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generator cavity. The load was set on a concrete support and the polar crane was placed in its parked position. To prevent similar events from occurring in the future, a procedure change has been made to RCS vent and leak test operating instructions to tag out the polar crane prior to exceeding 225 degrees Fahrenheit in the pressurizer.

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NRC Form 386A

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

ACILITY NAME (1)	DOCKET NUMBER (2)		LE	LER NUMBER (6)				PAGE (3)			
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On July 2, 1984 at 1400, the polar crane carried a skip box weighing approximately 250 pounds over part of the reactor coolant system when the fluid in the pressurizer was greater than 225 degrees Fahrenheit violating Technical Specification 2.11(1). When the violation was discovered, the load was suspended above grating over a steam generator cavity. The load was set on a concrete support and the polar crane was placed in its parked position. To prevent similar events from occurring in the future, a procedure change has been made to RCS vent and leak test operating instructions to tag out the polar crane prior to exceeding 225 degrees Fahrenheit in the pressurizer.

Loads are not to be allowed over the pressurized reactor coolant system to preclude dropping objects which could rupture the boundary of the reactor coolant system allowing loss of coolant and over-heating of the core. At the time of the incident the pressurizer was at approximately 388 degrees and 220 psia. The RCS was at approximately 185 degrees and being heated up with one reactor coolant pump in operation. Shutdown cooling was in operation with low pressure and high pressure safety injection pumps available. The load was 12 feet above the grating over the steam generator cavity. Had the load fallen, it is unlikely it would have penetrated the grating. Had this occurred and the RCS boundary been ruptured, the available safety injection system components would have been able to mitigate possible over-heating of the core.

## Omaha Public Power District 1623 Harney Omaha, Nebraska 68!02 402/536-4000

August 1, 1984 FC-397-84 LIC-84-252

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Reference: Docket No. 50-285

Gentlemen:

## Licensee Event Report for the Fort Calhoun Station

Please find attached Licensee Event Report 84-015 dated August 1, 1984. This report is being submitted per requirements of 10 CFR 50.73.

Sincerely,

andreus

R. L. Andrews Division Manager Nuclear Production

RLA/jmm

Attachment

cc: Mr. Richard P. Denise, Director Division of Resident, Reactor Project & Engineering Programs U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, Texas 76011

> INPO Records Center Mr. E. G. Tourigny, Project Manager

SARC Chairman PRC Chairman Mr. L. A. Yandell, Senior Resident Inspector Fort Calhoun File (2)

