



ENTERGY

Entergy Operations, Inc.
PO Box 750
Fort Calicut, MS 39125
Tel 601 457 0408

February 18, 1992

W. T. Cottle
Vice President
Operations
Grand Gulf Nuclear Station

U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
Grand Gulf Nuclear Station Individual Plant Examination

GNRO-92/00010

Gentlemen:

In a letter dated October 31, 1989 (AECM-89/0200), System Energy Resources (now Entergy Operations, Inc.), submitted the methodology and a schedule for completion of the Grand Gulf Nuclear Station (GGNS) Individual Plant Examination (IPE) as specified in Generic Letter (GL) 88-20. In that submittal, it was anticipated that the GGNS IPE would be completed in June 1992.

The purpose of this letter is to inform you that the IPE completion date is now expected to be December 31, 1992. This schedule change is due to several factors. The original schedule was based on the opinion that the GGNS IPE would be very similar to the NUREG/CR-4550 analysis of GGNS. However, during the course of the GGNS IPE, this assumption of similarity has proven to be erroneous. At this time we estimate that approximately 1200 sequences will have to be evaluated for the GGNS IPE (261 sequences were evaluated for NUREG/CR-4550). This large number of sequences is due to the additional initiators which are being evaluated for GGNS and some differences in accident sequence modeling. While not all of these sequences will have to be quantified due to similarities to other sequences, as was done in NUREG/CR-4550, this work scope is still considerably larger than originally anticipated.

G9201161/SNLICFLR - 1

810045

9202240330 920218
PDR ADOCK 05000416
P PDR

AD11 / 10

February 18, 1992

GNRO-92/00010

Page 2 of 3

In the original response to GL 88-20, we indicated the containment performance analysis or Level 2 portion of the PRA would be performed using a methodology consistent with the general guidance given in Appendix 1 of GL 88-20. This method allows the use of a "template" approach where the utility can reference analyses performed for a similar plant in lieu of performing new plant specific analysis for their plant. This approach was thought to be appropriate for GGNS since GGNS was a reference plant for the NUREG-1150 analysis. However, since this initial decision was made, Entergy has become aware that most utilities are performing their own plant specific analyses (primarily with MAAP) or at least some verification of important sequences. In addition, we have also become concerned that the existing calculations may not adequately describe the important GGNS accident sequences since our results may differ from the NUREG/CR-4550 work. For these reasons, we are considering the development of a MAAP model for GGNS to use in our Level 2 analysis. We believe that this model will allow us to better understand the response of the plant in severe accident sequences and provide a product useful in future applications such as accident management.

Should you have any questions please contact James Owens at (601) 437-6483.

Yours truly,



WTC/JEO/mtc

cc: Mr. D. C. Hintz
Mr. J. L. Mathis
Mr. R. B. McGehee
Mr. N. S. Reynolds
Mr. H. L. Thomas

Mr. Stewart D. Ebnetter
Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30323

Mr. P. W. O'Connor, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Stop 13H3
Washington, D.C. 20555