳

17 PERSONNEL EXPOSURES NUMBER [ 0 ] [ 0 ] [ 0 ] ㊴ TYPE [ Z ] ㊵ DESCRIPTION [ NA ] ㊶

18 PERSONNEL INJURIES NUMBER [ 0 ] [ 0 ] [ 0 ] ㊷ DESCRIPTION [ NA ] ㊸

19 LOSS OF OR DAMAGE TO FACILITY TYPE [ Z ] ㊹ DESCRIPTION [ NA ] ㊺

20 PUBLICITY ISSUED [ ] [ ] [ ] ㊻ DESCRIPTION [ No press release by Boston Edison Co. ] ㊼

NAME OF PREPARER

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S PDR

ONE (617) 746-7900

IE22  
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UPDATE REPORT

PREVIOUS REPORT DATE - 6/02/77

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

Attachment to LER 77-018/03X-1

On May 2, 1977, during plant heatup to operating temperature and pressure, Control Rod Drive 46-39 was withdrawn beyond the full-out position. "Rod Overtravel" and "Rod Drift" alarms were received indicating that the CRD was uncoupled from the control rod.

After several unsuccessful attempts to couple the drive, CRD 46-39 was inserted to the full in position and electrically disarmed.

On 9/1/77, during Refueling Outage #3, CRD 46-39, Serial Number 1790 was removed and replaced with Serial Number 1222. Disassembly revealed a badly bent uncoupling rod. Functional scram and friction testing of the new drive (#1222) was accomplished.

BOSTON EDISON COMPANY  
800 BOYLSTON STREET  
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

July 20, 1984

BECo Ltr. #84-111

Dr. Thomas E. Murley  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

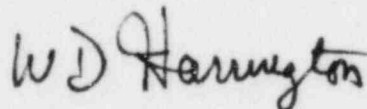
Docket Number 50-293  
License DPR-35

Dear Sir:

The attached update Licensee Event Report 77-018/03X-1, "CRD 46-39," is hereby submitted in accordance with the requirements of Pilgrim Nuclear Power Station Technical Specification 6.9.B.2.b.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

PH:caw

Enclosure: LER 77-018/03X-1

cc: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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